

## 3150-0011

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

PHONE (205) 729-0885

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

May 25, 1983

Mr. James P. O'Reilly, Director  
U.S. Nuclear Regulatory Commission  
Suite 2900  
101 Marietta Street, NW  
Atlanta, Georgia 30303


Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET  
NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE  
REPORT BFRO-50-259/83023

The enclosed report provides details concerning indications of intergranular stress corrosion cracking in recirculation system pipe welds. This report is submitted in accordance with Browns Ferry unit 1 Technical Specification 6.7.2.a(3).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

  
H. J. Green  
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Records Center  
Institute of Nuclear Power Operations  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

USNRC REGION I  
ATLANTA, GEORGIA  
83 MAY 31 11:23

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 83023 Technical Specification Involved 3.6.G

Reported Under Technical Specification 6.7.2.a(3) \* Date Due NRC 5/26/83

Event Narrative:

Both units 2 and 3 were operating normally at 99% power. Unit 1 was in a refueling outage and was the only unit affected by this event. On 5/13/83, during performance of inspections required by IEB 83-02 on the recirculation piping, two crack-like indications were detected and confirmed by ultrasonic inspection on weld KR-1-37 (T.S. 3.6.G). One crack indication was approximately 19 inches long between 11:30 and 3:00 and the other crack was approximately 9 inches long between 7:00 and 8:30. The crack indications are on the cap side of the weld and have an estimated maximum of 35% and 30% through wall depth for the longest and shortest crack indications, respectively. A follow-up report will be submitted when the remaining welds that are required to be inspected per IEB 83-02 have been inspected and evaluated. The follow-up report will include the details of the corrective actions to be performed for weld KR-1-37 and any additional defects found. Verbal notification to NRC on a weekly basis will be made for any additional weld defects that are found before the follow-up report is submitted. There was no effect on the health and safety of the public. There are no redundant systems.

\* Previous Similar Events:

BFRO-50-260/8240

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

\*Revision: JRP