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REPORT SOURCE L 6 0 5 0 0 0 3 2 7 7 0 5 0 6 8 3 8 0 6 0 1 8 3 9

60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | Unit 1 in mode 1 at 100% Rx power with the temperature and pressure at 578 degrees F
0 3 | and 2235 psig. While changing out one upper head injection (UHI) level switch
0 4 | and recalibrating the other three to the newly established setpoint; it was
0 5 | discovered that three of four were out of tolerance. There was no effect upon
0 6 | public health and safety. Previous occurrences - ten (SQRO-50-327/82135, 83002,
0 7 | 83022, 83041, 83052, and SQRO-50-328/82137, 83013, 83023, 83044, and 83062).

08		789		SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE	
09		78		S F 11		E 12		E 13		I N S T R U 14				S 15		Z 16	
78		9		910		11		12		1318				19		20	
17		LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.					
83		2122		066		03		L		0							
78		2324		2526		2728		2930		3132							
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
E 18		E 19		Z 20		Z 21		0000 22		N 23		N 24		N 25		I 204	
3334		3536		3740		41		42		43		44					

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 1-LS-87-21, -22, -23 level switches were found out of tolerance as specified in

1 1 surveillance requirement 4.5.1.2.c.1. 1-LS-87-21 was replaced and all four level

1 2 switches were calibrated to the new setpoint and returned to service. The most

1 3 probable cause is instrument drift of the Barton Model No. 288A level switch. As

1 4 previously committed to, the switches will be checked in 30 days. Barton will be

1 5 requested to inspect the switches.

Requested to inspect the switch.

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION					
1	5	E	28	1	0	0	29	NA	30	B	31	Surveillance test	32
7	8	9	10	11	12	13	14	15	16	17	18	19	20
ACTIVITY CONTENT		RELEASED OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE							
1	6	Z	33	1	0	0	34	NA	35	36	37	38	39
7	8	9	10	11	12	13	14	15	16	17	18	19	20

PERSONNEL EXPOSURES										
NUMBER			TYPE	DESCRIPTION						
1	7	0	0	0	7	Z	38	NA		

PERSONNEL INJURIES		DESCRIPTION	
NUMBER	DESCRIPTION	NUMBER	DESCRIPTION
18	000	50	NA

7	8	9	11	12	80
		LOSS OF OR DAMAGE TO FACILITY			
		TYPE DESCRIPTION (43)			
19	2	(42)	NA		

8306070039 830601
 PDR ADOCK 05000327
 S PDR

Name of Preparer: W. C. Ludwig /M. R. Harding

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TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401
1750 Chestnut Street Tower II

June 1, 1983

83 JUN 3 P10:55
USNRC REGION 1
ATLANTA, GEORGIA

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

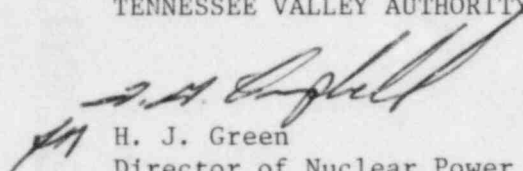
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET
NO. 50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE
REPORT SQRO-50-327/83066

The enclosed report provides details concerning the inoperability of the
upper head injection (UHI) system level switches. This report is submitted
in accordance with Sequoyah unit 1 Technical Specification 6.9.1.13.b.

Very truly yours,

TENNESSEE VALLEY AUTHORITY


H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

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Washington, D.C. 20555

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NRC Inspector, Sequoyah

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