

LICENSEE EVENT REPORT

CONTROL BLOCK

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 I L D R S 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

CONT

0 1 REPORT SOURCE L 6 0 5 0 0 0 2 3 7 7 0 5 0 5 8 3 3 0 5 2 0 8 3 9
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

0 2 While in the run mode, during the performance of DOS 500-3 APRM 4 flow biased rod
0 3 block setpoint was found to be 2.2 percent in excess of the Tech Spec limit.
0 4 Safety significance was considered minimal since APRM's 5 and 6 were operating
0 5 within limits. There was no effect on public health or safety. Previous
0 6 occurrence of this type was reported by R.O. 82-56/03L-0 on Docket 50-237.
0 7
0 8
0 9

0 9 SYSTEM CODE I A 11 CAUSE CODE E 12 CAUSE SUBCODE G 13 COMPONENT CODE I N S T R U 14 COMP SUBCODE Q 15 VALVE SUBCODE Z 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
EVENT YEAR 8 3 SEQUENTIAL REPORT NO. 0 3 8 OCCURRENCE CODE 0 3 REPORT TYPE L REVISION NO. 0
17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
ACTION TAKEN E 18 FUTURE ACTION Z 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0 0 0 0 ATTACHMENT SUBMITTED N 23 NPRD-4 FORM SUB Y 24 PRIME COMP. SUPPLIER N 25 COMPONENT MANUFACTURER G 0 8 0 0 26
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

1 0 The cause is attributed to instrument drift which accumulated during the refuel
1 1 outage time period. APRM 4 was adjusted per procedure DIS-700-6 and tested
1 2 satisfactorily using DOS 500-3. No further action was necessary since the
1 3 normal required weekly performance of DOS 500-3 would detect drift before Tech
1 4 Spec limits are reached.

1 5 FACILITY STATUS E 28 % POWER 0 5 3 29 OTHER STATUS N/A 30 METHOD OF DISCOVERY B 31 DISCOVERY DESCRIPTION Routine Surveillance 32
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
ACTIVITY CONTENT RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY N/A 35 LOCATION OF RELEASE N/A 36
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION N/A 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION N/A 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION N/A 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
PUBLICITY ISSUED N 44 DESCRIPTION N/A 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

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PDR ADOCK 05000237
S PDR

NRC USE ONLY

NAME OF PREPARER K. P. Beverly

PHONE (815) 942-2920 x526



Commonwealth Edison

DEVIATION REPORT

DVR NO. 12 - 2 - 83 - 73
STA UNIT YEAR NO.

PART 1		TITLE OF DEVIATION		OCCURRED	
		APRM Channel 4 Rod Block Set Non Conservative		5/5/83 0645	
SYSTEM AFFECTED 730		PLANT STATUS AT TIME OF EVENT		DATE TIME	
		MODE Run, PWR(MWT) 1331, LOAD(MWE) 410		TESTING <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	
DESCRIPTION OF EVENT During APRM Channel 4 Rod Block and Scram Test (DOS 500-3),					
APRM channel 4 rod block was found set at 69. Setpoint should be less					
than or equal to 66.80. Channel 4 was placed in bypass and repairs					
were initiated.					
				10 CFR50.72 NRC RED PHONE <input type="checkbox"/> <input checked="" type="checkbox"/>	
				NOTIFICATION MADE YES NO	
EQUIPMENT FAILURE D27667		WORK REQUEST NO.		RESPONSIBLE SUPERVISOR J. Gates	
<input checked="" type="checkbox"/> YES <input type="checkbox"/> NO				DATE 5/5/83	

PART 2	OPERATING ENGINEER'S COMMENTS
	The APRM 4 flow biased rod block was immediately readjusted
	and satisfactorily tested. APRM's 5 and 6 were operable.

<input type="checkbox"/> EVENT OF PUBLIC INTEREST	<input type="checkbox"/> 24-HOUR NRC NOTIFICATION REQ'D
<input type="checkbox"/> TECH. SPEC. VIOLATION	TELEPH N/A
<input type="checkbox"/> NON REPORTABLE OCCURRENCE	REGION III
<input type="checkbox"/> 14 DAY REPORTABLE/T.S.	DATE TIME
<input checked="" type="checkbox"/> 30 DAY REPORTABLE/T.S. 6.6.B.2.b	TELEGM/TELECOPY N/A
<input type="checkbox"/> ANNUAL/SPECL REPORT REQ'D	REGION III
	DATE TIME
A.I.R. #	<input type="checkbox"/> CECO CORPORATE NOTIFICATION MADE
L.E.R. # 83-38/03L-0	IF ABOVE NOTIFICATION IS PER 10CFR21
	<input type="checkbox"/> 5-DAY WRITTEN REPORT REQ'D PER 10CFR21
	Telecopy Dennis P. Galle
	CECO CORPORATE OFFICER
	5/5/83 1340
	DATE TIME

PRELIMINARY REPORT COMPLETED AND REVIEWED	Michael Wright	5/6/83
	OPERATING ENGINEER	DATE
INVESTIGATED REPORT & RESOLUTION ACCEPTED BY STATION REVIEW	J. Brunner	5/21/83
	5-23-83	
RESOLUTION APPROVED AND AUTHORIZED FOR DISTRIBUTION	P.M. Pappas	5/22/83
	STATION SUPERINTENDENT	DATE



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

DJS Ltr #83-504

May 20, 1983

James G. Keppler, Regional Administrator
Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reportable Occurrence Report #83-38/03L-0, Docket #050-237 is being submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(b), conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

DJS/kjl

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
U.S. NRC, Document Management Branch
File/NRC

D.J. Scott
D.J. Scott
Station Superintendent
Dresden Nuclear Power Station



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Morris, Illinois 60450
Telephone 815/942-2920

May 20, 1983

DJS Ltr #83-504

James G. Keppler, Regional Administrator
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