

**Florida
Power**
CORPORATION

September 17, 1983
#3F-0982-13
File: 3-0-26

Mr. Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Crystal River Unit 3
Docket No. 50-302
Operating License No. DPR-72
Status of NUREG- 0737 Items I.C.1, II.D.1.2, II.D.1.3,
II.K.3.30, II.K.3.31, III.A.1.2, and III.A.2.2
(Generic Letter 82-10)

Dear Mr. Eisenhut:

By letter dated June 4, 1982, Florida Power Corporation (FPC) responded to an NRC request of May 5, 1982, to provide an updated completion schedule for selected NUREG-0737 Items. Since that letter, more than three (3) months ago, we have had numerous conversations with your staff and, as requested, submit the following revised completion schedules for the subject Items.

Item I.C.1 - Guidance for the Evaluation and Development of Procedures for Transients and Accidents, Item III.A.1.2 - Upgrade Emergency Support Facilities; and Item III.A.2.2 - Improving Licensee Emergency Preparedness -- Long Term (Meteorological Data) are part of those Items encompassed by SECY 82-111, "Requirements for Emergency Response Capability", which was approved by the NRC Commissioners on July 16, 1982. Completion schedules for these Items will be submitted in accordance with that document.

Item II.D.1.2 - Performance Testing of Boiling-Water Reactor and Pressurized-Water Reactor Relief and Safety Valves (Relief Valve and Safety Valve Testing) was updated in our letter to you dated June 30, 1982. This Item will be further updated by November 1, 1982. This proposed date (vs. September 15, 1982) was discussed with our NRC Project Manager). In the interim, positive valve position indication has been installed on the relief and safety valves and operator training in the use of this indicator has been completed.

8209240212 820917
PDR ADOCK 05000302
P PDR

General Office 3201 Thirty-fourth Street South • P.O. Box 14042, St. Petersburg, Florida 33733 • 813-866-5151

A076

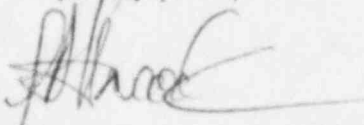
Mr. Darrell Eisenhut
September 17, 1982
Page 2

Item II.D.1.3 - Performance Testing of Boiling-Water Reactor and Pressurized-Water Reactor Relief and Safety Valves (Block-Valve Testing) is complete as stated in our letter to you dated June 30, 1982.

Item II.K.3.30 - Revised Small-Break Loss-of-Coolant-Accident Methods to Show Compliance with 10 CFR Part 50, Appendix K is being completed by the Analysis Subcommittee of the B&W Owners Group in conjunction with the Reactor Systems Branch of the NRC. Completion of this Item is predicated on NRC acceptance of the computer model being developed as a result of this effort; therefore, no current completion schedule can be estimated.

Item II.K.3.31 - Plant-Specific Calculations to Show Compliance With 10 CFR Part 50.46 will be completed within one year after staff approval of the LOCA analysis models from Item II.K.3.30, above.

Very truly yours,



J.A. Hancock
Vice President
Nuclear Operations

RMB/myf

cc: Mr. J.P. O'Reilly, Regional Administrator
Office of Inspection & Enforcement
U.S. Nuclear Regulatory Commission
101 Marietta St., Suite 3100
Atlanta, GA 30303