

OPERATING DATA REPORT

DOCKET NO. 50-315
 DATE 8/4/82
 COMPLETED BY A. MIGHT
 TELEPHONE (616) 465-5901

OPERATING STATUS

1. Unit Name: Donald C. Cook 1
2. Reporting Period: July 1982
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1089
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	5087	66,455
12. Number Of Hours Reactor Was Critical	50.4	3333.3	49,606.5
13. Reactor Reserve Shutdown Hours	0	0	463
14. Hours Generator On-Line	49.8	3301.6	48,523.8
15. Unit Reserve Shutdown Hours	0	0	321
16. Gross Thermal Energy Generated (MWH)	145,572	10,505,062	140,868,251
17. Gross Electrical Energy Generated (MWH)	46,390	3,453,420	46,335,200
18. Net Electrical Energy Generated (MWH)	44,588	3,332,395	44,573,073
19. Unit Service Factor	6.7	64.9	75.4
20. Unit Availability Factor	6.7	64.9	75.4
21. Unit Capacity Factor (Using MDC Net)	5.7	62.7	68.2
22. Unit Capacity Factor (Using DER Net)	5.7	62.2	64.8
23. Unit Forced Outage Rate	0	24.8	8.3
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):	Refueling outage July-August 1982		

25. If Shut Down At End Of Report Period. Estimated Date of Startup: August 28, 1982

25. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

(4/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-315
 UNIT 1
 DATE 8/4/82
 COMPLETED BY A. Might
 TELEPHONE (616)465-5901

MONTH July, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWE-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWE-Net)
1	<u>952</u>	17	<u>--</u>
2	<u>886</u>	18	<u>--</u>
3	<u>21</u>	19	<u>--</u>
4	<u>--</u>	20	<u>--</u>
5	<u>--</u>	21	<u>--</u>
6	<u>--</u>	22	<u>--</u>
7	<u>--</u>	23	<u>--</u>
8	<u>--</u>	24	<u>--</u>
9	<u>--</u>	25	<u>--</u>
10	<u>--</u>	26	<u>--</u>
11	<u>--</u>	27	<u>--</u>
12	<u>--</u>	28	<u>--</u>
13	<u>--</u>	29	<u>--</u>
14	<u>--</u>	30	<u>--</u>
15	<u>--</u>	31	<u>--</u>
16	<u>--</u>		

INSTRUCTIONS

On this format list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1982

DOCKET NO. 50-315
 UNIT NAME D.C. Cook - Unit 1
 DATE 8-10-82
 COMPLETED BY E.L. Townley
 TELEPHONE 616-465-5901
 PAGE 1 of 1

No	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
185	820703	S	694.2	B&C	1	N.A.	ZZ	ZZZZZZ	The Unit was removed from service at 0146 hours on 820703 for scheduled Cycle VI - VII refueling and maintenance outage, plus installation of several major design changes. The estimated duration of the outage is 57 days.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A Equipment Failure (Explain)
 B Maintenance or Test
 C Refueling
 D Regulatory Restriction
 E Operator Training & License Examination
 F Administrative
 G Operational Error (Explain)
 H Other (Explain)

³
 Method:
 1 Manual
 2 Manual Scram
 3 Automatic Scram
 4 Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG
 0161)

⁵
 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely¹. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation

¹Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT = Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved. If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following criteria:

- If a component failed, use the component directly involved.
- If not a component failure, use the related component, e.g., wrong valve operated through error; list valve as component.
- If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

Docket No.: 50-315
Unit Name: D. C. Cook Unit #1
Completed by: D. R. Campbell
Telephone: (616) 465-5901
Date: August 4, 1982
Page: 1 of 1

MONTHLY OPERATING ACTIVITIES - JULY, 1982

Highlights:

The Unit entered the reporting period at approximately 92% reactor power. The reactor core was in the end-of-life coast-down period. The Unit was removed from service at 0146 hours on July 3, 1982 to start our refueling outage. Present plans are to de-fuel the reactor in order to make repairs to IMO-128 (RCS letdown to RHR system).

We also intend to empty the refueling water storage tank to flush and clean it while we are de-fueled.

Total electrical generation for the month was 43,390 MWH.

Summary:

- 7-3-82 Unit removed from service for refueling outage.
- 7-4-82 Reached Mode 5.
- 7-7-82 Reactor Coolant System drained to half loop.
- 7-23-82 Reached Mode 6.

DOCKET NO.	50 - 315
UNIT NAME	D. C. Cook - Unit No. 1
DATE	8-10-82
COMPLETED BY	B. A. Svensson
TELEPHONE	(616) 465-5901
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MAJOR SAFETY-RELATED MAINTENANCE

JULY, 1982

- M-1 QRV-200, CVCS charging header regulating valve had a body-to-bonnet leak. Replaced the bonnet gasket and had the valve tested.
- M-2 Weld channel pressurization system containment isolation check valve, CA-181N, failed to meet type C leak rate test criteria. Valve internals were cleaned; valve was reassembled with new gasket, and valve was retested.
- M-3 The CD diesel jacket water safety valve, SV-16, was leaking by. Disassembled, cleaned, lapped seats and reassembled the valve. Had the valve reset.
- M-4 A socket weld on the west RHR spray header instrument line to flow instrument, IFI-331, developed a leak. Cut out the cracked weld and re-welded. Had necessary NDE performed.
- M-5 The east centrifugal charging pump lube oil filter differential pressure appeared high. Inspected entire lube oil system including the pump. Completed minor repairs. Reassembled and had the pump tested with acceptable differential pressure across the filter.
- M-6 Ice condenser air handler drain header containment isolation valves, DCR-610 and 611, failed to meet the type C leak rate test criteria. Replaced the valve diaphragms and had the valves retested.
- M-7 A leak developed in the instrument line to IFI-330, east RHR heat exchanger to the upper spray header. The tubing was broken off the downstream side of the root valve. Repaired by welding and had appropriate NDE performed.
- M-8 Auxiliary feedwater system safety valves, SV-141-E and W, were leaking by. Both valves were rebuilt and reinstalled.
- M-9 Component cooling water containment isolation check valve, CCW-243-72, failed to meet type C leak rate test criteria. Cleaned valve internals. Lapped the disc and seat, reassembled, and had the valve retested.
- M-10 SF-159, refueling cavity drain containment isolation valve failed to meet type C leak rate test criteria. Replaced the valve diaphragm and had valve retested.

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UNIT NAME	D. C. Cook - Unit No. 1
DATE	8-10-82
COMPLETED BY	B. A. Svensson
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PAGE	2 of 2

MAJOR SAFETY-RELATED MAINTENANCE

JULY, 1982

- M-11 RH-134, RHR system protection check valve had a body-to-bonnet leak. The flex gasket was replaced and the studs were inspected by QC to comply with NRC IE Bulletin 82-02.
- M-12 Reactor coolant drain tank vent header containment isolation check valve, N-160, failed to meet the type C leak rate test criteria. Lapped the valve seat and replaced the disc. Had the valve retested.
- M-13 The AB diesel generator aftercooler ESW regulating valve, WRV-721, was sticking partially open. Disassembled valve, replaced broken o-ring and bonnet gasket. Reassembled valve, had stroke set and tested.
- M-14 Inspected the controlled leakage seals on #12 reactor coolant pump. Replaced #1 seal insert, #2 seal runner, #3 seal ring and runner.
- C&I-1 Spray additive tank hydrogen sample return valve, ECR-10, was causing a "CD" battery circuit ground, even though the valve cycled correctly. The ground was located at the asco pilot valve solenoid coil for ECR-10. Electrical connections at this asco valve were retaped to eliminate the ground.
- C&I-2 Normal charging line isolation valve, QRV-62, indicated mid-position and could not be cycled. The valve stem coupling was found to be mal-positioned, causing the limit switch actuating arm to bypass the switch levers. Repositioning the coupling corrected the valve's operation.