

07/22/94

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PAGE 1 OF 1

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HOPE CREEK GENERATING STATION
EVENT CLASSIFICATION GUIDE
July 15, 1994

CHANGE PAGES FOR
REVISION #35

The Table of Contents forms a general guide to the current revision of the Hope Creek ECG. The changes that are made in this TOC Revision #35 are shown below. Please check that your revision packet is complete and remove the outdated material listed below:

ADD			REMOVE		
<u>Page</u>	<u>Description</u>	<u>Rev.</u>	<u>Page</u>	<u>Description</u>	<u>Rev.</u>
All	Table of Contents (TOC)	35	All	Table of Contents (TOC)	34
All	Sec. Sig. i-18	21	All	Sec. Sig. i-18	20
All	Sec. ii	2	All	Sec. ii	1
All	Sec. iii	7	All	Sec. iii	6
All	Sec. 1	6	All	Sec. 1	5
All	Sec. 6	5	All	Sec. 6	4
All	Sec. 17	6	All	Sec. 17	5

HOPE CREEK GENERATING STATION
EVENT CLASSIFICATION GUIDE
July 29, 1994

CHANGE PAGES FOR
REVISION #36

The Table of Contents forms a general guide to the current revision of the Hope Creek ECG. The changes that are made in this TOC Revision #36 are shown below. Please check that your revision packet is complete and remove the outdated material listed below:

ADD			REMOVE		
<u>Page</u>	<u>Description</u>	<u>Rev.</u>	<u>Page</u>	<u>Description</u>	<u>Rev.</u>
All	Table of Contents (TOC)	36	All	Table of Contents (TOC)	35
All	Sig. Att.	30	All	Sig. Att.	29
All	Att. 3	8	All	Att. 3	7
All	Att. 4	7	All	Att. 4	6

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EVENT CLASSIFICATION GUIDE
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July 15, 1994

<u>SECTION</u>	<u>TITLE</u>	<u>REV.</u>	<u>PAGES</u>	<u>EFFECTIVE DATES</u>
T.O.C.	Table of Contents	35	2	July 15, 1994
Sig. i-18	Section Identification/Signature Page	21	2	July 15, 1994
Sig. Att.	ECG Attachments/Signature Page	29	2	June 24, 1994
i.	Introduction	2	7	Jan 7, 1994
ii.	Cross Reference - Event to Requirement	2	10	July 15, 1994
iii.	Cross Reference - Attachment to Events	7	1	July 15, 1994
1.	REACTOR COOLANT LEAKAGE/LOCA	6	1	July 15, 1994
2.	STEAM BREAK OR SRV FAILED OPEN	0	2	May 26, 1989
3.	FAILURE TO SCRAM	2	1	Dec 21, 1992
4.	LOSS OF DECAY HEAT REMOVAL	2	1	Aug 21, 1992
	FUEL DAMAGE/DEGRADED CORE	3	2	Dec 21, 1992
6.	FISSION PRODUCT BOUNDARY FAILURE	5	1	July 15, 1994
7.	RADIOLOGICAL RELEASES/OCCURRENCES	5	5	Jan 28, 1994
8.	NON-RADIOACTIVE LEAK/SPILL (toxic gas, oil spill, hazmat)	3	2	Jan 7, 1994
9.	ELECTRICAL POWER FAILURE	3	2	Jan 7, 1994
10.	LOSS OF INSTRUMENTS/ALARMS/COMMUNICATIONS	4	2	May 10 1993
11.	CONTROL ROOM EVACUATION	0	1	May 26, 1989
12.	QUAKE/STORMS (earthquake, wind, floods, etc)	4	6	Jan 7, 1994
13.	SITE HAZARDS (aircraft crash, missiles, explosions, etc.)	1	5	Aug 21, 1992
14.	FIRE	3	1	Jan 7, 1994
15.	PERSONNEL EMERGENCIES/MEDICAL	3	2	Jan 7, 1994
16.	SECURITY EVENTS/FFD	5	3	Jan 7, 1994
17.	PUBLIC INTEREST ITEMS	6	3	July 15, 1994
18.	TECH SPECS/PLANT STATUS CHANGES	10	5	Jan 7, 1994

HOPE CREEK
EVENT CLASSIFICATION GUIDE
TABLE OF CONTENTS - (Continued)
July 15, 1994

<u>ATTACHMENT</u>	<u>TITLE</u>	<u>REV.</u>	<u>PAGES</u>	<u>EFFECTIVE DATE</u>
1.	Unusual Event	12	16	May 21, 1994
2.	Alert	7	7	May 21, 1994
3.	Site Area Emergency	7	7	May 21, 1994
4.	General Emergency	6	9	May 21, 1994
5.	Reserved			
6.	CM1 Log (UE/A/SAE)	18	10	May 21, 1994
7.	CM1 Log (GE)	18	10	May 21, 1994
8.	CM2 Log	13	15	Apr 29, 1994
9.	Non-Emergency Notifications Reference	17	3	June 24, 1994
10.	One Hour Report - NRC/Region	1	5	July 27, 1990
	One Hour Report - NRC/OPS (Security)	3	5	Sept 27, 1991
12.	One Hour Report - NRC/OPS	3	5	Apr 26, 1991
13.	Reserved			
14.	Four Hour Report - NRC/OPS	2	5	July 27, 1990
15.	Environmental Protection Plan	3	3	Sept 27, 1991
16.	Spill/Discharge Reporting	5	10	Jan 7, 1994
17.	Four Hour Report - Fatality/Medical	4	7	Apr 21, 1993
18.	Four Hour Report - Transportation Accident	1	6	July 27, 1990
19.	Twenty Four Hour Report - FFD	1	3	Sept 27, 1991
20.	Twenty Four Hour Report - NRC/OPS	2	5	July 27, 1990
21.	Reportable Event - LACT/MOU	0	2	May 26, 1989
22.	Other/Engineering	2	3	Sept 27, 1991
23.	Written Reports/LERS/Other	2	9	Jan 7, 1994

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ATTACHMENT 3

SITE AREA EMERGENCY

Table of Contents

	Page
I. Emergency Coordinator (EC) Log Sheet	2
II. Reporting	6
Initial Contact Message Form (ICMF)	7

I. EMERGENCY COORDINATOR LOG SHEET

Initials

INSTRUCTIONS

1. This is a permanent record.
2. Each step shall be initialed or marked N/A as appropriate.
3. Emergency Coordinator (EC) responsibility is

fulfilled by: _____

name

Title: _____

(SNSS/EDO/ERM)

- _____ A. **Declare a SITE AREA EMERGENCY.** Notify the Control Room staff and call the communicators to the Control Room.

EC

Initiating
ECG Section _____ Condition _____

Declared at _____ hrs on _____
time date

NOTE

If directed to implement this attachment due to a "Reduction" of the event, proceed to Section "C" and DO NOT implement Section "B".

B. NOTIFICATIONS

_____ EC

1. Check appropriate boxes and provide a brief description of the event on the INITIAL CONTACT MESSAGE FORM (ICMF) (pg 7 of this attachment). Complete, approve, and provide ICMF to the Designated Communicator (CM1).

B. NOTIFICATIONS (cont)

Initials

- _____
EC
2. Direct the Designated Communicator (CM1) to implement **Attachment 6** and make the notifications on the Communications Log within the time limits specified.
- _____
EC
3. Direct the Secondary Communicator (CM2) to implement **Attachment 8**.
- _____
EC
4. Notify the Salem SNSS (NETS X5127; 9-339-5200).
Direct the implementation of EPIP 101S, Section 3.2.

C. SUPPORT

EC

If not done previously, direct the OSC Coordinator to activate the OSC in accordance with EPIP 202H.

D. EMERGENCY PLAN IMPLEMENTATION

EC

If the EC is the EDO or SNSS, implement EPIP 103H, "Site Area Emergency."

OR

If the EC is the ERM, implement EPIP 401 and perform the following:

1. Notify the EDO of the change in Emergency Classification Level, the time of declaration, and direct the EDO to implement EPIP 103H, "Site Area Emergency."
2. Notify EOF staff of the change in emergency classification.

E. SECURITY

EC

For security event, notify the PSE&G Security Supervisor (X2222 to implement the Security Contingency Plan and Procedures.

NOTE

The Station Status Checklist shall be transmitted every 30 minutes or immediately if a significant change in station status occurs.

EC Log Sheet (Cont.)

Initials

F. TECHNICAL COMMUNICATIONS

EC

1. Upon receipt of the Station Status Checklist (SSCL) from the CM2, review and approve for transmittal. Implement more frequently for significant station status change.
2. Ensure completion and approval of the NRC Data Sheet form.
 - a) Obtain the form (both pages) from the CM2 (Att. 8)
 - b) Provide the approved form to the CM2 for transmittal to the NRC as soon as possible, but not to exceed ONE HOUR.

EC

NOTE:

As manpower permits, the Emergency Coordinator may assign an additional communicator (preferably an RO or SRO) to provide continuous updates to the NRC. The assignment of an additional communicator should not be made if personnel being considered are required to mitigate the event or to complete high priority Emergency Response functions.

EC

- c) Notify the NRC of any significant changes in Plant Status, Emergency Status, or any actions taken in accordance with 10CFR50.54(x).
- d) Direct CM2 to log or document (via NRC Data sheet) any additional information provided to the NRC. This includes, but is not limited to, changes in Plant Status, Emergency Status, or any actions taken in accordance with 10CFR50.54(x).
3. When turning over EC duties ensure your communicators are directed to turnover notifications responsibilities to the facility being activated.

EC Log Sheet (Cont.)

Initials

G. PRECAUTIONARY PROTECTIVE ACTION RECOMMENDATION (PAR)

EC

If a Precautionary PAR is developed with no escalation of the emergency status, THEN complete a new working copy ICMF (PAR Upgrade) for this Attachment and direct the CM1 to make new notifications with a clean copy of Attachment 6.

H. EMERGENCY ESCALATION

EC

If the event classification escalates above a SAE, THEN exit this attachment and implement a new attachment as directed by the classifying section EAL.

I. RELIEF/TURNOVER

EC

If relieved as EC prior to change in event classification, THEN turnover responsibility for this attachment to the oncoming EC and document your relief below and in EPIP 103H.

_____ assumed EC duties at _____ hrs.
name time

J. REPORTING

EC

Ensure that appropriate reports are made IAW Section II (page 6) of this Attachment.

K. RECORDS

EC

Ensure that all completed documents related to this Event are forwarded in accordance with reporting requirements of Section II of this Attachment.

II. REPORTING

Instructions

1. This is a permanent document - all pages of this Attachment.
2. Appropriate documents shall be appended to this form and the package expedited through all steps.
3. Responsible person shall initial each step.

Initials

SNSS

1. Ensure that an Incident Report is prepared.

SNSS

2. Forward this Attachment, the Incident Report, and any supporting documentation to the Operations Manager (OM).

OM

3. Review the Incident Report and any other relevant information for correct classification of event and corrective action taken.

OM

4. Contact the LER Coordinator (LERC) and request that the required reports be prepared. Provide this Attachment and any other supporting documentation to the LERC.

LERC

5. Prepare required reports. ECG Attachment 23 may be used as a guide for reporting requirements.

Report or LEK Number _____

LERC

6. When no longer required send this attachment and appended documents to the Emergency Preparedness Manager (EPM).

EPM

7. Forward this Attachment package to the Central Technical Document Room (CTDR) for microfilming.

INITIAL CONTACT MESSAGE FORM

I. THIS IS _____, COMMUNICATOR IN THE

CONTROL ROOM

TECHNICAL SUPPORT CENTER

EMERGENCY OPERATIONS CENTER

AT THE HOPE CREEK NUCLEAR GENERATING STATION.

☐ THIS IS NOTIFICATION OF A SITE AREA EMERGENCY WHICH WAS
DECLARED AT _____ ON _____
(TIME - 24 HOUR CLOCK) (DATE)

☐ THIS IS NOTIFICATION OF A PROTECTIVE ACTION RECOMMENDATION
UPGRADE WHICH WAS MADE AT _____ ON _____
(TIME - 24 HOUR CLOCK) (DATE)

II. ECG SECTION _____ INITIATING CONDITION _____

DESCRIPTION OF EVENT: _____

III. ☐ THERE IS NO RELEASE IN PROGRESS. }
☐ THERE IS A RELEASE IN PROGRESS. } see NOTE below for
release definition

33 FT. LEVEL WIND SPEED: _____ WIND DIRECTION (FROM): _____
(MPH) (DEGREES)

IV. ☐ NO PROTECTIVE ACTIONS ARE RECOMMENDED AT THIS TIME

☐ WE RECOMMEND EVACUATION AS FOLLOWS

	Sector(s)	Distance-Miles
	_____	_____
	_____	_____

☐ WE RECOMMEND SHELTERING AS FOLLOWS

	_____	_____
	_____	_____

EC Initials Time
(EC Approval to Transmit ICMF)

NOTE: Release is defined as: Plant Effluent > Tech Spec Limit of
1.20E+4 uCi/sec Noble Gas or 1.70E+1 uCi/sec I-131.

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ATTACHMENT 4

GENERAL EMERGENCY

Table of Contents

	Page
I. Emergency Coordinator (EC) Log Sheet	2
II. Reporting	6
Predetermined Protective Action Recommendations (flowchart)	7
Protective Action Recommendation Worksheet	8
Initial Contact Message Form (ICMF)	9

I. EMERGENCY COORDINATOR LOG SHEET

Initials

INSTRUCTIONS

1. This is a permanent record.
2. Each step shall be initialed or marked N/A as appropriate.
3. Emergency Coordinator (EC) responsibility is fulfilled by: _____

name

Title: _____

(SNSS/EDO/ERM)

- EC A. Declare a GENERAL EMERGENCY. Direct the Communicators to be prepared to make notifications.

ECG Section _____ Initiating Condition _____

Declared at _____ hrs on _____
time date

NOTE

A Protective Action Recommendation (PAR) shall be made on the Initial Contact Message Form (ICMF).

B. PROTECTIVE ACTION RECOMMENDATION (PAR)

- EC 1. Refer to page 7 of this attachment and choose the most appropriate Predetermined PAR. The worksheet (page 8) should be used to determine the affected downwind sectors.
- EC 2. If immediately available from the SRPT (RAC or RSM), obtain a Radiologically Based PAR for comparison.
- EC 3. Compare the Predetermined PAR and the Radiologically Based PAR and choose the most conservative for inclusion on the ICMF.

Initials

C. NOTIFICATIONS

- _____
EC
1. Check appropriate boxes and provide a brief description of the event on the INITIAL CONTACT MESSAGE FORM (ICMF) (pg 9 of this attachment). Complete, approve, and provide ICMF to the Designated Communicator (CM1).
 - _____
EC
 2. Direct the Designated Communicator (CM1) to implement **Attachment 7** and make the notifications on the Communications Log within the time limits specified.
 - _____
EC
 3. Direct the Secondary Communicator (CM2) to implement **Attachment 8**.
 - _____
EC
 4. Notify the Salem SNSS (NETS X5127; 9-339-5200).
Direct the implementation of EPIP 101S, Section 3.2.

D. SUPPORT

EC

If not done previously, direct the OSC Coordinator to activate the OSC in accordance with EPIP 202H.

E. EMERGENCY PLAN IMPLEMENTATION

EC

If the EC is the EDO or SNSS, implement EPIP 104H, "General Emergency."

OR

If the EC is the ERM, implement EPIP 401 and perform the following:

1. Notify the EDO of the change in Emergency Classification Level, the time of declaration, and direct the EDO to implement EPIP 104H, "General Emergency."
2. Notify EOF staff of the change in emergency classification.

F. SECURITY

EC

For security event, notify the PSE&G Security Supervisor (X2222) to implement the Security Contingency Plan and Procedures.

Initials

NOTE

The Station Status Checklist shall be transmitted every 30 minutes or immediately if a significant change in station status occurs.

G. TECHNICAL COMMUNICATIONS

EC

1. Upon receipt of the Station Status Checklist (SSCL) from the CM2, review and approve for transmittal. Implement more frequently for significant station status change.

EC

2. Ensure completion and approval of the NRC Data Sheet form.
 - a) Obtain the form (both pages) from the CM2 (Att. 8)
 - b) Provide the approved form to the CM2 for transmittal to the NRC as soon as possible, but not to exceed ONE HOUR,

NOTE:

As manpower permits, the Emergency Coordinator may assign an additional communicator (preferably an RO or SRO) to provide continuous updates to the NRC. The assignment of an additional communicator should not be made if personnel being considered are required to mitigate the event or to complete high priority Emergency Response functions.

- c) Notify the NRC of any significant changes in Plant Status, Emergency Status, or any actions taken in accordance with 10CFR50.54(x).
- d) Direct CM2 to log or document (via NRC Data Sheet) any additional information provided to the NRC. This includes, but is not limited to, changes in Plant Status, Emergency Status, or any actions taken in accordance with 10CFR50.54(x).

Initials

EC 3. When turning over EC duties ensure your communicators are directed to turnover notifications responsibilities to the facility being activated.

EC H. RELIEF/TURNOVER

If relieved as EC prior to de-escalation of the GE, then, turnover responsibility for this attachment to the oncoming EC and document your relief below and in EPIP 104H.

_____ assumed EC duties at _____ hrs.
name time

EC I. REPORTING

Ensure that appropriate reports are made IAW Section II (page 6) of this Attachment.

EC J. RECORDS

Ensure that all completed documents related to this Event are forwarded in accordance with reporting requirements of Section II of this Attachment.

II. REPORTING

Initials

Instructions

1. This is a permanent document - all pages of this Attachment.
2. Appropriate documents shall be appended to this form and the package expedited through all steps.
3. Responsible person shall initial each step.

SNSS

1. Ensure that an Incident Report is prepared.

SNSS

2. Forward this Attachment, the Incident Report, and any supporting documentation to the Operations Manager (OM).

OM

3. Review the Incident Report and any other relevant information for correct classification of event and corrective action taken.

OM

4. Contact the LER Coordinator (LERC) and request that the required reports be prepared. Provide this Attachment and any other supporting documentation to the LERC.

LERC

5. Prepare required reports. ECG Attachment 23 may be used as a guide for reporting requirements.

Report or LER Number _____

LERC

6. When no longer required send this attachment and appended documents to the Emergency Preparedness Manager (EPM).

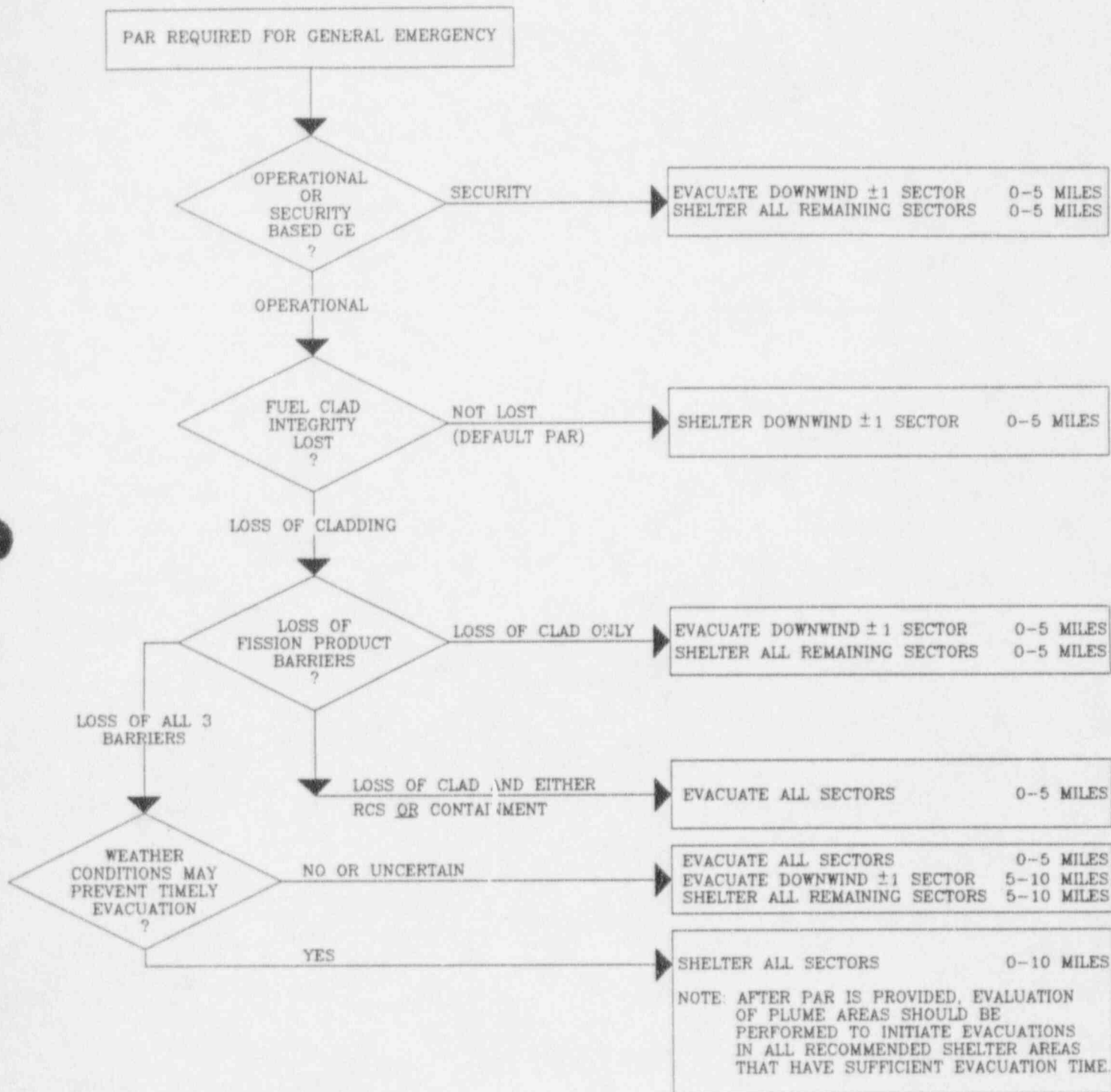
EPM

7. Forward this Attachment package to the Central Technical Document Room (CTDR) for microfilming.

PREDETERMINED PROTECTIVE ACTION RECOMMENDATIONS

ECG
ATT 4
Pg. 7 of 9

INITIAL CONDITIONS:



RECOMMENDED PROTECTIVE ACTIONS WORKSHEET

ECG
ATT 4
Pg. 8 of 9

WIND DIRECTION FROM

PAR AFFECTED SECTORS

DEGREES

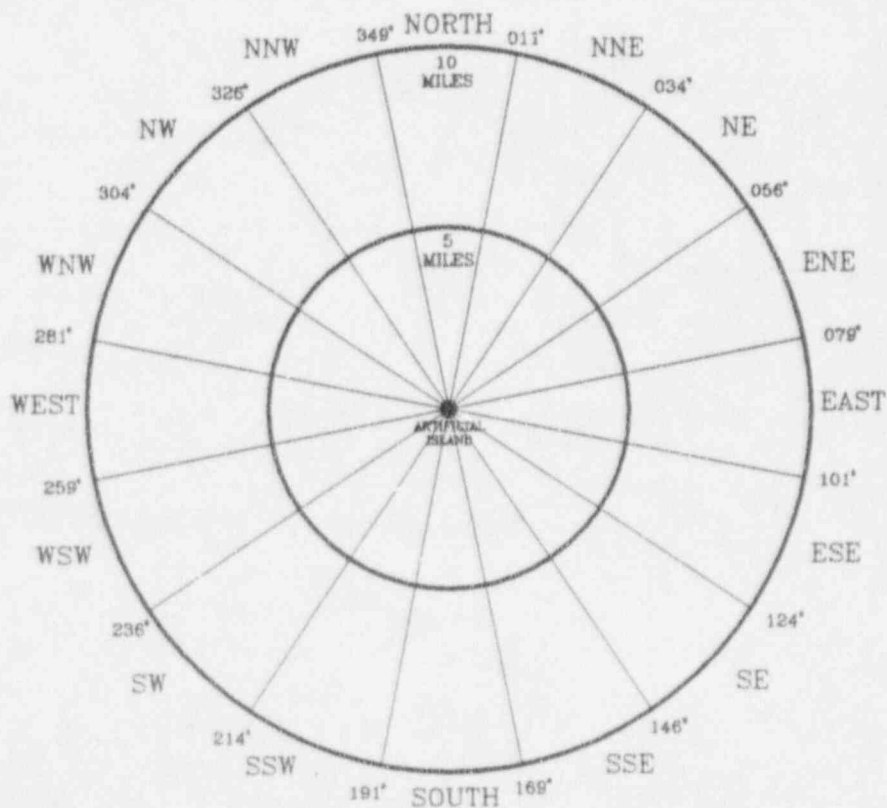
COMPASS



DOWNWIND ± 1 SECTOR

349 - 011	N	SSE	-	S	-	SSW
011 - 034	NNE	S	-	SSW	-	SW
034 - 056	NE	SSW	-	SW	-	WSW
056 - 079	ENE	SW	-	WSW	-	W
079 - 101	E	WSW	-	W	-	WNW
101 - 124	ESE	W	-	WNW	-	NW
124 - 146	SE	WNW	-	NW	-	NNW
146 - 169	SSE	NW	-	NNW	-	N
169 - 191	S	NNW	-	N	-	NNE
191 - 214	SSW	N	-	NNE	-	NE
214 - 236	SW	NNE	-	NE	-	ENE
236 - 259	WSW	NE	-	ENE	-	E
259 - 281	W	ENE	-	E	-	ESE
281 - 304	WNW	E	-	ESE	-	SE
304 - 326	NW	ESE	-	SE	-	SSE
326 - 349	NNW	SE	-	SSE	-	S

NOTE: CONSIDER ADDING A SECTOR TO THE PAR IF THE WIND DIRECTION (FROM) IS WITHIN $\pm 3^\circ$ OF A SECTOR DIVIDING LINE.



INITIAL CONTACT MESSAGE FORM

I. THIS IS _____, COMMUNICATOR IN THE
(NAME)

____ CONTROL ROOM
____ TECHNICAL SUPPORT CENTER
____ EMERGENCY OPERATIONS FACILITY

AT THE HOPE CREEK NUCLEAR GENERATING STATION.

☐ THIS IS NOTIFICATION OF A GENERAL EMERGENCY WHICH WAS
DECLARED AT _____ ON _____
(TIME - 24 HOUR CLOCK) (DATE)

☐ THIS IS NOTIFICATION OF A PROTECTIVE ACTION RECOMMENDATION
UPGRADE WHICH WAS MADE AT _____ ON _____
(TIME - 24 HOUR CLOCK) (DATE)

II. ECG SECTION _____ INITIATING CONDITION _____

DESCRIPTION OF EVENT: _____

III.

☐ THERE IS NO RELEASE IN PROGRESS. }
☐ THERE IS A RELEASE IN PROGRESS. } see NOTE below for
release definition

33 FT. LEVEL WIND SPEED: _____ WIND DIRECTION (FROM): _____
(MPH) (DEGREES)

IV.

Sector(s) Distance-Miles

☐ WE RECOMMEND EVACUATION AS FOLLOWS _____

☐ WE RECOMMEND SHELTERING AS FOLLOWS _____

EC Initials Time
(EC Approval to Transmit ICMF)

NOTE: Release is defined as: Plant Effluent > Tech Spec Limit of
1.20E+4 uCi/sec Noble Gas or 1.70E+1 uCi/sec I-131.

HOPE CREEK
EVENT CLASSIFICATION
CROSS REFERENCE
EVENT TO REQUIREMENT DOCUMENT
SECTION ii

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NOTE

This Section is not to be used for Event Classification. Refer to Sections 1 thru 18.

<u>Initiating Event/Condition</u>	<u>Reference*</u>
-----------------------------------	-------------------

1. Reactor Coolant Leakage Rate/LOCA

- | | |
|-------------------------------------------------------------------------------------------------------------|--------------------|
| A. Exceeding reactor coolant leak rate technical specification | UE5 |
| B. Reactor coolant leak rate greater than 50gpm | AL5 |
| C. Unisolable RCS leak outside containment | NUMARC,
Table 4 |
| D. Loss of coolant accident greater than makeup capacity | SA1 |
| E. Loss of coolant accident with failure of ECCS systems to perform, or with potential containment failure. | GE6b,C |

2. Steam Break or Safety/Relief Valve Failed Open

- | | |
|--------------------------------------------------------------------------------|-----|
| A. Failure of a safety/relief valve to close following a reduction of pressure | UE6 |
| B. Steam line break outside drywell | AL4 |
| C. Steam line break outside drywell with continuing leakage | SA4 |

* Unless otherwise identified, references are as outlined in Appendix I, NUREG 0654. Only initiating Event/Condition references for Emergency Classes UE -GE are listed here. Non -emergency references are included in the sections themselves.

NOTE
This Section is not to be used for Event
Classification. Refer to Sections 1 thru 18.

<u>Initiating Event/Condition</u>	<u>Reference*</u>
3. Failure to Scram	
A. Failure of the reactor protection system to automatically initiate and complete a scram which brings the reactor subcritical.	AL11
B. Conditions requiring operation of the standby liquid control system with subsequent failure to reduce power (continued power generation but no core damage evident).	SAS
C. Conditions requiring operation of the standby liquid control system with subsequent failure to reduce power (containment failure imminent with significant core damage).	GE6
4. Loss of Decay Heat Removal	
A. Complete loss of any decay heat removal capability needed for plant cold shutdown.	ALS AL10
B. Complete loss of any decay heat removal capability needed for plant hot shutdown.	SA8
C. Continued loss of all conventional means of decay heat removal with core damage and containment failure possible if cooling not restored.	GE6d
5. Fuel Damage/Degraded Core	
A. Fuel damage indications	UE3 a,b
B. Severe loss of fuel cladding	AL1a,b
C. Degraded core with indication of possible loss of coolable geometry	SA2
D. Fuel handling accident with radiological release to the Reactor Building from irradiated fuel.	AL12

NOTE
This Section is not to be used for Event
Classification. Refer to Sections 1 thru 18.

<u>Initiating Event/Condition</u>	<u>Reference*</u>
E. Major irradiated fuel damage with radiological release exceeding (or projected to exceed) the threshold dose rates at the MEA for a Site Area Emergency.	SA10
6. Fission Product Boundary Failures (2/3)	
A. Severe loss of fuel cladding	GE2
B. Loss of RPV integrity	GE2
C. Loss of primary containment	GE2
7. Radiological Releases	
A. Contaminated injured person transported from the site to an offsite medical facility.	UE 1 6
B. Loss, theft or diversion of any special nuclear material onsite 10CFR70.52	UE12
C. Increase in measured or calculated dose rates (mR/hr) or airborne activity levels by a factor of @ 1000 times (indication of degradation in control of radioactive materials)	A6
D. Liquid release that exceeds T/S limits for ≥ 15 minutes.	UE2
E. Gaseous release that exceeds T/S limits.	UE2
F. Gaseous release that exceeds 10 times T/S limits.	A15

NOTE
This Section is not to be used for Event
Classification. Refer to Sections 1 thru 18.

<u>Initiating Event/Condition</u>	<u>Reference*</u>
G. Dose Rate at Minimum Exclusion Area (MEA -0.56 miles from the affected unit) greater than or equal to 500 mR/hr Whole Body (WB) or 2500 mR/hr Thyroid.	SAE13
H. Dose Rate at Minimum Exclusion Area (MEA) greater than or equal to 1R/hr WB or 5 R/hr thyroid.	GE1
8. Nonradioactive Leak/Spill	
A. Toxic or flammable gas release that threatens plant personnel.	UE14d
B. Toxic or flammable gas release entering plant structures.	A18d
C. Toxic or flammable gas release entering into vital areas where safe plant operation is compromised.	SAE16c
9. Electrical/Power Failure	
A. Loss of A.C. Power capability that requires Unit Shutdown	UE7
B. Loss of all offsite power	UE7
C. Loss of all offsite power and loss of most onsite A.C. Power	A7
D. Loss of all offsite power and loss of most onsite A.C. Power for an extended period of time.	SAE6
E. Total loss of all A.C. Power	SAE6
F. Loss of all vital onsite D.C. Power	A8

NOTE
This Section is not to be used for Event
Classification. Refer to Sections 1 thru 18.

<u>Initiating Event/Condition</u>	<u>Reference*</u>
G. Loss of all vital onsite D.C. Power for > 15 minutes.	SAE7
10. Loss Of Instruments/Alarms/ Communications	
A. Indications or alarms on process or effluent parameters not functional in Control Room.	UE11
B. Loss of all or most OHAs	A14
C. Loss of all or most OHAs and plant transient initiated or in progress	SAE12
11. Control Room Evacuation	
A. Evacuation of Control Room anticipated or required.	A20
B. Evacuation of Control Room completed with control of S/D systems not established at the Remote S/D Panel within 15 minutes.	SAE18
12. Earthquake/Severe Weather	
A. Earthquake/ seismic event felt in-plant or instrument detected	UE13a
B. Earthquake/seismic event greater than Operating Basis Earthquake (OBE)	A17a
C. Earthquake/seismic event greater than Design Basis Earthquake (DBE)	SAE15a
D. Flood: Water level in river high	UE13b

NOTE
This Section is not to be used for Event
Classification. Refer to Sections 1 thru 18.

<u>Initiating Event/Condition</u>	<u>Reference*</u>
E. Flood: Water level in river high; near design basis	A17b
F. Flood: Water level in river high; greater than design basis	SAE15b
G. Water level in river low	UE13b
H. Water level in river low near design basis	A17b
I. Water level in river lower than design basis	SAE15b
J. Hurricane/unusual wind indicated by met tower instrumentation	UE13d
K. Hurricane/unusual wind indicated by met tower instrumentation near design basis	A17d
L. Hurricane/unusual wind indicated by met tower instrumentation greater than design basis	SAE15c
M. Tornado funnel observed, within MEA	UE13c
N. Tornado funnel observed within the protected area	A17c
O. Tornado funnel observed, affecting plant structures	SAE5c
P. Any major internal or external event substantially beyond design basis.	GE7

NOTE
This Section is not to be used for Event
Classification. Refer to Sections 1 thru 18.

<u>Initiating Event/Condition</u>	<u>Reference*</u>
13. Site Hazards (explosions, crashes, etc.)	
A. Aircraft unusual activity over Facility or crash within the MEA	UE14a
B. Aircraft crash within the Protected Area	A18a
C. Aircraft crash affecting Plant Structures	SAE16a
D. Turbine rotating component failure	UE14e
E. Turbine rotating component failure causing casing penetration	A18e
F. Missile impact onsite from any source within the Protected Area.	A18b
G. Missile impact onsite damaging a Plant Structure	SAE16b
H. Unplanned explosion affecting plant operations	UE14c
I. Unplanned explosion potentially compromising the function of one or more safety systems or normal operation of the plant	A18c
J. Unplanned explosion compromising the function of one or more safety systems	SAE16b

NOTE
This Section is not to be used for Event
Classification. Refer to Sections 1 thru 18.

<u>Initiating Event/Condition</u>	<u>Reference*</u>
K. Any major event substantially beyond design basis.	GE7
14. Fire	
A. Fire lasting > 10 min that affects plant operations	UE10
B. Fire potentially compromising the function of one or more safety systems	A13
C. Fire compromising the function of one or more safety systems	SAE11
D. Any major fire (substantially beyond design basis) which could cause massive common damage to safety systems.	GE7
15. Personnel Emergencies	
A. Contaminated injured person transported from site to an offsite medical facility.	UE16
H. Unplanned explosion affecting plant operations	UE14c
I. Unplanned explosion potentially compromising the function of one or more safety systems or normal operation of the plant	A18c
J. Unplanned explosion compromising the function of one or more safety systems	SAE16b
K. Any major event substantially beyond design basis	GE7

NOTE
This Section is not to be used for Event
Classification. Refer to Sections 1 thru 18.

<u>Initiating Event/Condition</u>	<u>Reference*</u>
16. Security Events	
A. Loss, theft or diversion of any special nuclear 10CFR70.52 material onsite	UE12
B. Substantiated threat, attempted entry or discovery of a suspected destructive device or evidence of a malicious act.	UE12
C. Security Alert	UE12
D. Substantiated threat, attempted entry or discovery of a suspected destructive device or evidence of a malicious act with a Security Alert declared.	A16
E. Ongoing security compromise	A16
F. Ongoing security compromise with imminent loss of physical control of the plant.	SAE14
G. Ongoing security compromise with in the loss of physical control of the plant.	GE3
17. Public Interest Items	
A. Any plant conditions exist that warrant increased awareness on the part of STATE/LOCAL authorities.	UE15

NOTE
This Section is not to be used for Event
Classification. Refer to Sections 1 thru 18.

Initiating Event/Condition

Reference*

- B. Any plant conditions exist that warrant precautionary standby of STATE/Local authorities.
- C. Any plant conditions exist that warrant precautionary activation of STATE/LOCAL authorities and notification to the general public.

ALS

SAE17

18. Tech. Specs/Plant Status Changes

- A. Unit shutdown to comply with the following T/S LCO's:
 - 1. Reactor Coolant System (RCS) leakage
 - 2. Specific activity of the Primary Coolant
 - 3. A.C. Electrical power sources
 - 4. RCS pressure/temperature limits
 - 5. Primary Containment Integrity

UE3, UE4,
UE5, UE7,
UE8

- B. Exceeding any T/S Safety Limit
- C. Manual or automatic ECCS actuation with discharge to the vessel.
- D. Liquid release that exceeds T/S limits for ≥ 15 mins.
- E. Gaseous release that exceeds T/S limits.

UE4
Tech. Spec
2.1 and 6.7.1

UE1

UE2

UE2

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CROSS REFERENCE
ATTACHMENTS TO EVENTS
SECTION iii

ATTACHMENT

EVENT/INITIATING CONDITION

1	1A, 2A, 5A, 7A, 7B, 7D, 7E, 8A, 9A, 9B, 10A, 12A, 12D, 12G, 12J, 12M, 13A, 13D, 13H, 14A, 15A, 16A, 16B, 16C, 17A, 18A, 18B, 18C, 18D, 18E
2	1B, 2B, 3A, 4A, 5B, 5D, 7C, 7F, 8B, 9C, 9F, 10B, 11A, 12B, 12E, 12H, 12K, 12N, 13B, 13E, 13F, 13I, 14B, 16D, 16E, 17B
3	1C, 1D, 2C, 3B, 4B, 5C, 5E, 7G, 8C, 9D, 9E, 9G, 10C, 11B, 12C, 12F, 12I, 12L, 12O, 13C, 13G, 13J, 14C, 16F, 17C
4	1E, 3C, 4C, 6, 7H, 12P, 13K, 14D, 16G
10	7I, 7L, 7M 7I, 7K, 16H
12	7N (2,4,6), 8D, 10D, 15B, 17D, 18F, 18G, 18H 18I, 18K
14	7N (1,3,5), 15C, 17E, 18L, 18M, 18N, 18O
15	17F
16	8D, 8E
17	15D, 15E
18	7J
19	16I
20	18P, 18Q
21	17G
22	18R, 18S, 18T

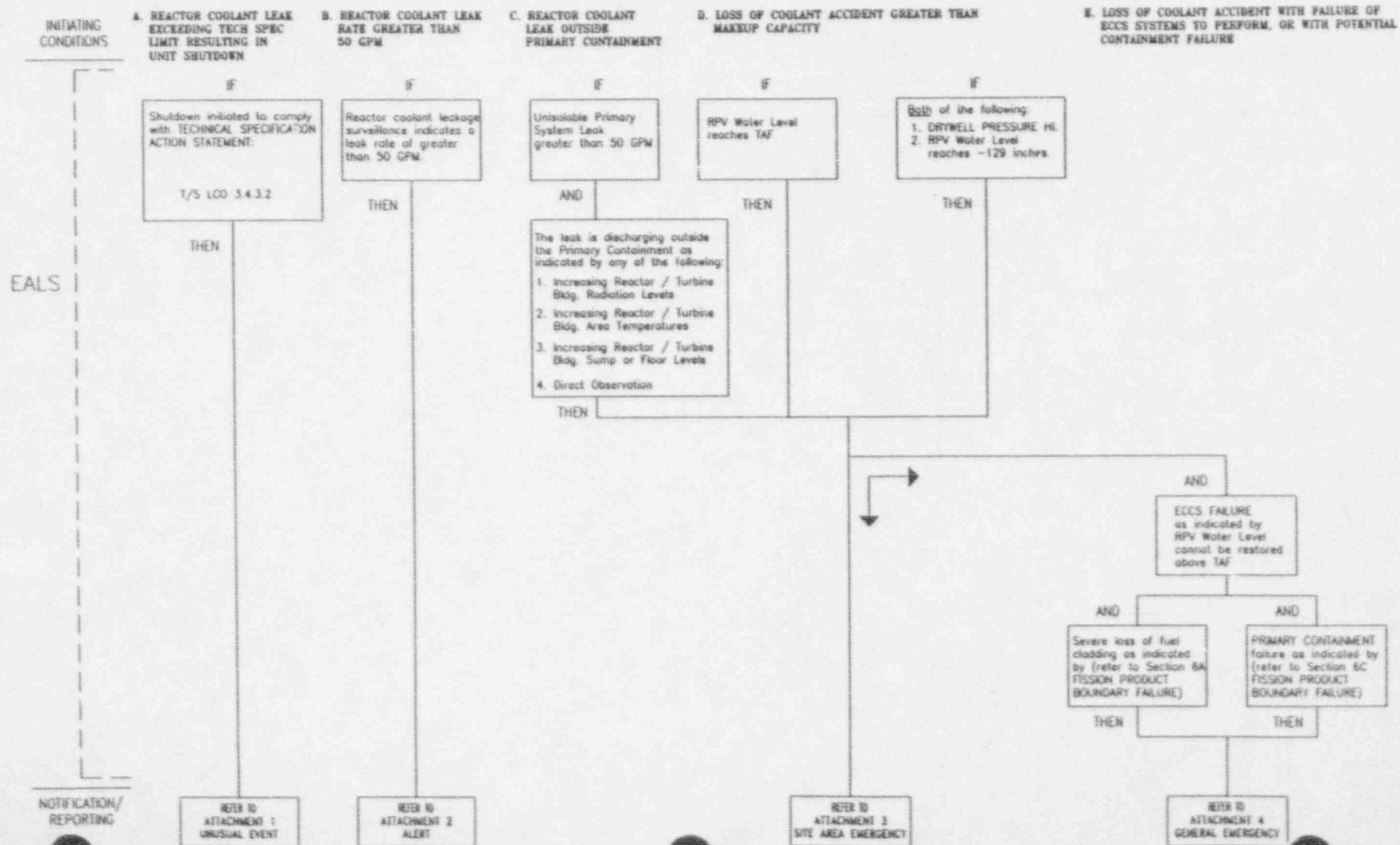
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SECTION 1

REACTOR COOLANT LEAKAGE / LOCA

ECG
SECTION 1
Pg. 1 of 1



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FISSION PRODUCT BOUNDARY FAILURES

SECTION 6

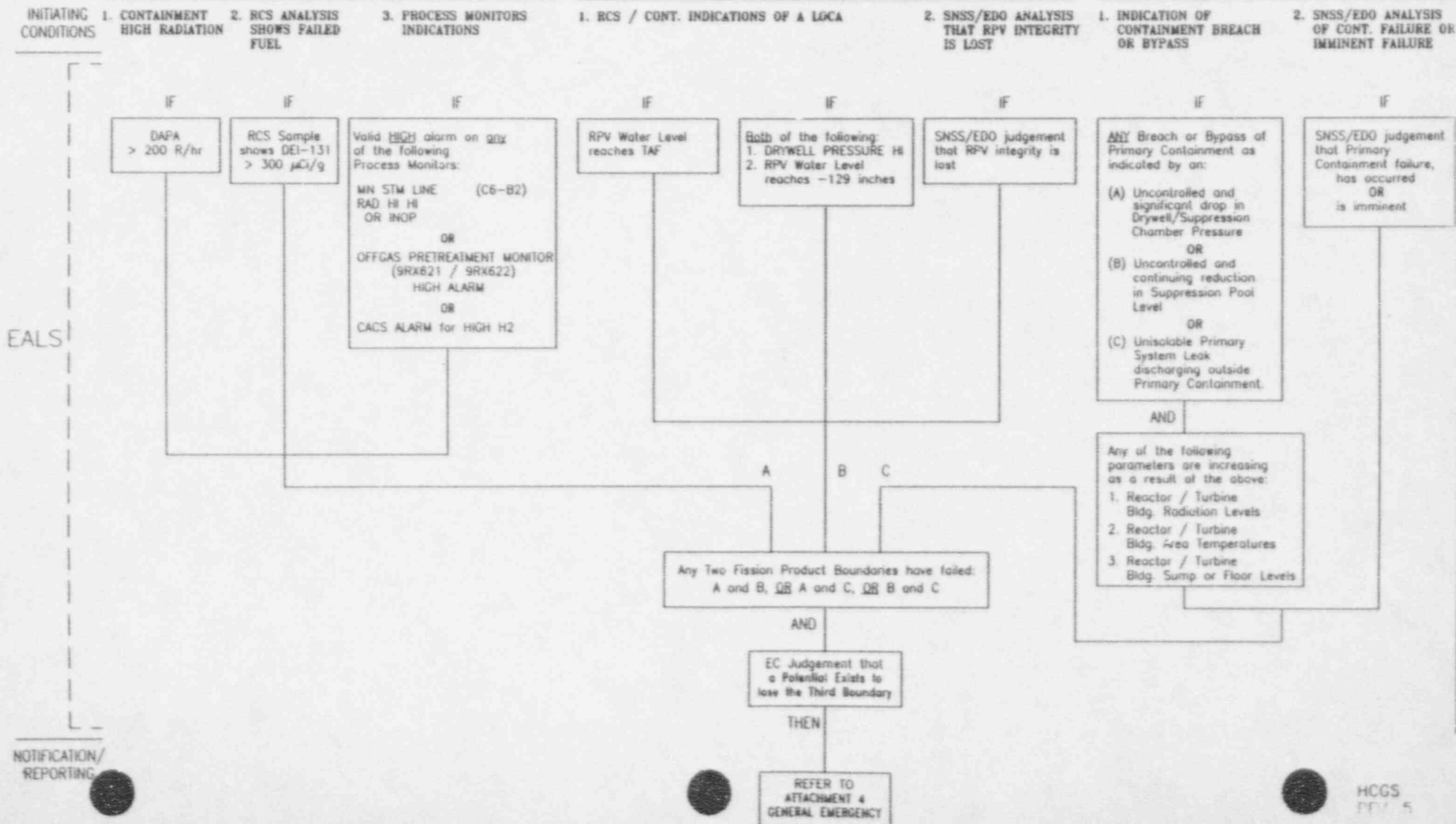
NOTE: The following conditions / EALS indicate Failure of a fission product boundary. Any two boundary Failures with a potential loss of the third boundary represents a GENERAL EMERGENCY CONDITION. Monitor for these conditions after any single boundary Failure.

ECG
SECTION 6
Pg. 1 of 1

A. SEVERE LOSS OF FUEL CLADDING

B. LOSS OF RPV INTEGRITY

C. LOSS OF PRIMARY CONTAINMENT



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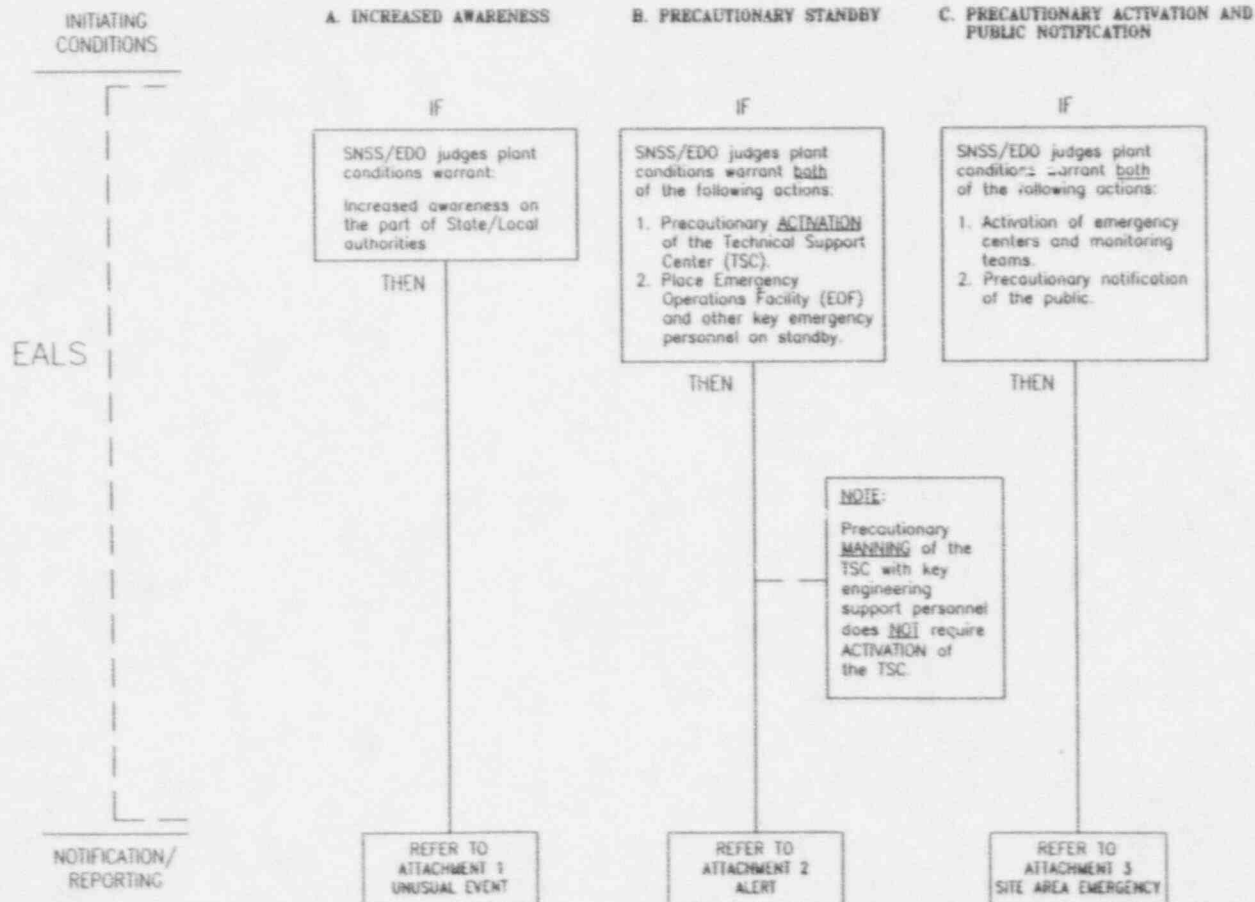
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SECTION 17

PUBLIC INTEREST

ECG
SECTION 17
Pg. 1 of 3

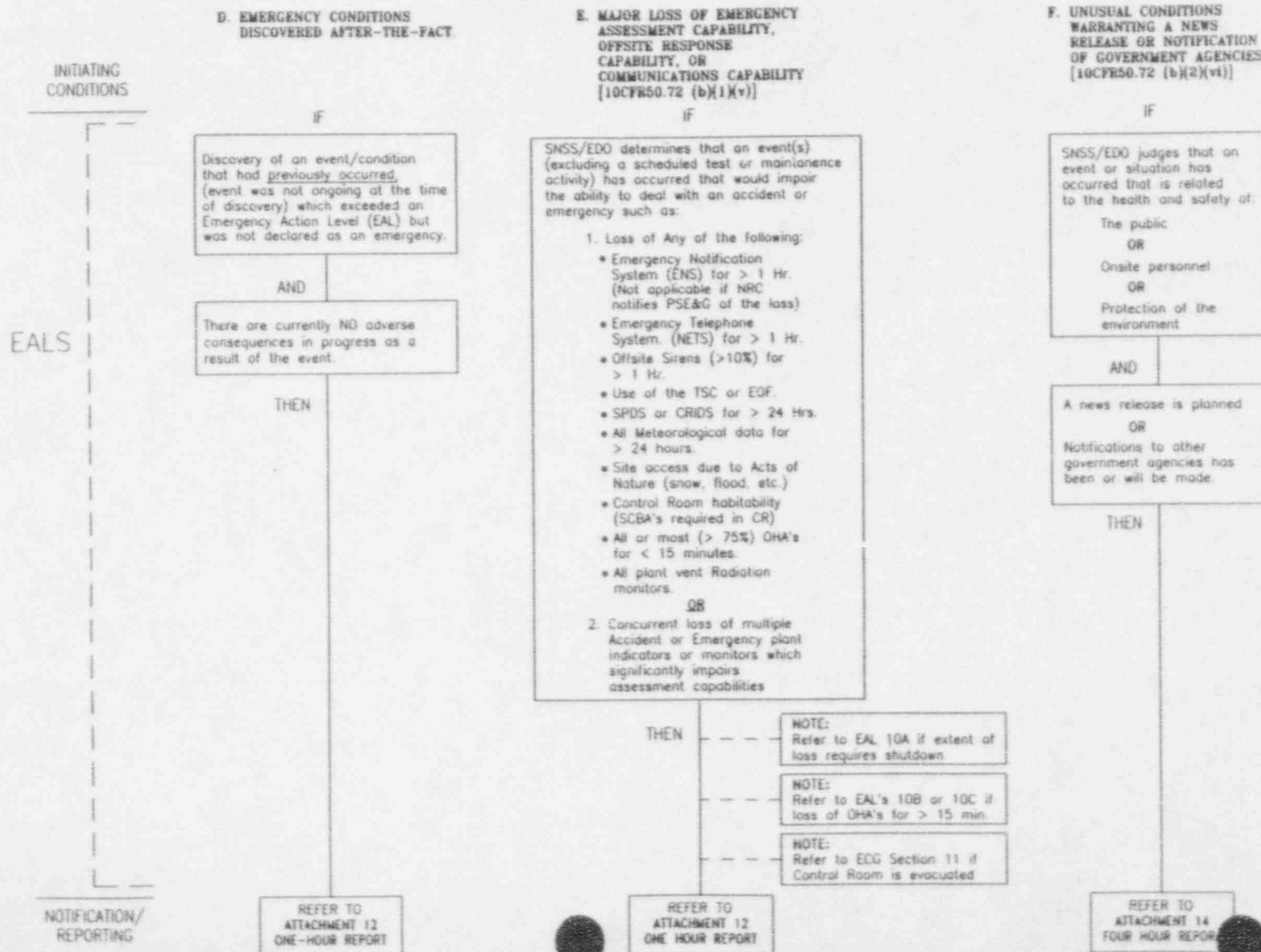
PLANT CONDITIONS EXIST THAT WARRANT THE ALERTING OF STATE AND LOCAL OFFICIALS



SECTION 17

PUBLIC INTEREST

ECG
SECTION 17
Pg. 2 of 3



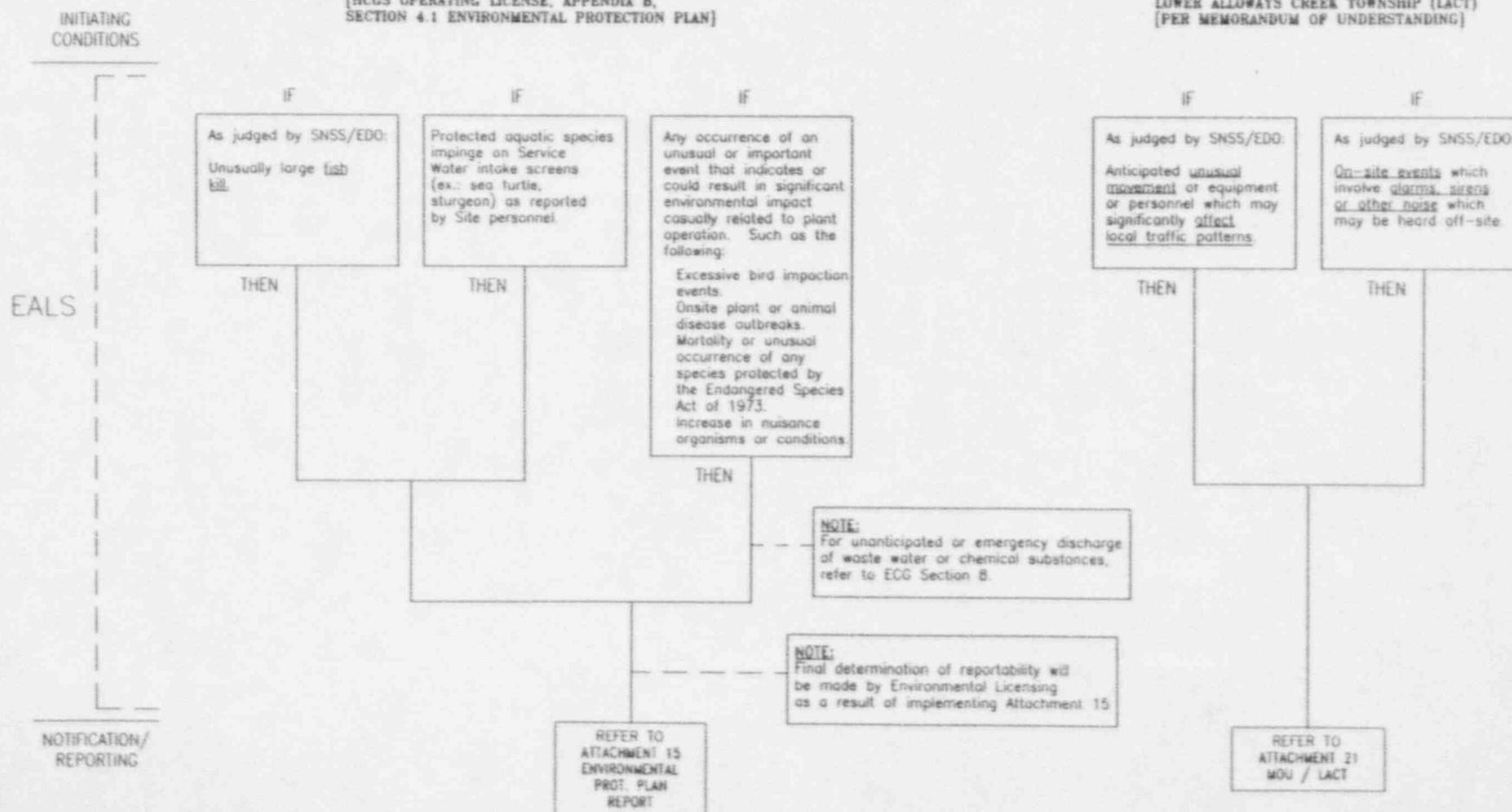
SECTION 17

PUBLIC INTEREST

ECG
SECTION 17
Pg. 3 of 3

G. UNUSUAL OR IMPORTANT ENVIRONMENTAL EVENTS [HCGS OPERATING LICENSE, APPENDIX B, SECTION 4.1 ENVIRONMENTAL PROTECTION PLAN]

H. UNUSUAL CONDITIONS DIRECTLY AFFECTING LOWER ALLOWAYS CREEK TOWNSHIP (LACT) [PER MEMORANDUM OF UNDERSTANDING]



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HOPE CREEK
EVENT CLASSIFICATION GUIDE
ATTACHMENTS SIGNATURE PAGE

July 29, 1994

<u>ATTACHMENT</u>	<u>TITLE</u>	<u>REV.</u>	<u>PAGES</u>	<u>EFFECTIVE DATE</u>
1.	Unusual Event	12	16	May 21, 1994
2.	Alert	7	7	May 21, 1994
3.	Site Area Emergency	8	7	July 29, 1994
4.	General Emergency	7	9	July 29, 1994
5.	Reserved			
6.	CM1 Log (UE/A/SAE)	18	10	May 21, 1994
7.	CM1 Log (GE)	18	10	May 21, 1994
8.	CM2 Log	13	15	Apr 29, 1994
9.	Non-Emergency Notification Reference	17	3	June 24, 1994
10.	One Hour Report - NRC/Region	1	5	July 27, 1990
11.	One Hour Report - NRC/OPS (Security)	3	5	Sept 27, 1991
12.	One Hour Report - NRC/OPS	3	5	Apr 26, 1991
13.	Reserved			
14.	Four Hour Report - NRC/OPS	2	5	July 27, 1990
15.	Environmental Protection Plan	3	3	Sept 27, 1991
16.	Spill/Damage Reporting	5	10	Jan 7, 1994
17.	Four Hour Report - Fatality/Medical	4	7	Apr 21, 1993
18.	Four Hour Report - Transportation Accident	1	6	July 27, 1990
19.	Twenty Four Hour Report - FFD	1	3	Sept 27, 1991
20.	Twenty Four Hour Report - NRC/OPS	2	5	July 27, 1990
21.	Reportable Event - LACT/MOU	0	2	May 26, 1989
22.	Other/Engineering	2	3	Sept 27, 1991
23.	Written Reports/LERS/Other	2	9	Jan 7, 1994

SIGNATURE PAGE

Prepared By: C BANNER Rev 29 7/18/94
(If Editorial Revisions Only, Last Approved Revision) Date

Reviewed By: N/A
Station Qualified Reviewer Date

Significant Safety Issue
() Yes () No

Reviewed By: N/A
Department Manager Date

Reviewed By: Tom D. Swartz 7/18/94
Emergency Preparedness Manager Date

Reviewed By: N/A
General Manager - Quality Assurance/Safety Review
(If Applicable) Date

SORC Review and Station Approvals

N/A
Mtg. No. Salem Chairman
Date

N/A
Mtg. No. Hope Creek Chairman
Date

N/A
General Manager - Salem
Date

N/A
General Manager - Hope Creek
Date

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HOPE CREEK
EVENT CLASSIFICATION GUIDE
SECTION SIGNATURE PAGES
July 15, 1994

<u>SECTION</u>	<u>TITLE</u>	<u>REV</u>	<u>PAGES</u>	<u>EFFECTIVE DATES</u>
i.	Introduction	2	7	Jan 7, 1994
ii.	Cross Reference - Event to Requirement	2	10	July 15, 1994
iii.	Cross Reference - Attachment to Events	7	1	July 15, 1994
1.	REACTOR COOLANT LEAKAGE/LOCA	6	1	July 15, 1994
2.	STEAM BREAK OR SRV FAILED OPEN	0	2	May 26, 1989
3.	FAILURE TO SCRAM	2	1	Dec 21, 1992
4.	LOSS OF DECAY HEAT REMOVAL	2	1	Aug 21, 1992
5.	FUEL DAMAGE/DEGRADED CORE	3	2	Dec 21, 1992
6.	FISSION PRODUCT BOUNDARY FAILURE	5	1	July 15, 1994
7.	RADIOLOGICAL RELEASES/OCCURRENCES	5	5	Jan 28, 1994
8.	NON-RADIOACTIVE LEAK/SPILL (toxic gas, oil spill, hazmat)	3	2	Jan 7, 1994
9.	ELECTRICAL POWER FAILURE	3	2	Jan 7, 1994
10.	LOSS OF INSTRUMENTS/ALARMS/COMMUNICATIONS	4	2	May 10, 1993
11.	CONTROL ROOM EVACUATION	0	1	May 26, 1989
12.	QUAKE/STORMS (earthquake, wind, floods, etc)	4	6	Jan 7, 1994
13.	SITE HAZARDS (aircraft crash, missiles, explosions, etc.)	1	5	Aug 21, 1992
14.	FIRE	3	1	Jan 7, 1994
15.	PERSONNEL EMERGENCIES/MEDICAL	3	2	Jan 7, 1994
16.	SECURITY EVENTS/FFD	5	3	Jan 7, 1994
17.	PUBLIC INTEREST ITEMS	6	3	July 15, 1994
18.	TECH SPECS/PLANT STATUS CHANGES	10	5	Jan 7, 1994

SIGNATURE PAGE

Prepared By: GARY BANNER 6-12-94
(If Editorial Revisions Only, Last Approved Revision) Date

Reviewed By: MITCHELL DIOR 6/12/94
Station Qualified Reviewer Date

Significant Safety Issue
() Yes (V) No
Reviewed By: W. P. O'Malley 6/20/94
Department Manager Date

Reviewed By: James F. [Signature] 6/14/94
Emergency Preparedness Manager Date

Reviewed By: N/A
General Manager - Quality Assurance/Safety Review Date
(If Applicable)

SORC Review and Station Approvals

Mtg. No. N/A Salem Chairman
Date

Mtg. No. N/A Hope Creek Chairman
Date

N/A
General Manager - Salem
Date

Robert J. Honey [Signature]
General Manager - Hope Creek
7/5/94
Date