

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0	9	C	C	11	F	12	B	13	V	A	L	V	E	X	14	X	15	A	16																					
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.		ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER		CAUSE DESCRIPTION AND CORRECTIVE ACTIONS										
17		83		042		03		L		0		B		Z		Z		Z		0000		Y		N		A		R344		27										

FACILITY STATUS		% POWER			OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION		
1	5	B	28	0	0	0	29	NA	C	31	Local Leak Rate Test
7	8	9	10	11	12	13	14	15	16	17	18
ACTIVITY CONTENT RELEASED OF RELEASE		AMOUNT OF ACTIVITY			LOCATION OF RELEASE						
1	6	Z	33	Z	34	NA	35	NA	36	37	38
7	8	9	10	11	12	13	14	15	16	17	18
PERSONNEL EXPOSURES		TYPE		DESCRIPTION							
1	7	0	0	0	37	L	38	NA	39	40	41
7	8	9	10	11	12	13	14	15	16	17	18
PERSONNEL INJURIES		TYPE		DESCRIPTION							
1	8	0	0	0	40	NA	41	42	43	44	45
7	8	9	10	11	12	13	14	15	16	17	18
LOSS OF OR DAMAGE TO FACILITY		TYPE		DESCRIPTION							
1	9	7	42	NA	43	44	45	46	47	48	49
7	8	9	10	11	12	13	14	15	16	17	18
PUBLICITY		TYPE		DESCRIPTION							
2	0	N	44	NA	45	46	47	48	49	50	51
7	8	9	10	11	12	13	14	15	16	17	18

8305270195 830520
PDR ADOCK 05000373
S PDR

NRC USE ONLY

68	69	70	71	72	73	74	75	76	77	78	79	80
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V. Masterson

PHONE:

357-6761

NRC USE ONLY

8305270195 830520
PDR ADOCK 05000373
S PDR



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

May 20, 1983

James G. Keppler
Regional Administrator
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Dear Sir:

Reportable Occurrence Report #83-042/03L-0 Docket #050-373 is being submitted to your office in accordance with LaSalle County Nuclear Power Station Technical Specification 6.6.B.2.(b), conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

G. J. Diederich
Superintendent
LaSalle County Station

GJD/GW/sjc

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
U. S. NRC Document Management Branch
Inpo-Records Center
File/NRC

MAY 23 1983

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- I. LER NUMBER: 83-042/03L-0
- II. LASALLE COUTNY STATION: Unit 1
- III. DOCKET NUMBER: 050-373
- IV. EVENT DESCRIPTION:

During the performance of LTS 100-3, Main Steam Isolation Valve Local Leak-Rate Test, the inboard MSIV's 1B21-F022A/B/C/D, were found to be exceeding the maximum allowable leak-rate of 25 scfh. Work Requests were initiated to correct the cause of the leakage.

V. PROBABLE CONSEQUENCES OF THE EVENT:

The Main Steam Isolation Valves are required to be tested at least once per 18 months per Technical Specification Surveillance Requirement 4.6.1.2.f. The maximum allowable leakage that the MSIV's can exhibit is 25 standard cubic feet/hour (SCFH).

On April 23, 1983, while performing LTS-100-3, Main Steam Isolation Valve Local Leak-Rate Test, the Main Steam Isolation Valves failed to satisfactorily pass the leak-rate criteria of less than 25 SCFH. At the time of the occurrence, it could not be determined whether the outboard MSIV's or inboard MSIV's were leaking. Further testing was conducted per LTS-100-3 effectively testing only the outboard MSIV's 1B21-F028A/B/C/D, which successfully passed. From these test results, the cause of the excessive leak-rate was isolated to the inboard MSIV's 1B21-F022A/B/C/D.

The purpose of the Main Steam Isolation Valves is to provide a barrier against release of radioactive fission products from the Primary Containment. When closed, the valves provide part of the reactor containment barrier.

Both inboard and outboard MSIV's were operational and both would have performed their intended function. Because the outboard MSIV's 1B21-F028A/B/C/D leakage rate was well below the 25 SCFH, any release would have been limited by the leak-rate of the outboard MSIV's. Any large leakage from the inboard MSIV's would have been controlled by the leakage control system and directed to the Standby Gas Treatment System for processing prior to release to environ. The safety of the public was maintained.

VI. CAUSE:

The reason for the Main Steam Isolation Valves failing to pass the leak-rate allowable limit of 25 SCFH, is due to a less than perfect match between the Main Steam Isolation Valve Disc and seat and/or the pilot valve disc and seat. The valves are manufactured by Rockwell International.

VII. CORRECTIVE ACTION:

Work Reuquests (L24255, L24256, L24257, L24258) were generated as a result of the excessive leak-rates of the inboard MSIV's 1B21-F022A/B/C/D.

The inboard MSIV valve seats and valve discs were lapped and machined for proper seating of mating surfaces. After completion of repair work, the valves were re-installed, and leak-rate tested individually. On May 15, 1983, leak-rate testing of MSIV's 1B21-F022A/B/C/D was completed successfully

VII. CORRECTIVE ACTION (Cont'd)

with all valves well below the allowable leakage of 25 SCFH.

Prepared by Vincent Masterson