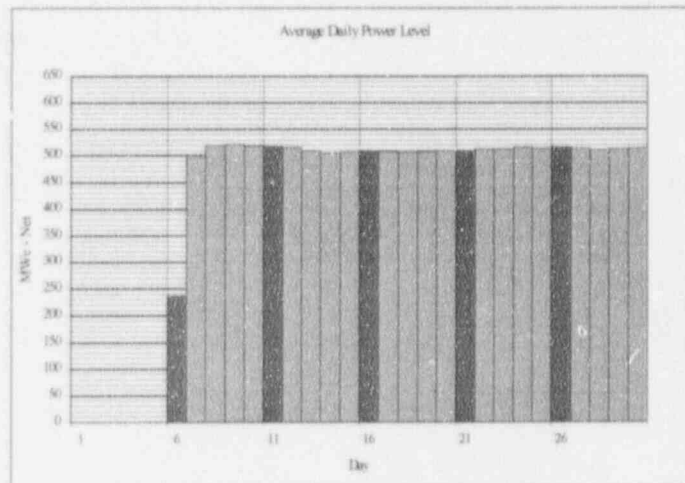


OPERATING DATA REPORT

DOCKET NO: 50-0331
 DATE: 07/15/94
 Unit: Duane Arnold Energy Center
 COMPLETED BY: Richard Woodward
 TELEPHONE: (319) 851-7318

OPERATING STATUS

1. Unit Name: Duane Arnold Energy Center
2. Reporting Period: June 1994
3. Licensed Thermal Power (MW_{th}): 1658
4. Nameplate Rating (Gross MW_e DER): 565.7 (Turbine)
5. Design Electrical Rating (Net MW_e DER): 538
6. Maximum Dependable Capacity (Gross MW_e MDC): 545
7. Maximum Dependable Capacity (Net MW_e MDC): 515
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) since the last report, Give Reasons: Not Applicable
9. Power Level to Which Restricted, If Any (Net MW_e): Not Applicable
10. Reasons for Restrictions, If Any: Not Applicable



		June-94	Year	Cummulative
11.	Hours in Reporting Period	720.0	4,343.0	170,159.0
12.	Number of Hours Reactor Was Critical	647.1	4,210.9	127,155.5
13.	Reactor Reserve Shutdown Hours	0.0	0.0	192.8
14.	Hours Generator On-Line	599.1	4,162.8	123,941.5
15.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated (MWH)	977,116.1	6,842,142.9	171,300,707.1
17.	Gross Electrical Energy Generated (MWH)	319,345.0	2,294,563.0	57,374,948.5
18.	Net Electrical Energy Generated (MWH)	299,252.9	2,161,668.9	53,793,331.8
19.	Unit Service Factor	83.2%	95.9%	72.8%
20.	Unit Availability Factor	83.2%	95.9%	72.8%
21.	Unit Capacity Factor (Using MDC Net)	80.7%	96.6%	62.7%
22.	Unit Capacity Factor (Using DER Net)	77.3%	92.5%	60.0%
23.	Unit Forced Outage Rate	6.1%	1.3%	11.6%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of each): Not Applicable
25. If Shutdown at End of Report Period, Est. Date of Startup: Not Applicable

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-0331
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MONTH June 1994

Day	Average Daily Power Level (MWe-Net)
1	-11.5
2	-11.5
3	-11.7
4	-13.2
5	-16.2
6	236.8
7	500.6
8	520.1
9	522.5
10	518.7
11	517.6
12	515.3
13	510.3
14	505.6
15	506.5
16	508.7
17	507.7
18	507.2
19	509.0
20	509.6
21	510.4
22	513.2
23	512.9
24	516.2
25	514.5
26	515.8
27	514.8
28	511.7
29	512.7
30	514.7
31	#N/A

REFUELING INFORMATION

DOCKET NO: 50-0331
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1. Name of facility.

Duane Arnold Energy Center

2. Scheduled date for next refueling shutdown.

February 24, 1995

3. Scheduled date for restart following refueling.

April 14 - 19, 1995

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

No

5. Scheduled date(s) for submitting proposed licensing action and supporting information.

Not applicable

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

No

7. Current and projected fuel assemblies inventory:

	Number of Fuel Assemblies	Projected date of last refueling that can be discharged
currently installed in reactor core	368	n/a
previously discharged from core to Spent Fuel Storage Pool	1280	n/a
under present physical capacity of Spent Fuel Storage Pool	1898	2001
under planned capacity of Spent Fuel Storage Pool following re-racking (currently under construction)	2411	2007
under Licensed Capacity of Spent Fuel Storage Pool	3152	2014

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UNIT SHUTDOWNS AND POWER REDUCTIONS
 REPORT MONTH: June 1994

No.	Date	Type (1)	Duration (Hours)	Reason (2)	Method of Shutting Down Reactor (3)	Licensee Event Report #	System Code (4)	Comp. Code (5)	Cause
1	6/1/94	S	82.0	B	4	Not applicable	SB	V	Planned repair of various Balance of Plant steam line valve packing leaks to improve steam cycle isolation and thermal/mechanical/electrical conversion and reliability for remainder of fuel operating cycle.
2	6/3/94	F	11.1	A	4	Not applicable	AA	RLY	Degraded relay in the Rod Sequence Control System logic caused interruption of rod-pull.
3	6/3/94	F	1.7	G	4	Not applicable	AA	HCU	While withdrawing control rods, operator inadvertently inserted control rod one notch. Pulled rod to correct position.
4	6/4/94	F	3.1	A	4	Not applicable	JM	V	During reactor startup and micro-purge flushing, a valve leak caused a reactor pressure vessel level indication disparity.
5	6/5/94	F	23.0	A	4	Not applicable	JC	CBL	Unanticipated 1/2 GP I when Hi Condenser back pressure logic bypass key was returned to normal. Cause was a severed cable from a hi condenser back pressure switch which resulted in an isolation signal when the bypass switch was taken out of bypass.

1 - F: Forced
 S: Scheduled

2 - Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3 - Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Continued
 5-Reduced Load
 9-Other (Explain)

4 - Exhibit G-
 Instructions for Preparation of Data
 Entry Sheets for Licensee Event
 Report (LER) File (NUREG-0161)

5 - Exhibit 1
 (Same Source)

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Monthly Operational Overview for June 1994:

The DAEC started-up June 6 at 00:57 following the May 29 shutdown for repairs on the turbine control valve electro-hydraulic system and balance-of-plant steam line valve packing leaks. The start-up was delayed 38.9 hours following completion of the pre-planned scope of work to respond to unforeseen problems which occurred during the start-up sequence. Following start-up, the unit operated at essentially full power throughout the rest of the month.

Allocation of Production & Losses:

	Electrical Output MWe	Capacity Factor % of 565.7 MWe	Full Power Equivalent Hours
Actual Metered Gross Electric Output	443.5	78.4%	564.5
Off-line June 1 00:00 - June 6 00:57	95.0	16.8%	121.0
Weather (seasonal losses, condenser pressure greater than design)	8.0	1.4%	10.2
Start-up June 6 & 7	8.6	1.5%	10.9
Other Capacity MWe Losses (Operating at less than full thermal	0.6	0.1%	0.7
Efficiency MWe Losses (thermal conversion @ less than full design	10.0	1.8%	12.7
Design Electric Output	565.7	100.0%	720.0

At 0109 hours on June 9, 1994 a differential pressure transmitter (PDT) failed, causing an "Excess Flow Check Valve" alarm and isolation of the respective check valve on the small instrument line, thereby preventing the release of CRD system process fluid to the reactor building. This event was considered an Emergency Safety Feature (ESF) actuation. The cause of this event was a packing leak that dripped condensate water onto the PDT housing. Over time, two bolts that coupled the PDT housing components failed, causing subsequent failure of the PDT and the resultant leak. Corrective actions include replacement of the PDT with a model that is not conducive to the entrapment of water in its housing, procedure revision, and performance of a detailed walk-down of like components to ensure there are no similar concerns. There was no effect on plant safety, personnel safety, or plant availability as a result of this event. (LER #94-09)

Licensing Action Summary:

Plant Availability:	83.2%	Unplanned Auto Scrams (while/critical) this month:	0
Number of reportable events:	1	Unplanned Auto Scrams (while/critical) last 12 months:	1