



Commonwealth Edison
1400 Opus Place
Downers Grove, Illinois 60515

July 24, 1994

Mr. William Russell, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attn: Document Control Desk

Subject: Braidwood Station Units 1 and 2
Additional Information Relating to the
Request to Amend Technical Specification
Sections 3.4.9.1 and 3.4.9.3
NPF-72/77; NRC Docket Nos. 50-456/457

References: See Attachment

Dear Mr. Russell:

Reference 5 transmitted the Commonwealth Edison Company (ComEd) request to amend Section 3.4.9.1 and 3.4.9.3 of the Technical Specification for Braidwood Units 1 and 2. Reference 3 transmitted additional information which was requested during the referenced 4 teleconference. As discussed during the reference 2 teleconference, it was brought to Com Ed's attention that Technical Specification submittal needed to be revised to account for a 60 pounds per square inch gauge (psig) pressure channel measurement uncertainty. This submittal was subsequently supplemented in the reference 1 letter. Reference 3 also transmitted information on the Braidwood 1 Capsule U LTOPS. This information was subsequently revised to include the 60 psig pressure channel measurement uncertainty and is attached for your review.

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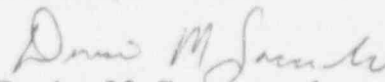
Mr. Russell

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If you have any questions concerning this correspondence please contact this office.

Sincerely,



Denise M. Saccomando
Nuclear Licensing Administrator

Attachment

cc: R. Assa, Braidwood Project Manager-NRR
S. Dupont, Senior Resident Inspector-Braidwood
B. Clayton, Branch Chief-Region III
Office of Nuclear Facility Safety-IDNS

RESPONSE TO NRC REQUEST FOR
ADDITIONAL INFORMATION FOR BRAIDWOOD 1 CAPSULE U LTOPS

				5.37 EFPY	
				App G	
<u>Temp. (°F)</u>	<u>LTOP PORV</u> <u>Setpoint (psig)</u>	<u>Overshoot (psig)</u>	<u>MI or HI</u> <u>Limiting</u>	<u>Limit (psig)</u> <u>Utilized</u>	<u>App G</u> <u>Margin (psig)</u>
70	490	30	MI	581	61
97	490	30	MI	628	108
127	494	30	MI	720	196
147	504	30	MI	808*	274(266)**
177	525	30	MI	998*	443(245)**
227	610	35	HI	1566*	921(155)**
277	670	54	HI	_*	- (76)**
327	725	74	HI	_*	- (1)**
377	700	98	HI	_*	- (2)**

* A constant PORV piping structural integrity criteria of 800 psig is conservatively imposed for setpoint development when the Appendix G limit exceeds 800 psig.

** Margin to PORV structural limit in parenthesis.

Pressure channel measurement uncertainty from WCAP-10529 (not accounted for in setpoint development): 60 psig

NEUTRON FLUENCE ON THE INSIDE DIAMETER (WETTED SURFACE) OF THE REACTOR VESSEL AT 5.37 EFPY IS 5.0×10^{18} N/CM².

REFERENCES

- 1) D. Saccomando letter to W. Russell dated July 21, 1994, transmitting supplement to request to amend Technical Specification Sections 3.4.9.1 and 3.4.9.3
- 2) Teleconference conducted July 18, 1993, between the Commonwealth Edison Company and the Nuclear Regulatory Commission concerning the proposed Technical Specification Amendment to the Heat Up/Cool Down Curves
- 3) D. Saccomando letter to W. Russell dated July 11, 1994, transmitting request to amend Technical Specification Sections 3.4.9.1 and 3.4.9.3
- 4) Teleconference conducted July 7, 1993, between the Commonwealth Edison Company and the Nuclear Regulatory Commission concerning the proposed Technical Specification Amendment to the Heat Up/Cool Down Curves
- 5) D. Saccomando letter to W. Russell dated March 30, 1994, transmitting request to amend Technical Specification Sections 3.4.9.1 and 3.4.9.3