

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
3/4.6.3 CONTAINMENT ISOLATION VALVES.....	3/4 6-18
3/4.6.4 COMBUSTIBLE GAS CONTROL	
Hydrogen Analyzers.....	3/4 6-19
Electric Hydrogen Recombiners.....	3/4 6-20
<u>3/4.7 PLANT SYSTEMS</u>	
3/4.7.1 TURBINE CYCLE	
Safety Valves.....	3/4 7-1
TABLE 3.7-1 MAXIMUM ALLOWABLE POWER RANGE NEUTRON FLUX HIGH SETPOINT WITH INOPERABLE STEAM LINE SAFETY VALVES DURING 4 LOOP OPERATION.....	3/4 7-2
TABLE 3.7-2 STEAM LINE SAFETY VALVES PER LOOP.....	3/4 7-3
Auxiliary Feedwater System.....	3/4 7-4
Auxiliary Feedwater Storage Tank.....	3/4 7-6
Specific Activity.....	3/4 7-7
TABLE 4.7-1 SECONDARY COOLANT SYSTEM SPECIFIC ACTIVITY SAMPLE AND ANALYSIS PROGRAM.....	3/4 7-8
Main Steam Line Isolation Valves.....	3/4 7-9
Atmospheric Steam Relief Valves .....	3/4 7-10
3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION.....	3/4 7-11
3/4.7.3 COMPONENT COOLING WATER SYSTEM.....	3/4 7-12
3/4.7.4 ESSENTIAL COOLING WATER SYSTEM.....	3/4 7-13
3/4.7.5 ULTIMATE HEAT SINK.....	3/4 7-14
3/4.7.6 (This specification number is not used.)	
3/4.7.7 CONTROL ROOM MAKEUP AND CLEANUP FILTRATION SYSTEM.....	3/4 7-16
3/4.7.8 FUEL HANDLING BUILDING (FHB) EXHAUST AIR SYSTEM.....	3/4 7-19
3/4.7.9 SNUBBERS.....	3/4 7-21
FIGURE 4.7-1 SAMPLE PLAN 2) FOR SNUBBER FUNCTIONAL TEST.....	3/4 7-26
3/4.7.10 SEALED SOURCE CONTAMINATION.....	3/4 7-27
3/4.7.11 (This specification number is not used.)	
3/4.7.12 (This specification number is not used.)	
3/4.7.13 AREA TEMPERATURE MONITORING.....	3/4 7-31
TABLE 3.7-3 AREA TEMPERATURE MONITORING.....	3/4 7-32
3/4.7.14 ESSENTIAL CHILLED WATER SYSTEM .....	3/4 7-33

MAIN  
FEEDWATER  
SYSTEM

3/4 7-10a

TABLE 3.3-1 (Continued)  
REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
13. Reactor Coolant Flow--Low					
a. Single Loop (Above P-8)	3/loop	2/loop in any oper- ating loop	2/loop in each oper- ating loop	1	6 <sup>(1)</sup>
b. Two Loops (Above P-7 and below P-8)	3/loop	2/loop in two oper- ating loops	2/loop each oper- ating loop	1	6 <sup>(1)</sup>
14. Steam Generator Water Level--Low-Low	4/stm. gen.	2/stm. gen. in any oper- ating stm. gen.	3/stm. gen. each oper- ating stm. gen.	1, 2	6 <sup>(1)</sup>
15. Undervoltage--Reactor Coolant Pumps (Interlocked with P-7)	4-1/bus	2	3	1	6
16. Underfrequency--Reactor Coolant Pumps (Interlocked with P-7)	4-1/bus	2	3	1	6
17. Turbine Trip (Interlocked with P-9)					
a. Low Emergency Trip Fluid Pressure	3	2	2	1	6
b. Turbine Stop Valve Closure	4	2	3	1	6

TABLE 4.3-1 (Continued)

TABLE NOTATIONS (Continued)

ATTACHMENT 3  
ST-HL-AE-4842  
PAGE 3 OF 11

- (10) Setpoint verification is not applicable.
- (11) The TRIP ACTUATING DEVICE OPERATIONAL TEST shall independently verify the OPERABILITY of the undervoltage and shunt trip attachments of the Reactor Trip Breakers.
- (12) OPERABILITY shall be verified by a check of memory devices, input accuracies, Boron Dilution Alarm setpoints, output values, and software functions.
- (13) (Not used)
- (14) The TRIP ACTUATING DEVICE OPERATIONAL TEST shall independently verify the OPERABILITY of the undervoltage and shunt trip circuits for the Manual Reactor Trip Function. The test shall also verify the OPERABILITY of the Bypass Breaker trip circuit(s).
- (15) Local manual shunt trip prior to placing breaker in service.
- (16) Automatic undervoltage trip.
- (17) Each channel shall be tested at least every 92 days on a STAGGERED TEST BASIS.
- (18) The surveillance frequency and/or MODES specified for these channels in Table 4.3-2 are more restrictive and, therefore, applicable.

\*Complete verification of operability of the shunt trip relay circuitry shall be initially implemented for each unit prior to the affected unit's startup from the first planned or unplanned shutdown occurring after May 19, 1992.

TABLE 3.3-1 (Continued)

TABLE NOTATIONS

\*When the Reactor Trip System breakers are in the closed position and the Control Rod Drive System is capable of rod withdrawal.

##Below the P-6 (Intermediate Range Neutron Flux Interlock) Setpoint.

###Below the P-10 (Low Setpoint Power Range Neutron Flux Interlock) Setpoint.

(1) The applicable MODES and ACTION statement for these channels noted in Table 3.3-3 are more restrictive and, therefore, applicable.

ACTION STATEMENTS

ACTION 1 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or be in HOT STANDBY within the next 6 hours.

ACTION 2 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:

- a. The inoperable channel is placed in the tripped condition within 6 hours,
- b. The Minimum Channels OPERABLE requirement is met; however, the inoperable channel may be bypassed for up to 4 hours for surveillance testing of other channels per Specification 4.3.1.1, and
- c. Either, THERMAL POWER is restricted to less than or equal to 75% of RATED THERMAL POWER and the Power Range Neutron Flux Trip Setpoint is reduced to less than or equal to 85% of RATED THERMAL POWER within 4 hours; or, the QUADRANT POWER TILT RATIO is monitored at least once per 12 hours per Specification 4.2.4.2.

NO CHANGE ON THIS PAGE  
FOR INFORMATION ONLY

INSTRUMENTATION

THIS IS A NEW PAGE

3.3.3. <sup>10</sup>9 (This specification number is not used.)

## INSTRUMENTATION

### 3/4.3.4 TURBINE OVERSPEED PROTECTION

#### LIMITING CONDITION FOR OPERATION

3.3.4 At least one Turbine Overspeed Protection System shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

#### ACTION:

- a. With one stop valve or one governor valve per high pressure turbine steam line inoperable and/or with one reheat stop valve or one reheat intercept valve per low pressure turbine steam line inoperable, restore the inoperable valve(s) to OPERABLE status within 72 hours, or close at least one valve in the affected steam line(s) or isolate the turbine from the steam supply within the next 6 hours.
- b. With the above required Turbine Overspeed Protection System otherwise inoperable, within 6 hours isolate the turbine from the steam supply.

## SURVEILLANCE REQUIREMENTS

4.3.4.1 The provisions of Specification 4.0.4 are not applicable.

4.3.4.2 The above required Turbine Overspeed Protection System shall be demonstrated OPERABLE:

- a. At least once per 31 days in MODES 1 and 2 when the main turbine is operating by cycling each of the following valves through at least one complete cycle from the running position:
  - 1) Four high pressure turbine stop valves,
  - 2) Four high pressure turbine governor valves,
  - 3) Six low pressure turbine reheat stop valves, and
  - 4) Six low pressure turbine reheat intercept valves.
- b. At least once per 31 days in MODES 1 and 2 when the main turbine is operating by direct observation of the movement of each of the above valves through one complete cycle from the running position,
- c. At least once per 18 months by performance of a CHANNEL CALIBRATION on the Turbine Overspeed Protection Systems, and
- d. At least once per 40 months by disassembling at least one of each of the above valves and performing a visual and surface inspection of valve seats, disks, and stems and verifying no unacceptable flaws or excessive corrosion. If unacceptable flaws or excessive corrosion are found, all other valves of that type shall be inspected.\*

\*Disassembly and inspection of the low pressure turbine reheat intercept valves are not required prior to the end of the first 40 month interval.

Unit 1 valves will be disassembled and inspected in the Unit 1 fifth refueling outage, following a one-time extension of the inspection interval from 40 months (50 months with the 25% grace period) to approximately 52 months.



## CONTAINMENT SYSTEMS

### RECIRCULATION FLUID PH CONTROL SYSTEM

#### LIMITING CONDITIONS FOR OPERATION

3.6.2.2 The recirculation fluid pH control system shall be operable with between 11,500 lbs. (213 cu. ft.) and 15,100 lbs (252 cu. ft.) of trisodium phosphate (w/12 hydrates) available in the storage baskets in the containment.

APPLICABILITY: MODES 1, 2, 3, and 4.

#### ACTION:

With less than the required amount of trisodium phosphate available, restore the system to the correct amount within 72 hours or be in at least HOT STANDBY within the next 6 hours; restore the system to the correct amount within the next 48 hours or be in COLD SHUTDOWN within the following 30 hours.

outside the specified range

#### SURVEILLANCE REQUIREMENTS

4.6.2.2 During each refueling outage, as a minimum, the recirculation fluid pH control system shall be demonstrated operable by visually verifying that:

- a. 6 trisodium phosphate storage baskets are in place, and
- b. have maintained their integrity, and
- c. are filled with trisodium phosphate such that the level is above the indicated fill mark.

within

specified range

PLANT SYSTEMS

AUXILIARY FEEDWATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.2 At least four independent steam generator auxiliary feedwater pumps and associated flow paths shall be OPERABLE with:

- a. Three motor-driven auxiliary feedwater pumps, each capable of being powered from separate emergency busses, and
- b. One steam turbine-driven auxiliary feedwater pump capable of being powered from an OPERABLE steam supply system.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With the Train A motor-driven auxiliary feedwater pump inoperable, initiate corrective actions to restore the pump to OPERABLE status as soon as possible. The provisions of Specification 4.0.4 are not applicable.
- b. With any of the following combinations of auxiliary feedwater pumps inoperable:
  - 1) Train B or Train C motor-driven pump,
  - 2) Train D turbine-driven pump and any one motor-driven pump,
  - 3) Train A and either Train B or Train C motor-driven pump, or
  - 4) Train D turbine-driven pump

Restore the affected auxiliary feedwater pump(s) to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.

- c. With Train D turbine-driven auxiliary feedwater pump inoperable, restore the inoperable pump to OPERABLE status within 168 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours. These actions are applicable for Unit 1 only during the restart of Unit 1 from the 1993 Unit 1 Outage.
- C. ~~d.~~ With Train B and Train C motor driven pumps, or any three auxiliary feedwater pumps inoperable, be in at least HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 6 hours.
- d. ~~e.~~ With four auxiliary feedwater pumps inoperable, immediately initiate corrective action to restore at least one auxiliary feedwater pump to OPERABLE status as soon as possible.



# SITE MAP

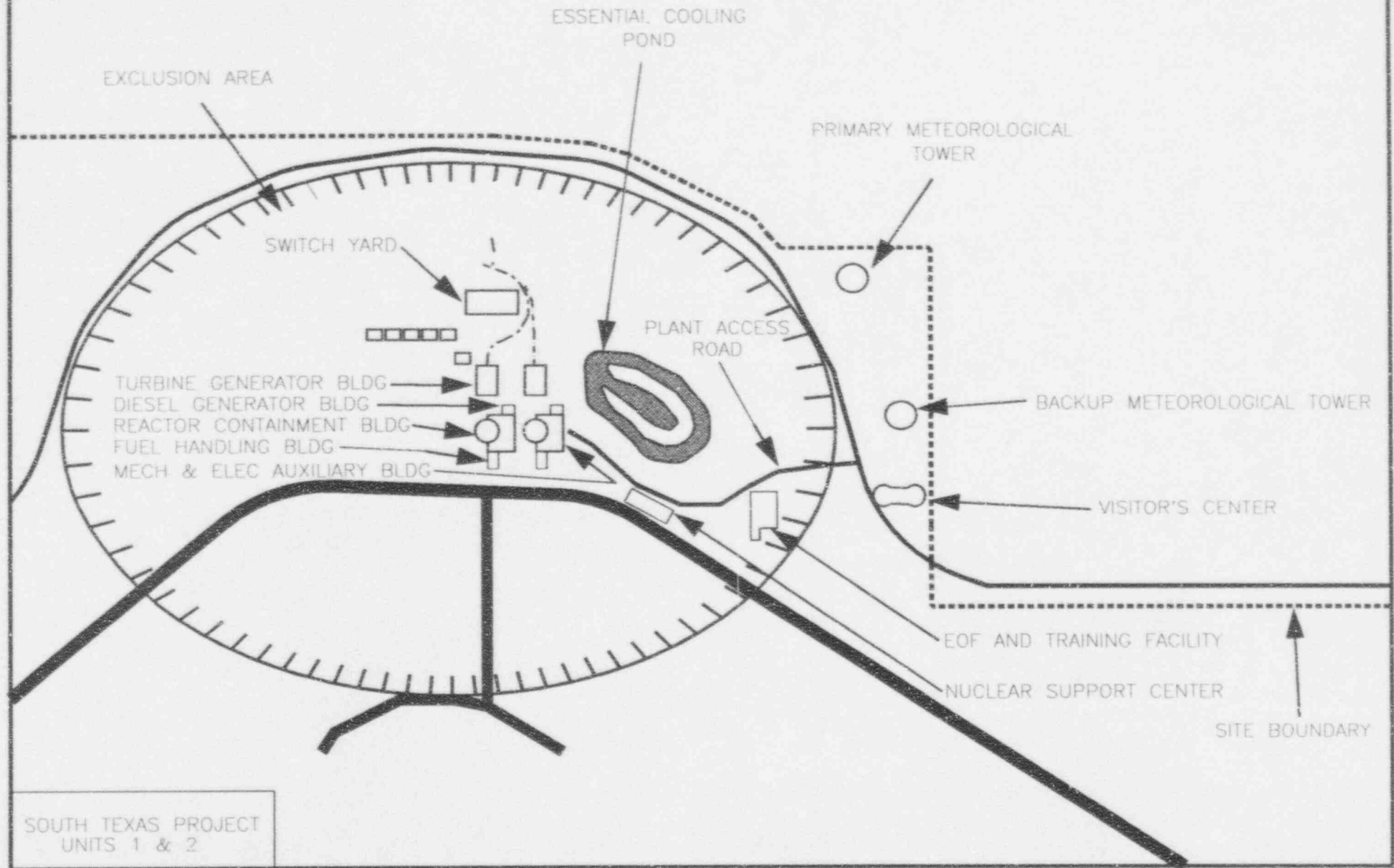


FIGURE 5.1-1  
EXCLUSION AREA

E-0008.DWG  
REV 2

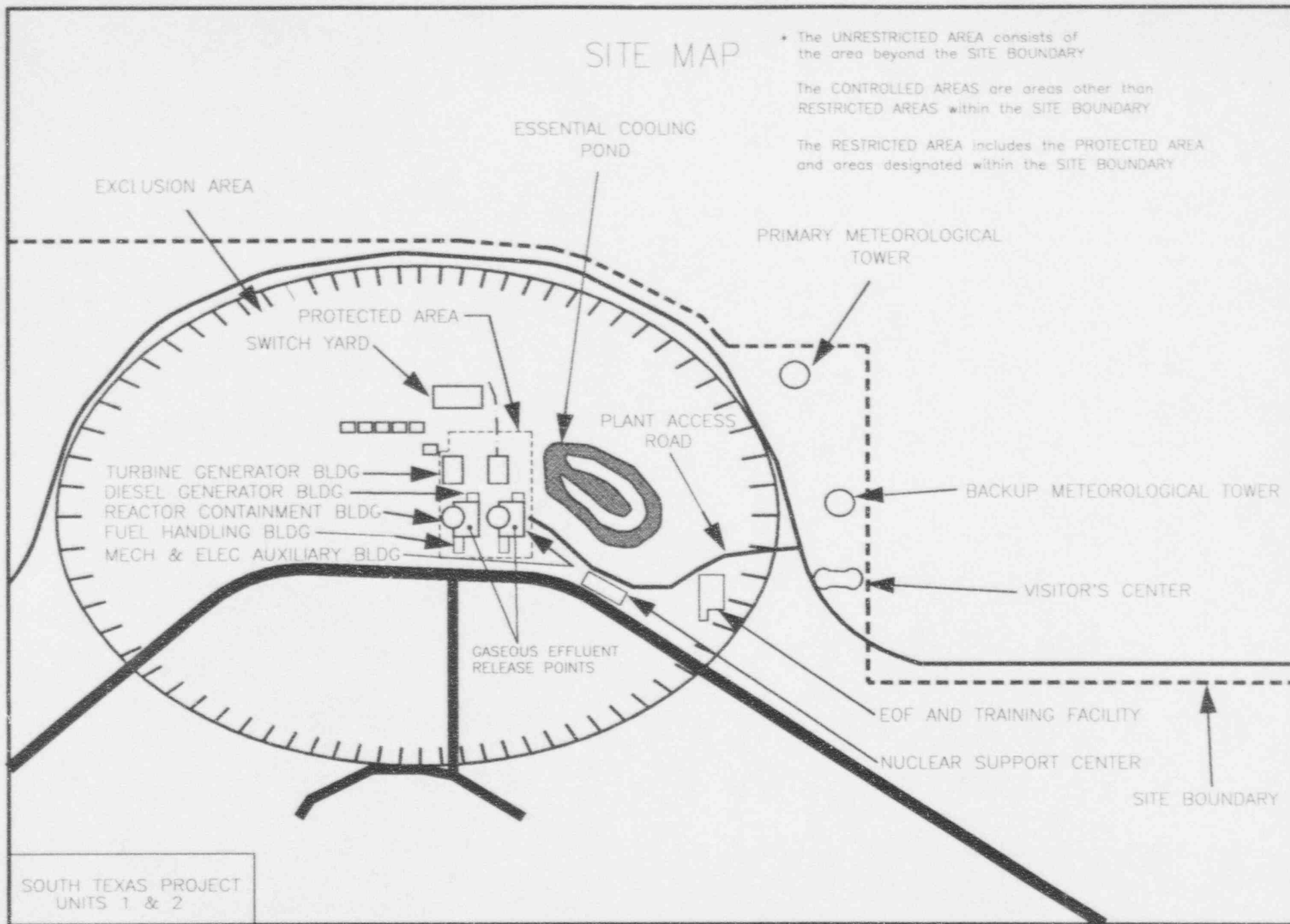


FIGURE 5.1-3  
UNRESTRICTED AREA\* AND SITE BOUNDARY FOR RADIOACTIVE GASEOUS EFFLUENTS  
(SEE ALSO FIGURE 5.1-4)

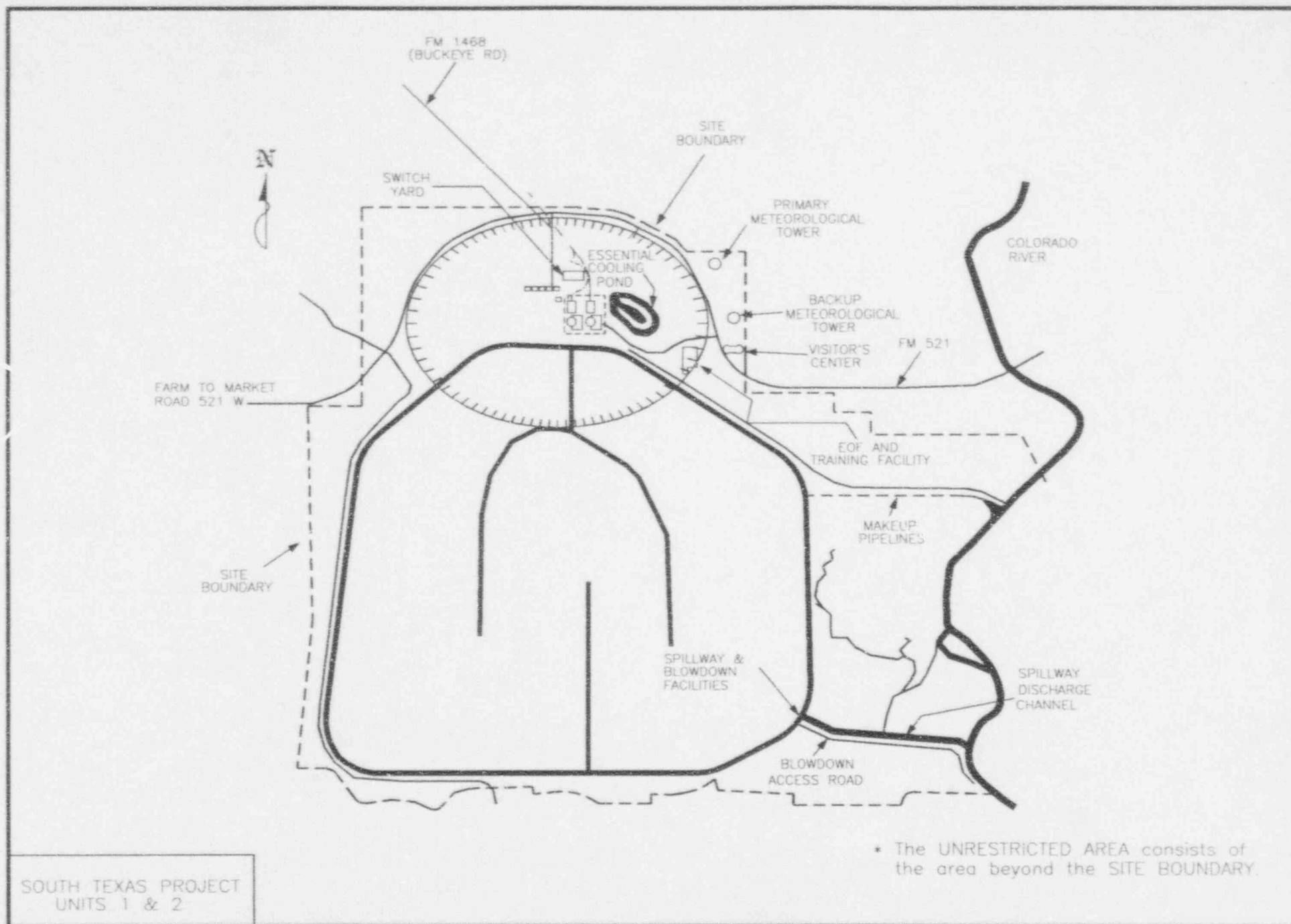


FIGURE 5.1-4  
UNRESTRICTED AREA\* AND SITE BOUNDARY FOR RADIOACTIVE LIQUID EFFLUENTS