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July 15, 1994  
ND3MNO:3594

**Beaver Valley Power Station, Unit No. 2**  
**Docket No. 50-412, Licensee No. NPF-73**  
**LER 94-008-00**

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 94-008-00, 10 CFR 50.73.a.2.i.B, "PORV-Inoperability-Condition Prohibited by Technical Specifications".

L. R. Freeland  
General Manager  
Nuclear Operations

GFZ:clp

Attachment

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cc: Mr. T. T. Martin, Regional Administrator  
United States Nuclear Regulatory Commission  
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## LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block.)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (D)

Beaver Valley Power Station Unit 2

DOCKET NUMBER (2)

05000412

PAGE (3)

1 OF 2

PORV - Inoperability - Condition Prohibited by Technical Specification

EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER	
06	15	94	94	-- 008 --	0	07	15	94	FACILITY NAME	DOCKET NUMBER 05000	
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 20 CFR § (Check one or more) (11)								
1			20.402(b)			20.405(c)			50.73(a)(2)(iv)		73.71(b)
POWER LEVEL (10)			20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)		73.71(c)
100			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)		OTHER
			20.405(a)(1)(iii)			X 50.73(a)(2)(i)			50.73(a)(2)(viii)(A)		(Specify in abstract below and in Text NRC Form 366A)
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)		
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (include Area Code)
L. R. Freeland, General Manager Nuclear Operations	(412) 643-1258

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

[illegible]

## SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE)	X	NO	SUBMISSION DATE (15)			
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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On June 15, 1994 at 1015 hrs, with the unit at 100% power, a pressurizer Power Operated Relief Valve (PORV) 2RCS-455D was removed from service for minor seat leakage using plant procedures. The PORV block valve was closed in accordance with Technical Specification 3.4.11.a. In addition, heat trace was de-energized in accordance with plant procedures.

On June 16, 1994 at 1440 hrs, the valve was additionally determined to be inoperable due to the heat trace being de-energized. Power was then removed from its associated block valve in accordance with Technical Specification 3.4.11.b

Since Technical Specification 3.4.11.b requires power be removed from the associated block valve within 1 hour and power was not removed until the following day, a condition prohibited by Technical Specifications resulted.

There were no safety implications due to this event. Heat Trace is required to minimize downstream piping stress during PORV discharge. Because the PORV block valve was closed, no discharge to downstream piping could occur.

This event is being reported in accordance with 10CFR50.73(a)(2)(i)B as a condition prohibited by Technical Specification.

**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)			PAGE (3)
Beaver Valley Power Station Unit 2		05000412		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 2
				94	-008-	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

**DESCRIPTION OF EVENT**

On June 15, 1994 at 1015 hrs, with the unit at 100% power, a pressurizer Power Operated Relief Valve (PORV) 2RCS-455D developed minor seat leakage. The action statement of Technical Specification 3.4.11.a was invoked, requiring the closing of the associated PORV block valve. This Technical Specification does not require power to be removed from the block valve if the PORV was isolated due to only minor seat leakage.

While removing PORV 2RCS-455D from service, procedure 2OM-6.4.Q, "Isolation of a Power Operated Relief Valve" was used. This procedure required that heat trace be de-energized. De-energizing the heat trace circuit will allow the PORV upstream piping to cool rendering the PORV inoperable due to piping stress concerns. PORV inoperability, for reasons other than minor seat leakage, requires that the action statement of Technical Specification 3.4.11.b be invoked. This action statement removes power from the block valve. Procedure 2.6.4.Q did not clearly identify the Technical Specification requirement to remove power from the block valve whenever heat trace is unable to maintain elevated piping temperatures.

At 1500 hrs, power was removed from its associated block valve in accordance with Technical Specification 3.4.11.b.

Since Technical Specification 3.4.11.b required power to be removed from the associated block valve within 1 hour (June 15 at 1115 hrs.), and power was not removed until the following day (June 16 at 1500 hrs.), this event is being reported in accordance with 10CFR50.73(a)(2)(i)B as a condition prohibited by the plant's Technical Specifications.

**CAUSE OF EVENT**

The primary cause of this event was a deficiency in procedure 2OM-6.4.Q "Isolation of a Power Operated Relief Valve". The procedure failed to caution the operator that de-energizing the heat tracing would render the PORV inoperable.

**CORRECTIVE ACTIONS**

1. The procedure deficiency will be corrected.
2. This event will be reviewed with licensed operators.

**REPORTABILITY**

This report is being submitted in accordance with 10CFR50.73(a)(2)(i)B as a condition prohibited by the plant's Technical Specifications.

**SAFETY IMPLICATIONS**

There were no safety implications due to this event. Heat trace is required to minimize downstream piping stress during PORV discharge. Because the PORV block valve was closed, no discharge to downstream piping could occur.

**SIMILAR EVENT**

There are no previously reported similar events.