



Commonwealth Edison
LaSalle County Nuclear Station
2601 N. 21st. Rd.
Marseilles, Illinois 61341
Telephone 815/357-6761

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U.S. Nuclear Regulatory Commission
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Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for June 1994.

D. J. Ray
Station Manager
LaSalle County Station

DJR/tmb

Enclosure

cc: John B. Martin, Regional Administrator - Region III
NRC Senior Resident Inspector - LaSalle
IL Department of Nuclear Safety - LaSalle
NRR Project Manager - LaSalle
GE Representative - LaSalle
Regulatory Assurance Supervisor - LaSalle
Manager of Nuclear Licensing - Downers Grove
Nuclear Fuel Services Manager - General Office
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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JUNE 1994

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

TABLE OF CONTENTS
(UNIT 1)

I. INTRODUCTION

II. REPORT

A. SUMMARY OF OPERATING EXPERIENCE

B. AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

C. LICENSEE EVENT REPORTS

D. DATA TABULATIONS

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

E. UNIQUE REPORTING REQUIREMENTS

1. Main Steam Safety Relief Valve Operations
2. Major Changes to Radioactive Waste Treatment System
3. Static O-Ring Failures
4. Off-Site Dose Calculation Manual Changes

I. INTRODUCTION (UNIT 1)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor sub-critical, Generator off-line, refuel outage (L1R06) in progress.
23	0018	Reactor critical.
27	2047	Generator on-line at 50 Mwe.
	2200	Power level at 151 Mwe.
28	1141	Generator off-line for turbine overspeed trip test.
	1245	Generator on-line at 50 Mwe.
29	1900	Power level at 200 Mwe.
	2108	Power level at 156 Mwe.
30	2400	Reactor critical, Generator on-line at 160 Mwe.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION

Deletion of the Primary Containment structural integrity tendon testing from the Technical Specifications.

C. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
94-008	06/07/94	Testing of standby liquid control system discharge safety relief valve.
94-009	06/03/94	Shutdown cooling isolation due to high suction flow.

D. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 1)
2. Average Daily Unit Power Level (See Table 2)
3. Unit Shutdowns and Significant Power Reductions (See Table 3)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 1)

1. Safety Relief Valve Operations
(None)
2. Major Changes to Radioactive Waste Treatment Systems
(None)
3. Static O-Ring Failures
(None)
4. Changes to the Off-Site Dose Calculation Manual
(None)

TABLE 1
D.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE July 8, 1994
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD:	June 1994	GROSS HOURS IN REPORTING PERIOD	720
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):	3,323	MAX DEPEND CAPACITY (MWe-Net):	1,036
		DESIGN ELECTRICAL RATING (MWe-N	1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

4. REASONS FOR RESTRICTION (IF ANY):

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	191.7	1,150.0	62,476.9
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	74.2	1,020.5	61,034.2
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWht)	53,191	3,012,603	180,338,240
10. ELECTRICAL ENERGY GENERATED (MWhe-Gross)	11,282	1,011,308	60,265,679
11. ELECTRICAL ENERGY GENERATED (MWhe-Net)	1,934	936,907	57,785,661
12. REACTOR SERVICE FACTOR (%)	26.6	26.5	67.9
13. REACTOR AVAILABILITY FACTOR (%)	26.6	26.5	69.7
14. UNIT SERVICE FACTOR (%)	10.3	23.5	66.3
15. UNIT AVAILABILITY FACTOR (%)	10.3	23.5	66.3
16. UNIT CAPACITY FACTOR (USING MDC) (%)	0.3	20.8	60.6
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	0.2	20.0	58.2
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	39.6	8.3

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 2
D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE July 8, 1994
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: June 1994

DAY	POWER	DAY	POWER
1	-12	17	-12
2	-12	18	-12
3	-12	19	-12
4	-12	20	-12
5	-12	21	-12
6	-12	22	-16
7	-12	23	-16
8	-12	24	-15
9	-12	25	-14
10	-12	26	-14
11	-12	27	6
12	-12	28	124
13	-12	29	127
14	-12	30	151
15	-12	31	
16	-12		

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 1)

<u>YEARLY SEQUENTIAL NUMBER</u>	<u>DATE {YYMMDD}</u>	<u>TYPE F: FORCED S: SCHEDULED</u>	<u>DURATION (HOURS)</u>	<u>REASON</u>	<u>METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER</u>	<u>CORRECTIVE ACTIONS/COMMENTS (LER # if applicable)</u>
4	940318	S	645.8	C	4	Refuel outage (L1R06).
5	940628	S	1.0	B	4	Manual turbine trip for performance of overspeed test.

SUMMARY OF OPERATION:

The unit entered the month in a scheduled refuel outage. The unit was returned to service on 06/27/94. On 06/28/94 the main turbine was tripped for the performance of an overspeed test. The unit finished the month at low power.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

JUNE 1994

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

TABLE OF CONTENTS
(UNIT 2)

I. INTRODUCTION

II. REPORT

A. SUMMARY OF OPERATING EXPERIENCE

B. AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

C. LICENSEE EVENT REPORTS

D. DATA TABULATIONS

1. Operating Data Report
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E. UNIQUE REPORTING REQUIREMENTS

1. Main Steam Safety Relief Valve Operations
2. Major Changes to Radioactive Waste Treatment System
3. Static O-Ring Failures
4. Off-Site Dose Calculation Manual Changes

I. INTRODUCTION (UNIT 2)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1130 Mwe.
4	0330	Reduced power level to 1040 Mwe due to system load.
	0530	Increased power level to 1130 Mwe.
21	2230	Automatic reactor scram due to loss of bus 231A and 231B.
25	2055	Reactor critical.
26	1450	Generator on-line at 60 Mwe.
	1608	Power level at 160 Mwe.
27	0600	Power level at 362 Mwe, reactor recirculation pumps upshifted.
	0800	Power level at 610 Mwe.
	1430	Increased power level to 1085 Mwe.
29	0100	Reduced power level to 700 Mwe to perform rod set.
	0900	Increased power level to 1130 Mwe.
30	2400	Reactor critical, Generator on-line at 1130 Mwe.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION

Deletion of the Primary Containment structural integrity tendon testing from the Technical Specifications.

C. LICENSEE EVENT REPORTS (Unit 2)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
94-003	06/16/94	Request for notice of enforcement discretion for critical surveillances.
94-004	06/21/94	Loss of DC power to bus 231A and 231B resulting in an automatic reactor scram.
94-005	06/22/94	Reactor recirculation system piping snubber support deformed.
94-006	06/27/94	Hydraulic control unit low pressure switch out of tolerance exceeding a limiting condition of operation.

D. DATA TABULATIONS (Unit 2)

1. Operating Data Report (See Table 1)
2. Average Daily Unit Power Level (See Table 2)
3. Unit Shutdowns and Significant Power Reductions (See Table 3)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 2)

1. Safety Relief Valve Operations
(None)
2. Major Changes to Radioactive Waste Treatment Systems
(None)
3. Static O-Ring Failures
(None)
4. Changes to the Off-Site Dose Calculation Manual
(None)

TABLE 1
D.1 OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE July 8, 1994
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: June 1994 GROSS HOURS IN REPORTING PERIOD: 720
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):
4. REASONS FOR RESTRICTION (IF ANY):

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	625.6	4,034.1	60,959.7
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	607.7	3,948.4	59,830.7
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWhr)	1,957,800	12,600,003	180,700,456
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	658,782	4,290,617	60,275,257
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	636,157	4,161,211	57,917,102
12. REACTOR SERVICE FACTOR (%)	86.9	92.9	71.7
13. REACTOR AVAILABILITY FACTOR (%)	86.9	92.9	73.7
14. UNIT SERVICE FACTOR (%)	84.4	90.9	70.4
15. UNIT AVAILABILITY FACTOR (%)	84.4	90.9	70.4
16. UNIT CAPACITY FACTOR (USING MDC) (%)	85.3	92.5	65.7
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	82.0	88.9	63.2
18. UNIT FORCED OUTAGE FACTOR (%)	15.6	2.8	10.9

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 2
D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE July 8, 1994
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: June 1994

DAY	POWER	DAY	POWER
1	1,093	17	1,079
2	1,095	18	1,071
3	1,095	19	1,072
4	1,078	20	1,073
5	1,091	21	1,006
6	1,090	22	-16
7	1,088	23	-16
8	1,095	24	-14
9	1,096	25	-14
10	1,094	26	38
11	1,091	27	689
12	1,091	28	989
13	1,091	29	1,017
14	1,090	30	1,084
15	1,089	31	
16	1,083		

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 2)

<u>YEARLY SEQUENTIAL NUMBER</u>	<u>DATE (YYMMDD)</u>	<u>TYPE F: FORCED S: SCHEDULED</u>	<u>DURATION (HOURS)</u>	<u>REASON</u>	<u>METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER</u>	<u>CORRECTIVE ACTIONS/COMMENTS (LER # if applicable)</u>
4	940621	F	112.3	A	3	Reactor scram due to the loss of buss 231A and the 231B.

SUMMARY OF OPERATION:

The unit remained on-line at high power throughout most of the month. The unit experienced a automatic reactor scram on 06/21/94. The unit was returned to service 06/26/94.