



Wisconsin Electric POWER COMPANY

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May 12, 1983

Mr. J. G. Keppler, Regional Administrator
Office of Inspection and Enforcement,
Region III
U. S. NUCLEAR REGULATORY COMMISSION
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

CANCELLATION OF LICENSEE EVENT REPORT NO. 83-003/01T-0
REPORTABLE INDICATIONS IN CLASS 1 PIPING
POINT BEACH NUCLEAR PLANT, UNIT 2

Licensee Code:	WIPBH2	Docket No:	50-301
License Type:	41111	Source:	L
License Number:	00-00000-00	Event Date:	04-13-83

Event Description: During the performance of PBNP's inservice inspection program, conducted in accordance with ASME Section XI, 1977 edition with Summer 1979 addenda, three linear surface indications were found in two different ASME Class 1 piping systems that exceeded the allowable indication lengths set forth in Table IWB-3514-3, ASME Section XI. A 3/8" long surface indication was found in line 4RC-2501R-1, the pressurizer spray line, exceeding the allowable length of 0.14". In line 2CH-2501R-1, the "A" reactor coolant pump (RCP) seal injection line, one 1/2" surface indication and one 3/8" surface indication were found on a coupling in the line, exceeding the allowable length of 1/8". This was the first inservice inspection of these weld areas.

The unit was in a refueling and steam generator sleeving outage at the time of the examination.

The indication found on the pressurizer spray line was reexamined by an SNT Level III dye penetrant test examiner, and it was judged that further surface buffing and minor hand filing was required, based on the general surface condition of the pipe section housing the indication. This was to provide additional information necessary to interpret the indication. The indication was removed by minor

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filing and the SNT Level III examiner evaluated the indication as an irrelevant surface discontinuity. Ultrasonic wall thickness measurements verified that the amount of wall removal was less than allowable per ASTM A376 for initial fabrication acceptance of nominal wall thickness variance.

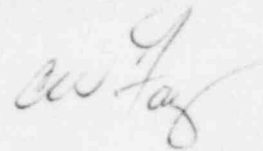
Concerning the two indications found on the 2" reactor coolant pump seal injection line, reexamination by an SNT Level III dye penetrant test examiner revealed the indications to be forging seams on the coupling and thus not service related. The seams have been filed out and the area reexamined to ensure that these seams are not masking significant indications. Acceptable wall thickness was also verified.

The disposition of these indications and follow-up examinations were conducted in accordance with ASME Section XI and Section V.

The event was initially reported in accordance with Technical Specification 15.6.9.2.A.3, "abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment". After the evaluation, investigation was made on both areas of concern, neither was found to violate minimum required wall thickness nor could either be considered an abnormal degradation of the reactor coolant pressure boundary. With respect to Technical Specification 15.6.9.2.A.3, an indication is a reportable abnormal degradation of the reactor coolant pressure boundary when it is determined to be service related and violating minimum required wall thickness mandating repair.

Licensee Event Report No. 83-003/01T-0, for PBNP Unit 2, will be cancelled and for recordkeeping purposes this number will not be reassigned to another event.

Very truly yours,



Vice President-Nuclear Power

C. W. Fay

Copy to NRC Resident Inspector