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 (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

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CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

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PHONE: 414/277-2811

ATTACHMENT TO LICENSEE EVENT REPORT NO. 83-006/03L-0

Wisconsin Electric Power Company
Point Beach Nuclear Plant Unit 2
Docket No. 50-301

On April 19, 1983 during refueling shutdown calibration testing the time delay of one actuating contact from the combination of time delay 2-TDRC/A06 and 4160V degraded grid voltage relay 2-275/A06 was found to be out of specification at 30.5 seconds. This exceeded the time of less than 30 seconds specified in Technical Specification Table 15.3.5-1, Item No. 9.

The affected contact is in a two-out-of-three channel logic matrix which is used to trip breaker 2A52-70, 4160V normal supply breaker to safeguards bus 2A06, when a degraded voltage exists on bus 2A06 for 30 seconds or more. The other two channels tested satisfactorily and provided redundant automatic operability for tripping of breaker 2A52-70. The result of the time delay being less conservative than that established by Technical Specifications would not have prevented the fulfillment of the functional requirement to trip breaker 2A52-70 upon a degraded grid condition on bus 2A06.

The cause for the out-of-specification time delay was apparently due to bumping or mispositioning of the time set knob on relay 2-TDRC/A06 sometime between installation in April 1982 and the first calibration test on April 19, 1983. Relay 2-TDRC/A06 is an Agastat Model No. E7014PD002 time delay relay with adjustable delay from 10 to 100 seconds. It is used in conjunction with the phase "C" voltage setpoint and time delay from 4160V degraded grid voltage relay 2-275/A06 which was found to be within specification.

Agastat time delay relay 2-TDRC/A06 was reset so that the time delay was once again within specification. Subsequent testing proved that the time delay is now satisfactory.

This event is reportable in accordance with Technical Specification 15.6.9.2.B.1 since an engineered safety feature instrument setting was found to be less conservative than that established by Technical Specifications.

Technical Specification Change Request No. 81, submitted June 30, 1982, has proposed to adjust the specified time delay to "less than 60 seconds". Subsequent discussion concerning this time delay has shown that the 60-second time delay is satisfactory for engineered safety feature operation. Therefore, the violation of present Technical Specification Table 15.3.5-1, Item No. 9, should not be considered a safety issue.

The Resident Inspector has been notified of this event.



Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

May 10, 1983

Mr. J. G. Keppler, Regional Administrator
Office of Inspection and Enforcement,
Region III
U. S. NUCLEAR REGULATORY COMMISSION
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

DOCKET NO. 50-301
LICENSEE EVENT REPORT NO. 83-006/03L-0
POINT BEACH NUCLEAR PLANT, UNIT 2

Enclosed is Licensee Event Report No. 83-006/03L-0
(a 30-day report) with an attachment which provides a description
of an event reportable in accordance with Technical Specification
15.6.9.2.B.1, "Reactor protection system or engineered safety
feature instrument settings which are found to be less conservative
than those established by the Technical Specifications but which
do not prevent the fulfillment of the functional requirements of
affected systems."

Very truly yours,

Vice President-Nuclear Power

C. W. Fay

Enclosure

Copy to NRC Resident Inspector

MAY 13 1983

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