

TENNESSEE VALLEY AUTHORITY  
DIVISION OF NUCLEAR POWER  
SEQUOYAH NUCLEAR PLANT

MONTHLY OPERATING REPORT  
APRIL 1, 1983 - APRIL 30, 1983

UNIT 1

DOCKET NUMBER 50-327

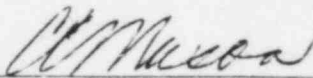
LICENSE NUMBER DPR-77

UNIT 2

DOCKET NUMBER 50-328

LICENSE NUMBER DPR-79

Submitted By:

  
Power Plant Superintendent

8305240323 830509  
PDR ADOCK 05000327  
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## Operations Summary

April, 1983

The following summary describes the significant operational activities for the month of April. In support of this summary, a chronological log of significant events is included in this report.

### Unit 1

Unit 1 was critical for 719 hours, produced 824,830 MWH (gross), resulting in an average hourly gross load of 1,147,191 kW during the month. There are 201.91 full power days estimated remaining until the end of cycle 2 fuel. With a capacity factor of 85 percent, the target EOC exposure would be reached December 25, 1983. The capacity factor for the month was 98.6 percent.

There was no reactor scrams, no manual shutdowns, and three power reductions during April.

### Unit 2

Unit 2 was critical for 719 hours, produced 829,530 MWH (gross), resulting in an average hourly gross load of 1,153,530 kW during the month. There are 78.61 full power days estimated remaining until the end of cycle 1 fuel. With a capacity factor of 85 percent the target EOC exposure would be reached August 1, 1983. The capacity factor for the month was 99.2 percent.

There were no reactor scrams, no manual shutdowns, and one power reduction during April (a continuation from March).

## Significant Operational Events

### Unit 1

<u>Date</u>	<u>Time</u>	<u>Event</u>
04/01/83	0001	Reactor in mode 1 at 100% power producing 1190 MWe.
04/02/83	0530	Began reducing load for the turbine acceptance test.
	0956	Reactor at 60% power. The cards were pulled on #2 and #3 governor valves.
	1040	Began power ascension.

# Significant Operational Events

## Unit 1

(Continued)

<u>Date</u>	<u>Time</u>	<u>Event</u>
04/02/83	1410	Reactor at 75% power producing 883 MWe.
	1715	Began reducing load to replace the cards in #2 and #3 governor valves.
	1826	Reactor at 62% power producing 725 MWe.
	1838	Began power ascension.
04/03/83	0100	Reactor at 100% 1180 MWe.
04/08/83	0838	#3 governor valve closed for the turbine acceptance test. Reactor at 100% power producing 1163 MWe.
	1440	#1 throttle valve was found only 10% open, reactor at 92% power.
	1900	#3 governor valve reopened.
	2030	Reactor at 100% power.
	2107	#1 throttle valve card replaced valve full open.
04/09/83	0630	Began reducing load for the turbine acceptance.
	0910	#2 and #3 governor valves closed. Reactor at 75% power.
	1650	Governor valves reopened, began power ascension.
	2115	Reactor at 100% producing 1190 MWe.
04/12/83	1450	Began reducing T-avg for the turbine acceptance test.
	1510	Reactor at 98% power.
	1935	Began power ascension.
	2220	Reactor at 100% power producing 1186 MWe.

## Significant Operational Events

### Unit 1

(Continued)

<u>Date</u>	<u>Time</u>	<u>Event</u>
04/13/83	1018	Reduced T-Avg for the turbine acceptance test.
	1419	Reactor at 98% power producing 1180 MWe.
	1910	Began power ascension.
	2215	Reactor at 100% power producing 1186 MWe.
04/14/83	1615	Began reducing load for maintenance on #3B heater drain tank pump.
	2035	Reactor at 75%.
04/15/83	0247	#3B heater drain tank pump maintenance completed, began power ascension.
	1420	Reactor at 100% power producing 1190 MWe.
04/24/83	0200	Switched from standard time to daylight savings time.
04/30/83	2359	Reactor in Mode 1 at 100% power producing 1190 MWe.

### Unit 2

<u>Date</u>	<u>Time</u>	<u>Event</u>
04/01/83	0001	Reactor in Mode 1 at 75% power producing 918 MWe. Incore/excore calibration in progress.
	1200	Began power ascension
	1800	Reactor at 98% power, producing 1171 MWE. #3 governor valve closed.
04/06/83	1127	Reduced load for SI-139. Determination of the at power moderator temperature coefficient.

# Significant Operational Events

## Unit 2

(Continued)

<u>Date</u>	<u>Time</u>	<u>Event</u>
04/06/83	1133	Reactor at 94% power producing 1145 MWe.
	1152	Began power ascension
	1157	Reactor at 98% power producing 1170 MWe.
	1212	Began reducing load for SI-139.
	1217	Reactor at 94% power producing 1145 MWe.
	1237	Began power ascension.
	1245	Reactor at 98% power producing 1179 MWe.
	1305	Began load reduction for SI-139.
	1310	Reactor at 94% power producing 1145 MWe.
	1315	Began power ascension
	1320	Reactor at 98% power producing 1170 MWe.
	1322	Reducing power for SI-139.
	1327	Reactor at 94% producing 1145 MWe.
	1342	Began power ascension.
	1345	Reactor at 98% power producing 1170 MWe.
	1352	Reducing power for SI-139.
	1357	Reactor at 94% power producing 1145 MWe.
	1402	Began power ascension.
	1407	Reactor at 98% power producing 1170 MWe. SI-139 complete.
04/24/83	0200	Switched from Standard Time to Daylight Savings Time.
04/30/83	2359	Reactor in Mode 1 at 98% power producing 1170 MWe. #3 governor valve closed.

## PORV'S and Safety Valves Summary

No PORV's or safety valves were challenged during the month.

### Licensee Events and Special Reports

The following Licensee Event Reports (LER's) were sent during April 1983, to the Assistant Director of Nuclear Power (Operations) for reporting to the Nuclear Regulatory Commission.

#### Unit 1

<u>LER</u>	<u>SUBJECT</u>
SQR0-50-327/83045	No. 1 steam generator wide range level channel failed low due to defective switches in the connector of recorder 1-LR-3-43.
SQR0-50-327/83047	Reactor coolant system subcooling margin monitor declared inoperable due to the plant computer failing due to memory parity error.
SQR0-50-327/83048	During review of SI-111 performed 09/11/82, safety relief valves 1-RV-1-512, -513, -518, -515, and -521 did not meet the required setpoint criteria. During the performance of SI-111 on 01/08/83 safety relief valves 1-RV-1-525 and -527 did not meet the specified setpoint criteria.
SQR0-50-327/83049	Ice condenser intermediate deck door in bay 11 found frozen closed. Ice resulted from leaks in the condensate drain lines of AHU 13 and 14.
SQR0-50-327/83052	During the performance of SI-196, 'Valve Closure Verification Test of Upper Head Injection System Hydraulic isolation Valves', 2 of 4 level switches were out of tolerance on April 7, 1983.
SQR0-50-327/83053	On 03/19/83 steam generator blowdown isolation valve 1-FCV-1-14 would not stay open and the post maintenance stroke test was not accomplished.
SQR0-50-327/83054	Containment isolation valve 1-FCV-1-182 failed to open during the performance of special test SQ-STEAR-INST-83-03.

#### Unit 2

SQR0-50-328/83044	UHI level switches LS-87-21, -22, -23, -24 found out of tolerance during the performance of SI-196 on March 24, 1983.
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Licensee Events and Special Reports

(Continued)

Unit 2

<u>LER</u>	<u>SUBJECT</u>
SQRO-50-328/83046	Reactor coolant system subcooling margin monitor was declared inoperable due to the plant computer failing caused by a faulty area on the disc.
SQRO-50-328/83050	Control rod D-8 failed to move with the rest of the bank upon demand and became misaligned more than 12 steps. Troubleshooting indicates a lift coil inoperable due to a loose connector at the vessel head.
SQRO-50-328/83051	2-FCV-63-40 BIT inlet isolation valve would not operate during the performance of SI-166.1 due to dirty contactors in the motor starter.

Special Reports

There were no special reports transmitted during the month of April.

Offsite Dose Calculation Manual Changes

There were no changes to the ODCM during the month.



## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-327  
 UNIT NAME Sequoyah One  
 DATE May 8, 1983  
 COMPLETED BY M. Eddings  
 TELEPHONE (615) 870-6543

REPORT MONTH APRIL

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method Of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
15.	830402	S	0	B	5				STEAR Test on the Turbine.
16.	830409	S	0	B	5				STEAR Test on the Turbine.
17.	830414	F	0	B	5				Maint on 3B heater drain tank pump.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Cont. of Existing  
 Outage  
 5-Reduction  
 9-Other

<sup>4</sup>  
 Exhibit G-Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File (NUREG-  
 0161)

<sup>5</sup>  
 Exhibit I-Same Source

(9/77)

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-327  
 UNIT 1  
 DATE May 3, 1983  
 COMPLETED BY D. Patterson  
 TELEPHONE (615) 870-6543

MONTH APRIL

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>1143</u>
2	<u>941</u>
3	<u>1136</u>
4	<u>1143</u>
5	<u>1139</u>
6	<u>1136</u>
7	<u>1141</u>
8	<u>1111</u>
9	<u>989</u>
10	<u>1146</u>
11	<u>1141</u>
12	<u>1145</u>
13	<u>1142</u>
14	<u>1059</u>
15	<u>1085</u>
16	<u>1141</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>1141</u>
18	<u>1141</u>
19	<u>1140</u>
20	<u>1140</u>
21	<u>1140</u>
22	<u>1140</u>
23	<u>1142</u>
24	<u>1144</u>
25	<u>1142</u>
26	<u>1140</u>
27	<u>1141</u>
28	<u>1140</u>
29	<u>1137</u>
30	<u>1136</u>
31	<u>N/A</u>

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

# OPERATING DATA REPORT

DOCKET NO. 50-327  
DATE MAY 6, 1993  
COMPLETED BY M. G. EDDINGS  
TELEPHONE (615) 570-6249

## OPERATING STATUS

1. UNIT NAME: SECONUM NUCLEAR PLANT, UNIT 1
2. REPORT PERIOD: APRIL 1 - 30, 1993
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEFENDABLE CAPACITY (GROSS MWE): 1163.0
7. MAXIMUM DEFENDABLE CAPACITY (NET MWE): 1128.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBERS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: \_\_\_\_\_
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): \_\_\_\_\_
10. REASONS FOR RESTRICTIONS, IF ANY: \_\_\_\_\_

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	719.00	2879.00	14056.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	719.00	2240.85	9876.55
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	719.00	2261.70	9581.20
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MMB) 2401824.27		7211705.36	30494447.36
17. GROSS ELECTRICAL ENERGY GEN. (MMH) 824830.00		2490020.00	10247556.00
18. NET ELECTRICAL ENERGY GENERATED (MMH) 795714.00		2396001.00	9812095.00
19. UNIT SERVICE FACTOR 100.00		78.56	59.67
20. UNIT AVAILABILITY FACTOR 100.00		78.56	59.67
21. UNIT CAPACITY FACTOR (USING MDC NET) 98.23		73.73	54.29
22. UNIT CAPACITY FACTOR (USING DER NET) 96.52		72.50	53.34
23. UNIT FORCED OUTAGE RATE 0.00		6.34	15.36
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: \_\_\_\_\_

NOTE THAT THE YR.-TO-DATE AND CUMULATIVE VALUES ARE NOT UPDATED.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-328

UNIT NAME Sequoyah Two

DATE May 9, 1983

COMPLETED BY David Dupree

TELEPHONE (615) 870-6543

REPORT MONTH APRIL

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method Of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
9	830330	F	0	B	5				Incore/Excore calibration, RX@ 75%. (Continuation of previous reductions.)

<sup>1</sup>  
F: Forced  
S: Scheduled

<sup>2</sup>  
Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

<sup>3</sup>  
Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Cont. of Existing  
Outage  
5-Reduction  
9-Other

<sup>4</sup>  
Exhibit G-Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File (NUREG-  
0161)

<sup>5</sup>  
Exhibit I-Same Source

(9/77)

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-328  
UNIT 2  
DATE May 9, 1983  
COMPLETED BY David Dupree  
TELEPHONE (615) 870-6543

MONTH APRIL

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	968	17	1119
2	1121	18	1117
3	1122	19	1118
4	1121	20	1120
5	1109	21	1119
6	1116	22	1116
7	1119	23	1121
8	1114	24	1121
9	1120	25	1119
10	1120	26	1117
11	1118	27	1125
12	1117	28	1121
13	1121	29	1122
14	1119	30	1121
15	1121	31	N/A
16	1119		

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

# OPERATING DATA REPORT

DOCKET NO. 50-328  
DATE 5-9-83  
COMPLETED BY D.C. DUPREE  
TELEPHONE (615) 970-6543

## OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 2
2. REPORT PERIOD: APRIL 1 THRU 30, 1983
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1163.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1128.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBERS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: \_\_\_\_\_

NOTES:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): \_\_\_\_\_

10. REASONS FOR RESTRICTIONS, IF ANY: \_\_\_\_\_

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	719.00	2879.00	8016.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	719.00	2866.22	6754.92
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	719.00	2831.40	6638.15
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MMH)	2411559.45	9278395.52	21432996.32
17. GROSS ELECTRICAL ENERGY GEN. (MMH)	829530.00	3202930.00	7284780.00
18. NET ELECTRICAL ENERGY GENERATED (MMH)	800593.00	3099889.00	7016178.60
19. UNIT SERVICE FACTOR	100.00	98.35	82.81
20. UNIT AVAILABILITY FACTOR	100.00	99.35	82.81
21. UNIT CAPACITY FACTOR (USING MDC NET)	98.71	95.15	77.60
22. UNIT CAPACITY FACTOR (USING DER NET)	96.99	93.49	76.24
23. UNIT FORCED OUTAGE RATE	0.00	1.65	10.99

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
Refueling outage to begin approximately August 5, 1983.

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: \_\_\_\_\_

NOTE THAT THE THE YR.-TO-DATE AND  
CUMULATIVE VALUES HAVE BEEN UPDATED.

## Plant Maintenance Summary

The following significant maintenance items were completed during the month of April 1983:

### MECHANICAL MAINTENANCE

1. Repaired a steam leak on level switch 2-LS-6-104B on No. 3 Heater Drain Tank, Unit 2.
2. Repaired leaks on letdown radiation monitor 1-RE-90-104, Unit 1.
3. Installed the rebuilt condenser vacuum pump 1C.
4. Rebuilt & replaced parts on auxiliary air compressor "A".
5. Installed the cooling tower blowdown radiation booster pump and cooling tower blowdown flow transmitter.
6. Installed a temperature control valve on the glycol chiller room cooler.
7. Installed a temporary line to the ERCW station from the Hypochlorite Building to inject hypochlorite to the ERCW Pump Pits.

### Electrical Maintenance

1. Continued installation of the Dimension 2000 phone system.
2. Continued systematic walk down and inspection of the E-field wiring and made necessary repairs.
3. Continued repairs on paging system to fulfill INPO commitment.
4. Continued modification work on the spare RCP motor upper bearings.
5. Began major repairs on the CTL pump 1B motor and added new sight-glass to oil reservoirs of several other CTL pump motors.
6. Repaired transformer cooling fans.
7. Continued repairs on 6.9 kV SD BD breakers.
8. Painted in plant DI sand filter A.
9. Investigated D-8 control rod drive problem on Unit 2.
10. Completed repairs to the Unit 1 and Unit 2 heater drain level switches.



## Plant Maintenance Summary

(Continued)

### Instrument Maintenance

1. Continued monthly calibration surveillance on the Unit 1 and 2 UHI level switches. Two of the four switches were found out of tolerance on Unit 1 on April 7, 1983 and three of the four switches were found out of tolerance on Unit 2 on April 21, 1983. A new calibration procedure was used on Unit 2 which resulted in no switch being more than .9' H<sub>2</sub>O out of tolerance. All switches were returned to service after the surveillance was completed. Expecting Tech Spec change by next month's report.
2. On Unit 1, control bank D failed to move on demand after receiving a rod control system urgent alarm. Initial indication was a phase failure in the lift circuit of power cabinet 1BD. The problem was found to be the "C" phase reference voltage fuse (FU-10) blown. The fuse was replaced and rod control system verified operable within 2 hours.
3. Completed removal of the status monitoring system.
4. Completed modification on the fuel handling fan damper controls in an effort to improve the auxilliary building pressure control problem. This continues to be a problem in the fact that the pressure controllers are not designed to recover from a transient condition such as aux. bldg. isolation. DCR 1717 will need to be completed before this problem can be resolved.
5. Aux. feedwater level control valve 2-LCV-3-156 failed during performance of SI-276. The valve would not travel full open on demand. Investigation found that the valve positioner installed was for a 3-15 psi output and should have been for a 60 psig supply pressure. An inspection was made for both units and found one other positioner for 1-LCV-3-164 to have the incorrect range specified on the nameplate data. Both positioners were changed out by ones obtained from WBNP unit 1, verified operable, and returned to service. The valve on unit 1 was operable at the time of changeout and had not experienced any stroke problems.
6. Initiated draft revisions for surveillance test to extend the time interval required to conduct analog channel functional test of the engineered safety features and reactor trip systems from one month to three months per a Technical Specification change.
7. Removed 75% of the Oak Ridge National Lab online analyzer. Will complete removal next month.
8. Replaced the Hathaway data logger printer in main control room with a new DEC printer.



## Plant Maintenance Summary

(Continued)

9. Unit 1 EHC #1 throttle valve started drifting closed several times. Found problem on valve test circuit card which could not be repaired on line. Jumpered output to keep valve from closing on its own. This did not affect any normal or safety functions of the valve, only the test circuit output.
10. Started working on remaining initial instrument calibrations and loop test to be done on the CDWE system.

## Field Services Group

1. ECN 5510--Containment Personnel Airlock Penetrations (Units 1 and 2)

Post-modification functional testing and adjustment of the containment personnel airlock door annunciation limit switches for both units was conducted this month. The unit 1 switches functioned satisfactorily. Work is continuing to adjust the unit 2 switches.

2. ECN 5634--High-Pressure Feedwater Heater Solenoid Relief Valves (Units 1 and 2)

Painting of hangers has been completed and insulation work is continuing.

3. ECNs 2780/5200--Post-Accident Sampling Facility (Units 1 and 2)

The installation of conduit and junction boxes is continuing on elevations 706 and 714 in the auxiliary building. Core drilling in the auxiliary building along the A5 line wall is completed and two sleeves have been installed. Grouting of these sleeves is in progress. Prefabrication of the HVAC ductwork serving the PASF area is continuing and some ductwork has been installed. Installation of HVAC equipment serving both units is in progress in the PASF area. The installation of component cooling piping to the PASF coolers on unit 2 is in progress. The component cooling piping tie-in to the system will require a unit outage. Work is continuing to install the post-accident sample tubing in the unit 1 room. Work has begun to install the unit 2 post-accident sample tubing with work concentrating in the unit 2 annulus. Work is also underway to fabricate the steel containment vessel penetration plates for the post-accident sample tubing to the primary containment. Tie-in of the demineralized water piping serving the unit 2 PASF equipment to the demineralized water system is scheduled to be made soon. This will complete work on this portion of the PASF.

## Plant Maintenance Summary

(Continued)

4. ECN 5096--Common Station Service Transformer (CSST) "C"

Cable pulling is continuing to 161-kV switchyard bay 7 and 14, and CSST "B" with approximately 95 percent of the work completed. Termination and lay down of control cables for the CSST "B" has begun and is approximately 10-percent complete.

5. ECN 5429--Containment Hydrogen Mitigation System (Unit 2)

Installation of conduit and junction boxes in upper containment (excluding the ice condenser) has begun. Fabrication of hanger supports for conduit in the ice condenser is in progress and approximately 75-percent complete. Installation of conduit of elevation 759 in the auxiliary building is in progress. Preparation for installing the hydrogen mitigation system electrical equipment and annunciation in the control and auxiliary buildings is underway.

6. ECN 5009--ERCW Piping Changeout (Units 1 and 2)

Fabrication of small diameter ERCW piping serving the penetration room coolers 2A1, 2A2, 2A3, 2B1, 2B2, and 2B3 was completed during April. Installation of piping for the unit 2 "B" train penetration room coolers is in progress.

7. ECN 5645--Steam Generator Blowdown Heat Exchangers

This ECN is to install first and second stage heat exchangers in the steam generator blowdown system for both units 1 and 2. Installation of piping and hangers is continuing for the unit 1 heat exchangers but has been delayed because of the lack of sufficient hanger design. Fabrication and installation of piping, hangers, and flow elements on elevation 662 of unit 2 turbine building is in progress.

8. ECN 5451--Diesel Generator Engine Lube Oil System

Work performed this month was the functional testing of oil pumps and pressure switches.

9. ECN 5104--Main Steam Reheater (MSR) Steam Bypass (Unit 1)

The frames are installed for the extension of the three MSR housings (dog houses) around the newly installed MSR steam bypass pipes. Fabrication of the housing sheetmetal covers is complete, but installation of the covers will be delayed until all work associated with bypass valves is completed. All conduit serving the MSR bypass valves is installed.

## Plant Maintenance Summary

(Continued)

10. ECN 5495--FSG Office Building Power Supply

Work required for occupancy of the new building was completed this month. Installation of miscellaneous and communication cables for the building is continuing. Cable trays are being fabricated for installation in the turbine building.

11. ECN 5106--Reactor Vessel Level Indication System (Units 1 and 2)

Work is continuing to install conduit outside the reactor building. The majority of this conduit that can be installed at this time is in place. Work has begun to install cables for this system outside containment as well.

12. ECN 5237--Laundry Room Improvements

Work is continuing to install the remaining laundry room equipment and associated mechanical systems. All equipment has been placed in position except for the washing machine and sink which are currently awaiting installation of the drain serving this equipment. Remaining work includes extensive modification of the laundry room ventilation system, building a masonry divider wall, and completion of all conduit and power cables serving the facility.

13. ECN 5596--Install Batch Neutralization Tank

Piping and hanger installation is continuing unimpeded in the turbine building with an overall completion estimate of 40 percent. There has been 99 of 175 feet of 6-inch pipe, 126 of 260 feet of 3-inch pipe, and 60 of 275 feet of 1-inch pipe installed.

14. ECN 5198--Locate and Design A Technical Support Center (TSC)

The TSC room is structurally complete as the drop ceiling has been installed and touchup to the floors and walls is complete. All HVAC system work in the TSC room is complete and work on this system in the existing relay room is nearing completion. The fire protection system piping is completely installed in the TSC room, but work remains to be completed in the relay room. Lighting for the TSC area is complete except for miscellaneous work items to finish the system installation. The major work remaining in the TSC room is installation of the fire detection system, computer terminals, and carpet. Work is scheduled to continue throughout the spring and summer to install electrical equipment, conduit, and cable for the TSC.

## Plant Maintenance Summary

(Continued)

15. ECN 5460--Reroute Auxiliary Feedwater Pump Seal Water to Turbine Building (Units 1 and 2)

All mechanical work is complete to install this drain system. Final painting and gout touchup in the unit 1 auxiliary feedwater ferry turbine pump room remains to be completed.

16. ECN 5726--Reroute Feedwater Flow Sense Lines (Units 1 and 2)

All painting was completed during this reporting period. This completes this ECN.

17. ECN 5642--Add N<sub>2</sub> Regulating Station To Supply N<sub>2</sub> To Deaeration Distribution System Inside Condensate Storage Tanks

The nitrogen supply piping to the tanks remain in work between the new liquid nitrogen supply station and the condensate storage tanks. This piping is installed except for tie-in to the tanks, internal valve cabinet piping installation, and completion of the yard piping installation between the liquid nitrogen supply station and the unit 2 No. 1 valve cabinet. Conduit and cable installation began this month.

18. ECN 5119--Radiation Monitors For Containment Isolation Waste Disposal System (Unit 1)

The rerouting of radiation monitor cables to place them in conduit for the condenser vacuum exhaust and steam generator blowdown systems was completed during this time period. This does not complete this ECN.

19. ECN 2773--Install Radiation Detectors (Unit 2)

Work was completed this month to install radiation detectors, local indications, and conduit for the unit 2 condenser vacuum exhaust area monitors. Pulling of cables through these conduits is underway. Installation of conduit for the unit 2 reactor building ventilation radiation monitor is in progress as well.

## Plant Maintenance Summary

(Continued)

20. ECN 2923--RHR Lines Radiation Monitors (Unit 2)

Conduit installation and cable pulling for these monitors is nearing completion. Remaining work includes final cable pulling in the RHR pump rooms, terminations at the monitors, and the main control room panels 2-M-30 and 2-M-31 which are not yet installed.

21. ECN 5582--Add N<sub>2</sub> Regulating Stations To Supply N<sub>2</sub> To Steam Generators

This system is supplied with nitrogen by the same system that supplies nitrogen to the condensate storage tanks as discussed in this report (ECN 5642). In addition to the outstanding work described under the ECN 5642 portion of this report, the nitrogen piping inside the unit 2 main steam valve rooms remain to be completed as well as excavation and installation of the unit 1 yard piping.

22. ECN 5248--Nitrogen Truck Fill

This work is to relocate the nitrogen truck fill pipes from the north to the east side of the auxiliary building. All pipe hanger unistruts are mounted and the pipe fabrication is complete.

23. ECN 5086--Turbogenerator Floor Handrails

Installation of handrails adjacent to the Nos. 1 and 3 governor valves for both units 1 and 2 turbogenerators was completed this month.

24. ECN 5519--Plant Crane Escape Devices

Structural members used to attach the escape devices to the two turbine building cranes above the elevation 732 floor have been installed. These members are also to be installed on the polar cranes in the reactor buildings as soon as authorization is granted to perform this work.

25. ECN 5449--6.9-kV Shutdown Board Degraded Voltage Circuitry

Work began this month to pull 25 cables between the following panels: 6.9-kV shutdown boards, 6.9-kV logic panels, main control room communication panel 1-M-21, and the auxiliary control room annunciation panel 0-L-4.

26. ECN 5780--Upper Head Injection Water Accumulator Tank Sample Tubing Extension

Work was completed this month in the additional equipment building to extend sample tubing from the upper head injection water accumulator tanks to newly installed sample sinks. This work was completed for both units this reporting period.



## Plant Maintenance Summary

(Continued)

27. ECN 5607--Plant Overhead Door Control Circuitry

The plant overhead door control circuits were modified during this reporting period but will not be functionally tested until completion of all maintenance on the doors.

28. ECN 2775--Containment High Range Area Radiation Monitors

Work began this month to pull cables outside containment for these monitors. Completion of the containment cable pulling, radiation detector installation, final conduit installation, and cable terminations cannot be accomplished until an outage period.

29. ECN 5867--Fuel Transfer System

Decontamination of the fuel transfer system was accomplished this month in preparation of modifications to the system equipment. Because of insufficient design information, this modification will only be partially implemented prior to the unit 2 cycle 1 refueling outage.

30. LDCR 1847--Chemical Feed Injection to Condensate Polisher discharge

All hangers have been installed and tubing installation is in progress for tie-in of the chemical feed system to the condensate polisher discharge piping. This job has an overall completion estimate of 45 percent.

31. LDCR 1883--Liquid Nitrogen Station

Excavation, form erection, and rebar installation have been completed this month in preparation for pouring the liquid nitrogen station concrete pad.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant  
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Daisy, Tennessee 37319

MAY 13 1983

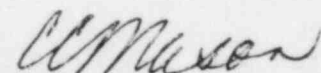
Nuclear Regulatory Commission  
Office of Management Information  
and Program Control  
Washington, DC 20555

Gentlemen:

Enclosed is the April 1983 Monthly Operating Report for Sequoyah Nuclear Plant.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

  
C. C. Mason  
Power Plant Superintendent

Enclosure

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