

LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | At 1330 hours, on April 12, 1983, the 1-1001-65C Residual Heat Removal (RHR) Service

0 3 | Water pump was isolated and taken out of service to facilitate the repair of a

0 4 | broken coupling on the drain line of the 1-5745-C RHR Service Water Vault Room

0 5 | Cooler. All surveillances which are required by Technical Specification 4.5.B.2

0 6 | were successfully performed prior to taking the pump out of service. The 'A' RHR

0 7 | Containment Cooling Loop, all ECCS Systems, and the Diesel Generators were all

0 8 | operable at this time. Therefore, safe Reactor operation was not affected.

09		SYSTEM CODE S B		CAUSE CODE E	CAUSE SUBCODE C	COMPONENT CODE P I P E X X		COMP. SUBCODE A	VALVE SUBCODE Z
7	8	9	10	11	12	13	14	15	16
LER/RO REPORT NUMBER 17		EVENT YEAR 8 3			SEQUENTIAL REPORT NO. 0 1 7		OCCURRENCE CODE /	REPORT TYPE L	REVISION NO. 0
21	22	23	24	25	26	27	28	29	30
ACTION TAKEN A	FUTURE ACTION Z	EFFECT ON PLANT Z	SHUTDOWN METHOD Z		HOURS 0 0 0 0	ATTACHMENT SUBMITTED N	NPRD-4 FORM SUB. 11	PRIME COMP. SUPPLIER N	COMPONENT MANUFACTURER X 9 9 9
33	34	35	36	37	38	39	40	41	42
18	19	20	21	22	23	24	25	26	27

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The 3/8 inch hydraulic hose coupling broke due to an over-pressurization of the

1 1 | piping when the pump was started. The high pressure was caused by the failure of

1 2 | the RHR heat exchanger discharge valve to open. This event is being reported in

1 3 | LER/RO 83-18/03L. A socket-weld coupling and a section of carbon steel piping was

1 4 | installed. The pump was returned to service and satisfactorily tested

at 0040 hours on April 14, 1983.

FACILITY STATUS (8) 7 8 9
[1][5] E (28) % POWER (10) [1][0][0] (29) OTHER STATUS (30) NA (44)
METHOD OF DISCOVERY (45) [A] (31) Operator Observation (32) (80)

ACTIVITY CONTENT RELEASED OF RELEASE (10) [Z] (33) AMOUNT OF ACTIVITY (35) NA (44)
LOCATION OF RELEASE (36) (80)

PERSONNEL EXPOSURES NUMBER (9) [0][0][0] (37) TYPE (12) [N] (38) DESCRIPTION (39) NA (80)

PERSONNEL INJURIES NUMBER (9) [0][0][0] (40) DESCRIPTION (41) NA (80)

LOSS OF OR DAMAGE TO FACILITY TYPE (9) [Z] (42) DESCRIPTION (43) NA (80)

PUBLICITY ISSUED (9) [N] (44) DESCRIPTION (45) 8305240317 830510 PDR ADOCK 05000254 S PDR (80)

NRC USE ONLY (68-80)

NAME OF PREPARER W Leaverton

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NJK-83-168

May 10, 1983

J. Keppler, Regional Administrator
Office of Inspection and Enforcement
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reference: Quad-Cities Nuclear Power Station
Docket Number 50-254, DPR-29, Unit One
Appendix A, Section 3.5.B.2

Enclosed please find Reportable Occurrence Report Number RO 83-17/03L-0
for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of
Technical Specification 6.6.B.2.b; condition leading to operation in a
degraded mode permitted by a limiting condition for operation.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Superintendent

NJK:DGC/bb

Enclosure

cc B. Rybak
N. Chrissotimos
INPO Records Center

MAY 13 1983

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