

LICENSEE EVENT REPORT

CONTROL BLOCK: [ ] [ ] [ ] [ ] [ ] [ ] [ ] [ ] (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

3 1 9  
 N Y F C S 1 2 0 0 0 0 0 0 0 0 0 0 0 0 0 0 3 4 1 1 1 1 1 1 4 5  
 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 37 CAT 38  
 3  
 0 1  
 REPORT SOURCE 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40  
 0 1 5 0 0 0 0 2 8 5 7 1 2 1 4 7 6 8 1 2 1 9 7 8 9  
 40 DOCKET NUMBER 48 49 EVENT DATE 54 55 REPORT DATE 60

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

During the 1978 Refueling Outage certain surveillance tests required by Section 3.0 and Appendix B of the Fort Calhoun Station Technical Specifications were not performed as scheduled because of the refueling shutdown condition (Operating Mode 5) of the plant. Facility License Change No. 77-11 was submitted to the Nuclear Regulatory Commission on September 9, 1977, and again on November 9, 1978. The approval of this Facility License Change would delete the requirements to perform these surveillance tests when the plant is in the refueling shutdown condition.

SYSTEM CODE X X (11)		CAUSE CODE X (12)		CAUSE SUBCODE Z (13)		COMPONENT CODE Z Z Z Z Z Z (14)				COMP. SUBCODE Z (15)		VALVE SUBCODE Z (16)	
LER RO REPORT NUMBER (17) [7 8]		EVENT YEAR [7 8]		SEQUENTIAL REPORT NO. [0 4 4]		OCCURRENCE CODE [0 3]				REPORT TYPE [1]		REVISION NO. [0]	
ACTION TAKEN X (18) X (19)		EFFECT ON PLANT Z (20)		SHUTDOWN METHOD Z (21)		HOURS [0 0 0 0] (22)				ATTACHMENT SUBMITTED Y (23)		NPRD-4 FORM SUB. N (24)	
FUTURE ACTION		PRIME COMP. SUPPLIER Z (25)		COMPONENT MANUFACTURER [Z 0 0 0 0] (26)									

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

10 The subject surveillance tests either could not be performed, or those which could be performed would not yield useful or meaningful data when the plant is in the refueling shutdown condition. Facility License Change No. 77-11 was submitted to the Nuclear Regulatory Commission on September 9, 1977, and again on November 9, 1978.

7 8 9  
FACILITY STATUS (28) % POWER (29) OTHER STATUS (30) METHOD OF DISCOVERY (31) DISCOVERY DESCRIPTION (32)  
1 5 H 0 0 0 NA A Technical Specification Review  
7 8 9 10 11 12 13 14 15 16  
ACTIVITY CONTENT (33) RELEASED OF RELEASE (34) AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)  
1 5 Z Z NA NA  
7 8 9 10 11 12 13 14 15 16  
PERSONNEL EXPOSURES (37) NUMBER (38) TYPE (39) DESCRIPTION (40)  
1 7 0 0 0 Z NA  
7 8 9 10 11 12 13 14 15 16  
PERSONNEL INJURIES (41) NUMBER (42) DESCRIPTION (43)  
1 7 0 0 0 NA  
7 8 9 10 11 12 13 14 15 16  
LOSS OF OR DAMAGE TO FACILITY (44) TYPE (45) DESCRIPTION (46)  
1 9 1 NA  
7 8 9 10 11 12 13 14 15 16  
PUBLICITY (47) SOURCE (48) DESCRIPTION (49)  
1 10 1 NA  
7 8 9 10 11 12 13 14 15 16  
7812270333 NRC USB ONLY

7812270333

NFC USE ONLY

NAME OF DECEASED

L. T. Kisek

PHONE

402-426-4022

LER 78-044  
Omaha Public Power District  
Fort Calhoun Station Unit No. 1  
Docket No. 05000285

Attachment No. 1

Safety Analysis

A review of the subject surveillance tests (see Attachment No. 2) indicates that failure to perform these tests during the refueling shutdown condition would not result in degraded system performance or any unsafe condition. Some of these surveillance tests cannot be performed because test initial conditions cannot be satisfied due to the refueling shutdown status of the plant. If these initial conditions were relaxed or deleted, damage to equipment may result. Other surveillance tests if performed during the refueling shutdown condition would yield results/test data which would not be useful or meaningful.

Selected surveillance tests were performed in order to determine the operability of safety systems prior to the plant's return to operation after the 1978 Refueling Outage. In addition, it is and has been common practice to test equipment and systems after maintenance or modifications have been completed.

*M. Andrews*

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Attachment No. 2

Cause Description and Corrective Action

The following surveillance tests were not performed during the 1978 Refueling Outage because of the refueling shutdown status of the plant.

<u>Surveillance Test Number</u>	<u>Title</u>	<u>Surveillance Test Section</u>
ST-CEA-1	Regulating CEA Groups/Transient Insertion Limit Check	F.2
ST-CEA-1	Regulating CEA Groups/Transient Insertion Limit Check	F.3
ST-CEA-1	Control Element Assembly Check	F.4
ST-CEA-1	PDIL, DEV and SEQ Monitoring System Test	F.5
ST-CEA-1	Secondary CEA Position Indicating System PDIL, DEV, OOS and Overlap Monitoring System Test	F.6
ST-CLT-1	RCS Cold Leg Temperature Check	F.1
ST-CHEM-1	S/G Water Iodine-131 Concentration	F.1
ST-CHEM-1	Secondary Plant Blowdown and Leakage Releases Isotopic Activity Analysis	F.1
ST-CHEM-1	Secondary Plant Blowdown and Leakage Proportional Composite for Tritium Analysis and Gross Alpha Analysis	F.2
ST-CHEM-1	RM-057 Condenser Air Ejector Releases Gross Gamma and Individual Gamma Emitters Activity Analysis	F.2
ST-DG-1	Diesel Fuel Inventory	F.1
ST-ENV-1	River Temperature Monitoring-Data Collection	F.1
ST-ENV-1	River Temperature Monitoring-Data Collection	F.2
ST-ENV-2	Surface Thermal Plume Measurements and Isotherm Plots	F.1

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Attachment No. 2 (Continued)

<u>Surveillance Test Number</u>	<u>Title</u>	<u>Surveillance Test Section</u>
ST-ENV-2	Surface Thermal Plume Isotherm Reports	F.2
ST-ENV-2	Triple-depth Thermal Plume Measurements and Isotherm Plots	F.3
ST-ENV-2	Triple-depth Thermal Plume Isotherm Reports	F.4
ST-ENV-2	Thermal Plume Measurements (Infrared Scanning)	F.5
ST-ENV-2	Thermal Plume Isothermal Plots (Infrared Scanning)	F.6
ST-ENV-4	Traveling Screen Impingement (Not Required When Circulators Are Down)	F.1
ST-ENV-4	Processing of Impingement Data (Data Not Required When Circulators Are Down)	F.2
ST-ESF-1	Pressurizer Pressure Channel Check	F.1
ST-ESF-1	Pressurizer Pressure Channel Check	F.2
ST-ESF-2	Channel "A" Safety Injection Actuation Signal Test	F.1
ST-ESF-2	Channel "B" Safety Injection Actuation Signal Test	F.2
ST-ESF-3	Containment Pressure Channel Check	F.1
ST-ESF-4	Channel "A" Containment Spray Actuation Signal Test	F.1
ST-ESF-4	Channel "B" Containment Spray Actuation Signal Test	F.2
ST-ESF-5	Automatic Load Sequencer Check	F.1
ST-ESF-6	Diesel Generator Check	F.2
ST-ESF-7	SIRW Tank Level Channel Check	F.1
ST-ESF-7	SIRW Tank Level Channel Check	F.2

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<u>Surveillance Test Number</u>	<u>Title</u>	<u>Surveillance Test Section</u>
ST-ESF-8	Safety Injection Tank Level and Pressure Instruments Check	F.1
ST-ESF-9	Boric Acid Tank Level Check	F.1
ST-ESF-11	Steam Generator Pressure Channel Check	F.1
ST-ESF-12	SIRW Tank Temperature Check	F.1
ST-ESF-12	SIRW Tank Temperature Check	F.2
ST-ESF-13	Channel "A" Recirculation Actuation Signal Test	F.1
ST-ESF-13	Channel "B" Recirculation Actuation Signal Test	F.2
ST-FW-1	Pump and Remotely Operated Valve Check	F.1
ST-ICI-1	Calculation of Total Integrated Radial Peaking Factor and Total Planar Radial Peaking Factor	F.1
ST-ICI-2	Incore Detector Alarm Limits Verification	F.1
ST-ISI-CVCS-3	CVCS Pump Test	F.1
ST-PL-1	Pressurizer Level Channel Check	F.1
ST-PL-1	Pressurizer Level Channel Check	F.2
ST-RA-1	Reactivity Balance	F.1
ST-RA-1	Normalization of Computed Boron Concentration vs. Burnup Curve	F.2
ST-RCF-1	Reactor Coolant System Flow Rate Determination	F.1
ST-RLT-3	Reactor Coolant System Leak Rate Calculation	F.1
ST-RPS-1	Power Range Safety Channels Check	F.1
ST-RPS-1	Power Range Safety Channel Adjustment	F.2

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Attachment No. 2 (Continued)

<u>Surveillance Test Number</u>	<u>Title</u>	<u>Surveillance Test Section</u>
ST-RPS-1	Power Range Safety Channels Test	F.3
ST-RPS-2	Wide Range Logarithmic Channel Functional Check	F.2
ST-RPS-3	Reactor Coolant Flow Check	F.1
ST-RPS-3	Reactor Coolant Low Flow Trip Check	F.2
ST-RPS-4	Thermal Margin/Low Pressure Channels Check	F.1
ST-RPS-4	Thermal Margin/Low Pressure Channels Check	F.2
ST-RPS-5	High Pressurizer Pressure Channels Check	F.1
ST-RPS-5	High Pressurizer Pressure Channels Check	F.2
ST-RPS-6	Steam Generator Level Channels Test	F.2
ST-RPS-7	Steam Generator Pressure Check	F.1
ST-RPS-7	Steam Generator Pressure Channels Test	F.2
ST-RPS-8	High Containment Pressure Channels Check	F.1
ST-RPS-9	Turbine Loss of Load Channel Check	F.1
ST-RPS-10	Manual Trip Check	F.1
ST-RPS-11	RPS Logic Units Test	F.1
ST-RPS-12	Axial Power Distribution Channels Check	F.1
ST-RPS-12	Axial Power Distribution Channels Test	F.2
ST-SCEAPIS-1	Calibration Secondary CEA Positioning	F.1
ST-SCEAPIS-1	Calibration of Secondary CEA Interlocks and Alarms	F.2
ST-SI/CS-1	Engineered Safeguards Mechanical Checks	F.1

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Attachment No. 2 (Continued)

<u>Surveillance Test Number</u>	<u>Title</u>	<u>Surveillance Test Section</u>
ST-SI/CS-1	SI Tank Check Valve Check	F.2
ST-TR-1	Turbine Runback Circuit Check	F.1

It was determined that the above surveillance tests either could not be performed or those which could be performed would not yield useful or meaningful data when the plant was in the refueling shutdown condition.

Facility License Change No. 77-11 was submitted to the Nuclear Regulatory Commission on September 9, 1977, and again on November 9, 1978, in order to fulfill the requirements specified by the Omaha Public Power District as corrective action to a previously identified item of noncompliance. The approval of this Facility License Change would delete the requirements to perform the subject surveillance tests when the plant is in the refueling shutdown condition (Operating Mode 5).

*Aluminum*