

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285

UNIT Fort Calhoun #1

DATE November 8, 1978

COMPLETED BY B. J. Hickie

TELEPHONE 402/536-4413

MONTH October, 1978

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>437.4</u>
2	<u>437.1</u>
3	<u>436.7</u>
4	<u>427.1</u>
5	<u>425.3</u>
6	<u>424.0</u>
7	<u>422.5</u>
8	<u>420.8</u>
9	<u>418.8</u>
10	<u>411.2</u>
11	<u>407.9</u>
12	<u>404.0</u>
13	<u>383.2</u>
14	<u>7.3</u>
15	<u>0.0</u>
16	<u>0.0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0.0</u>
18	<u>0.0</u>
19	<u>0.0</u>
20	<u>0.0</u>
21	<u>0.0</u>
22	<u>0.0</u>
23	<u>0.0</u>
24	<u>0.0</u>
25	<u>0.0</u>
26	<u>0.0</u>
27	<u>0.0</u>
28	<u>0.0</u>
29	<u>0.0</u>
30	<u>0.0</u>
31	<u>0.0</u>

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

78111.48161  
R

(9/77)

# OPERATING DATA REPORT

DOCKET NO. 50-285  
 DATE November 8, 1978  
 COMPLETED BY B. J. Hickie  
 TELEPHONE 402/536-4413

## OPERATING STATUS

1. Unit Name: Fort Calhoun
2. Reporting Period: October, 1978
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:  
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	745.0	7,296.0	44,713.0
12. Number Of Hours Reactor Was Critical	317.6	6,432.0	35,695.3
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,136.0
14. Hours Generator On-Line	315.2	6,403.7	34,887.6
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	420,284.0	8,872,237.9	41,534,760.2
17. Gross Electrical Energy Generated (MWH)	138,440.0	2,959,733.7	13,765,071.7
18. Net Electrical Energy Generated (MWH)	131,121.1	2,813,750.6	12,985,719.6
19. Unit Service Factor	42.3	87.8	78.0
20. Unit Availability Factor	42.3	87.8	78.0
21. Unit Capacity Factor (Using MDC Net)	38.5	84.5	64.2
22. Unit Capacity Factor (Using DER Net)	38.5	84.4	63.6
23. Unit Forced Outage Rate	0.0	3.9	5.0
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>None</u>			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: December 8, 1978

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1978DOCKET NO. 50-285UNIT NAME Fort Calhoun #1DATE November 8, 1978COMPLETED BY B. J. HinkleTELEPHONE 402/536-4413

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
78-04	781014	S	429.8	C	1	N/A	N/A	N/A	N/A

<sup>1</sup>  
F: Forced  
S: Scheduled

<sup>2</sup>  
Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

<sup>3</sup>  
Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Other (Explain)

<sup>4</sup>  
Exhibit G - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File (NUREG-  
0161)

<sup>5</sup>  
Exhibit I - Same Source

(9/77)

Refueling Information  
Fort Calhoun - Unit No. 1

Report for the month ending October 31, 1978.

1. Scheduled date for next refueling shutdown.
2. Scheduled date for restart following refueling.
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

In Progress

December 1, 1978

Yes

- a. If answer is yes, what, in general, will these be?

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.

- c. If no such review has taken place, when is it scheduled?

4. Scheduled date(s) for submitting proposed licensing action and support information.

Submitted Aug. 4, 1978

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

- |                                   |                            |            |            |
|-----------------------------------|----------------------------|------------|------------|
| 6. The number of fuel assemblies: | a) in the core             | <u>133</u> | assemblies |
|                                   | b) in the spent fuel pool  | <u>113</u> | "          |
|                                   | c) spent fuel pool         |            |            |
|                                   | storage capacity           | <u>483</u> | "          |
|                                   | d) planned spent fuel pool |            |            |
|                                   | storage capacity           | <u>483</u> | "          |

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

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Prepared by R L Jaworski

Date November 3, 1978

OMAHA PUBLIC POWER DISTRICT  
Fort Calhoun Station Unit No. 1

October 1978  
Monthly Operations Report

I. OPERATIONS SUMMARY

Unit operation continued at a nominal 100% power until the second week of October when a gradual power reduction was initiated in preparation for core physics testing.

The physics testing was completed and the unit shutdown on October 14 in preparation for a refueling outage.

Extensive maintenance operations continued throughout the remainder of the month.

The radioactive waste solidification modification preoperational test was initiated and will be completed when plant conditions allow.

An additional reactor operators license was issued to a member of the Fort Calhoun operations staff after successfully completing his examination by the Nuclear Regulatory Commission on October 11, 12.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
78-031	During normal steady state power operation the hot leg temperature indicator for "D" channel went to full scale. The remaining three hot leg temperature channels remained operable throughout this event.
78-032	During the performance of Surveillance Test ST-ESF-5, Section F.1, the monthly AC sequencer timer test, timers SI-1B and CH-1B on sequencer S2-2 failed to time out within the prescribed limit. The timers were exercised and tested within specified limits.

B. CHANGES IN OPERATING METHODS

<u>Procedure Change No.</u>	<u>Title</u>	<u>Reason for Change</u>
3684	OI-FP-6-Fire Protection System Inspection & Test	To update procedure to conform with recent additions of fire protection equipment per Fire Protection SER

C. RESULTS OF SERVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which resulted in Operations Incidents and are not reported elsewhere in the report:

Operations  
Incident #

Deficiency

OI 657

During the performance of ST-FP-2 F.2 Steps 1 and 2 of the procedure were performed out of sequence without prior PRC approval (procedure change)

The following are surveillance tests where an unusual result occurred:

Surveillance  
Test No.

Description

ST-ESP-5

Run to determine if timers were accurate

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure

Description

SP-FAUD-1

Fuel Assembly Uplift Condition Detection. Two loops showed greater than 99% assurance that more than 150 lbs. downward force exists on the fuel assembly. Core  $\Delta p$  transmitters on the other two loops gave erroneous readings and MO 20280 was prepared to repair instrument loops.

SP-VA-6

Charging Pump Room Temp. Rise Test/Test results satisfactory.

E. RESULTS OF LEAK RATE TESTS

The Refueling Leak Rate Testing program was started this month and will continue throughout the outage in accordance with ST-CONT-2 and 3. A follow up report will be submitted within 90 days of the completion of the Leak Rate Testing Program conducted during the 1978 Refueling Outage.

F. CHANGES IN PLANT OPERATING STAFF

Robert Luikens employed as Auxiliary Operator-Nuclear.

G. TRAINING

October training consisted of Radiation Protection refresher for most of the unrestricted and restricted personnel in preparation for the 1978 refueling outage. There was a large number of contract personnel that were give initial radiation protection instructions. Three hot license persoonel were tested by the NRC and the R.O. candidate received a Reactor Operator license. The two SRO upgrades are still pending.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59.

Modifications designated in NRC safety evaluation report for fire protection program at Fort Calhoun Station designated to be completed in October 1978 were completed this month.

Approved by Lawrence T. Kusek  
for Manager-Fort Calhoun Station



# Monthly Operations Report

October 1978

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## II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
18209	10/19/78	Inspect & adjust as necessary fan blades on VA-7C	Completed per MP-VA-7-1
18210	10/20/78	Inspect and adjust as necessary for blades on VA-7D	Completed per MP-VA-7-1
20427	10/27/78	Cotter pin missing on Snubber SIS 122A	Replaced missing cotter pin
20414	10/23/78	Grease bearing in small hook of polar crane	Greased small hook bearing
17543	10/16/78	Remove missile shields	Completed per procedure
18004	10/19/78	Remove equipment hatch	Completed per procedure
20424	10/25/78	RMO 50/51 change set points	Returned to Containment setpoints
20425	10/16/78	Repair seals on PAL door	Fluffed inner door seal
18002	10/16/78	Annual pre-operations check on Polar crane	Completed per procedure
20288	10/16/78	Magnaflux hooks on polar crane	Completed
20367	10/18/78	PAL door repair inner locks	Freed interlocks
19749	8/29/78	Relocate fire hose cabinet	Completed
20522	11/1/78	C/122 H-1 Zero pot malfunction	Installed temp. transmitter & cal.
20383	11/1/78	HCV 1388A,B leaking	Reworked valves and retested sat.
20053	11/1/78	D/112H reading full scale	Installed new RTD
20437	10/30/78	HCV-1108A repair and repack	Repaired and repacked.
20040	11/3/78	FIC-2890 and FIC-2891 Raw Water header flow indicators giving spurious alarm	Flow nozzles cleaned and channels recalibrated
20372	10/21/78	C/PIA-102X out of spec.	Trimmed meter span & checked.
20390	10/25/78	LI-105 Sigmas stuck at 24%	Calibrated loop & refilled ref. leg
20323	10/21/78	HCV-326 valve will not operate via controller	Cleaned orifice
20428	10/23/78	TC 123 Wide Range Temperature indicator indicates 100.0 degrees high	Calibrated
20273	10/18/78	Perform SP-CPTP-12	Completed
19946	10/22/78	Security zone #26 door malfunctioning	Door adjusted and inspected
20344	10/19/78	Weather Tower 110 meter AT check and calibrate	Checked and found no problems.
20404	10/21/78	RMO-61 filter advance solenoid broke	Replaced solenoid advance mechanism
20440	10/25/78	RMO-60 gas meter inoperable	Replace flow meter & cleaned filters
18512	9/29/78	Fire Protection-jockey pump packing leak	Installed ring of packing.
20162	9/29/78	PAL Door-failed leak test	Fluffed seal & retested sat.
18578	10/2/78	Fire Protection-diesel fire pump hose leading from oil filter to bottom casing leaking.	Replaced gaskets
20379	10/24/78	FW-163 - Inspection only.	Correct orientation.
20378	10/24/78	FW-164 - Inspection only.	Wrong orientation - MO 20429 written to change orientation.