

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285

UNIT Fort Calhoun #1

DATE October 9, 1978

COMPLETED BY B. J. Hickie

TELEPHONE 402-536-4413

MONTH September, 1978

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>434.8</u>
2	<u>435.4</u>
3	<u>435.0</u>
4	<u>432.9</u>
5	<u>432.6</u>
6	<u>431.5</u>
7	<u>430.5</u>
8	<u>430.4</u>
9	<u>431.1</u>
10	<u>431.1</u>
11	<u>430.5</u>
12	<u>430.5</u>
13	<u>433.0</u>
14	<u>432.3</u>
15	<u>433.0</u>
16	<u>434.4</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>434.9</u>
18	<u>434.8</u>
19	<u>436.2</u>
20	<u>437.7</u>
21	<u>438.5</u>
22	<u>439.4</u>
23	<u>439.5</u>
24	<u>439.7</u>
25	<u>438.6</u>
26	<u>438.0</u>
27	<u>438.2</u>
28	<u>438.4</u>
29	<u>438.0</u>
30	<u>439.0</u>
31	<u> </u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

LPDR

(9/77)

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OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE October 9, 1978
 COMPLETED BY B. J. Hickie
 TELEPHONE 402-536-4413

OPERATING STATUS

1. Unit Name: Fort Calhoun Station Unit No. 1
2. Reporting Period: September, 1978
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
 N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	6,551.0	43,968.0
12. Number Of Hours Reactor Was Critical	720.0	6,114.4	35,377.7
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,136.0
14. Hours Generator On-Line	720.0	6,088.5	34,572.4
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,011,328.4	8,451,953.9	41,114,476.2
17. Gross Electrical Energy Generated (MWH)	329,508.0	2,821,293.7	13,626,631.7
18. Net Electrical Energy Generated (MWH)	313,192.8	2,682,629.5	12,854,598.5
19. Unit Service Factor	100.0	92.9	78.6
20. Unit Availability Factor	100.0	92.9	78.6
21. Unit Capacity Factor (Using MDC Net)	95.2	89.7	64.6
22. Unit Capacity Factor (Using DER Net)	95.2	89.6	64.0
23. Unit Forced Outage Rate	0.0	4.1	5.1
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Refueling Outage, October 15, 8 weeks			

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

UNIT SHUTDOWNS AND POWER REDUCTIONS

50-285
 DOCKET NO. Fort Calhoun #1
 UNIT NAME October 9, 1978
 DATE B. J. Hickie
 COMPLETED BY 402-536-4413
 TELEPHONE

REPORT MONTH September, 1978

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
	NONE								

- 1 F: Forced
S: Scheduled
- 2 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)
- 3 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)
- 4 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)
- 5 Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending Sept. 30, 1978.

1. Scheduled date for next refueling shutdown. October 15, 1978
2. Scheduled date for restart following refueling. December 1, 1978
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
Yes
 - a. If answer is yes, what, in general, will these be?
Technical Specifications changes with reload application submitted to NRC on August 4, 1978.
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.

 - c. If no such review has taken place, when is it scheduled?

4. Scheduled date(s) for submitting proposed licensing action and support information. Submitted Aug. 4, 1978
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
6. The number of fuel assemblies:

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>113</u>	"
c) spent fuel pool storage capacity	<u>483</u>	"
d) planned spent fuel pool storage capacity	<u>483</u>	"
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1985

Prepared by R L Jaworski Date October 2, 1978

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

September 1978
Monthly Operations Report

I. OPERATIONS SUMMARY

During the month of September, installation of the security system modifications continued. Waste solidification system modifications are essentially complete. Preoperational checkout of individual waste system components has been initiated. Unit operations continued at near rated capacity throughout the month. Maintenance on A condensate pump is continuing. Refueling outage preparations continued. FCS Unit is scheduled to be shutdown the evening of Friday, October 13, 1978, for the fall refueling outage.

Extensive preventative maintenance on the waste evaporator was completed. This should provide for optimum performance of the evaporator during the outage.

Operations personnel identified a reactor coolant system leak on sampling root valve (RC-138) packing. RC-138 was shut to terminate the leak.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
78-026 Rev. 1	During normal steady state power operation, "B" channel RPS variable overpower trip reset demand alarm actuated and would not reset. Shortly after this alarm came in, the High power and thermal margin/low pressure trip units for "B" channel RPS failed to the tripped condition. The three remaining RPS channels remained operable throughout this event.
78-027	During the performance of Surveillance Test ST-ESF-5, Section F.1, the monthly AC sequencer timer test, timer SI-1B on sequencer S2-2 failed to time out within the prescribed limit. The timer was exercised and tested within specified limits.
78-028	During the compilation and review of weather data for the first 1978 semi-annual report, historically consistent wind stability classes could not be determined due to a failure of the Tri-X weather tower's 100 meter Delta-T sensor system between March 14, 1978 and July 21, 1978. Inability to report stability class information is not in compliance with USNRC Regulatory Guide 21.

A. PERFORMANCE CHARACTERISTICS (Continued)

<u>LER Number</u>	<u>Deficiency</u>
78-029	Inadvertent radioactive gas release to the atmosphere occurred due to leakage from the chemical volume control tank through check valves CH-284 and HG-100 and relief valve HG-105. Operations noted gradual decrease in tank pressure and initiated action to gag relief valve HG-105 and isolate HG-105 with valve HG-101. Releases were within Technical Specification Limits.
78-030	While performing an operability check after preventive maintenance on the auxiliary steam driven feed pump, the steam inlet valve YCV-1045 did not open. The instrument air supply to YCV-1045 was found to be closed and YCV-1045 was in its failed closed position. Manual operator action would have been required to start FW-10.

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which results in Operations Incidents and are not reported elsewhere in the report:

<u>Operations Incident #</u>	<u>Deficiency</u>
OI-650	While performing ST-FD-1, 13 fire detectors in Zone 13 were found out of calibration. Although out of calibration, the detectors were operable and were recalibrated to within specification.

The following is a surveillance test where an unusual result occurred:

<u>Surveillance Test No.</u>	<u>Description</u>
ST-CONT-2	P.A.L. door test failed.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
DCR 77-84	Seismic supports installed per design package on Raw Water, Component Cooling, Chemical and Volume Control and Blowdown systems to alleviate possible over stress conditions on pipe supports in those systems.
SP-FAUD-1	Fuel Assembly Uplift Condition Detector. All loops show greater than 99% assurance that greater than 150 lbs downward force exists in the fuel assemblies.
SP-RPS-5	Excore Detector Symmetric Offset Recalibration. The upper and lower excore subchannels were adjusted as necessary to conform with actual power tilts in the core as determined by the incore detectors.
ST-WDS-8	Waste Evaporator Chemical Flush.
ST-CONT-2	Personnel Air Lock (PAL) O-Ring Seal Test. Daily results are acceptable.

E. RESULTS OF LEAK RATE TESTS

All RCS leak rate results in spec.
Completed leak rate tests on penetrations M-87 and M-88. Results satisfactory.
ST-CONT-2 - PAL door test failed. Retest after maintenance was satisfactory.

F. CHANGES IN PLANT OPERATING STAFF

Mr. Kenneth Stier was employed as an Auxiliary Operator-Nuclear.

G. TRAINING

Hot license training was increased for three candidates that are preparing for NRC exams. Two candidates are upgrading their licenses from RO to SRO and one candidate is seeking a RO license. Normal plant training continued for Maintenance Group such as Crane Operator requalification and Security and Radiation Protection Refresher.

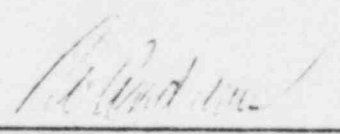
H. CHANGES, TESTS AND EXPERIEMNTS REQUIRING NUCLEAR REGULATORY
COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59.

Changes to Operating License DPR-40

Amendment No. 40 dated August 23, 1978.

- a. Section 3.15 revised to specify fire pump strainer surveillance and revised fire detector inspection frequency for detectors which would require off-normal equipment alignment.
- b. Modifications designated in NRC safety evaluation report for fire protection program at Fort Calhoun Station made condition of operating license. All September 1978 fire protection implementation committment items were completed this month.

Approved by


Manager-Fort Calhoun Station

Monthly Operations Report
 September 1978
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II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
19880	8-28-78	Check ref. legs for SI-6A, B, C, D level transmitters	Reference legs filled and returned to service.
19943	9-5-78	CH-1B change oil.	Completed.
19923	9-1-78	FP-156, FP-158 check for leakage	Changed rubber seats.
19920	9-1-78	FP-311 flushing valve bonnet broken	Replaced.
19939	9-3-78	CH-1A pump showed loss of flow with erratic amps.	Repaired pump
19914	8-30-78	RPS channel "B" VHPT operating erratically.	Replaced \pm 18 volt power supply
20007	9-13-78	FP-156 will not close.	Replaced pin and returned to normal.
19851	9-12-78	Test spare R.C. pump seal lower pressure device.	Pressure breakdown device tested satisfactorily in accordance with PRC approved procedure.
20032	9-16-78	CH-1A leaks	Repacked pump and replaced all cap gaskets.
20070	9-21-78	Preventive Maintenance on Auxiliary Feed Pump (FW-10)	Changed oil in Auxiliary Feed Pump (FW-10)
200	9-15-78	CH-1B packing cooling pump	Impeller set screw replaced.
20140	9-28-78	PAL Door failed test.	Fluffed seal.
20024 20059 20078	9-28-78	Fire Protection S.E.R. September 1978 implementation items	Completed.