

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346
UNIT Davis-Besse Unit 1
DATE February 8, 1983
COMPLETED BY Bilal Sarsour
TELEPHONE 419-259-5000,
Ext. 384

MONTH January, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>878</u>	17	<u>864</u>
2	<u>878</u>	18	<u>708</u>
3	<u>878</u>	19	<u>0</u>
4	<u>874</u>	20	<u>0</u>
5	<u>877</u>	21	<u>0</u>
6	<u>879</u>	22	<u>0</u>
7	<u>873</u>	23	<u>0</u>
8	<u>874</u>	24	<u>0</u>
9	<u>880</u>	25	<u>0</u>
10	<u>876</u>	26	<u>0</u>
11	<u>876</u>	27	<u>0</u>
12	<u>881</u>	28	<u>0</u>
13	<u>884</u>	29	<u>0</u>
14	<u>884</u>	30	<u>0</u>
15	<u>593</u>	31	<u>0</u>
16	<u>412</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

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OPERATING STATUS

1. Unit Name: Davis-Besse Unit 1
2. Reporting Period: January, 1983
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 918
7. Maximum Dependable Capacity (Net MWe): 874

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	744	39,505
12. Number Of Hours Reactor Was Critical	420.5	420.5	21,316
13. Reactor Reserve Shutdown Hours	310.7	310.7	3,674.8
14. Hours Generator On-Line	414.6	414.6	20,174.2
15. Unit Reserve Shutdown Hours	0	0	1,732.5
16. Gross Thermal Energy Generated (MWH)	1,117,005	1,117,005	46,489,766
17. Gross Electrical Energy Generated (MWH)	375,675	375,675	15,481,329
18. Net Electrical Energy Generated (MWH)	351,833	351,833	14,467,273
19. Unit Service Factor	55.7	55.7	51.1
20. Unit Availability Factor	55.7	55.7	55.5
21. Unit Capacity Factor (Using MDC Net)	54.1	54.1	41.9
22. Unit Capacity Factor (Using DER Net)	52.2	52.2	40.4
23. Unit Forced Outage Rate	44.3	44.3	21.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: February 1, 1983

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January, 1983

DOCKET NO. 50-346
 UNIT NAME Davis-Besse Unit 1
 DATE February, 1983
 COMPLETED BY Bilal Sarsour
 TELEPHONE 419-259-5000, Ext. 384

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	83 01 15	F	12.8	H	3	NA			The reactor tripped on Reactor Protection System $\phi/\Delta\phi$ /flow due to imbalance.
2	83 01 18	F	316.6	A	3	NA			<p>The reactor tripped on high Reactor Coolant System pressure due to a loss of the non-nuclear instrumentation X-AC power supply.</p> <p>A unit shutdown was initiated to investigate the Main Steam Line No. 2 Isolation Valve problem.</p> <p>See Operational Summary for further details.</p>

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuation from Previous Month
 5-Load Reduction
 9-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

OPERATIONAL SUMMARY
January, 1983

1/1/83 - 1/15/83: Reactor power was maintained at approximately 99 percent power until 0945 hours on January 15, 1983, when power was reduced to 85 percent to perform PT 5193.01, "Main Turbine Steam Valves".

Reactor power was slowly increased and attained 99 percent power at 1600 hours on January 15, 1983.

Reactor power was maintained at 99 percent until 1700 hours on January 15, 1983, when the reactor tripped on Reactor Protection System flux-delta flux-flow. The imbalance occurred after an amount of unborated water in excess of required for the power increase and core conditions was added.

The reactor was critical at 2250 hours.

1/16/83: The turbine generator was synchronized on line at 0550 hours.

1/17/83 - 1/31/83: Reactor power was slowly increased and attained 99 percent power on January 17, 1983.

Reactor power was maintained at 99 percent power until 1925 hours on January 18, 1983, when the reactor tripped on high Reactor Coolant System pressure due to a loss of the non-nuclear instrumentation X-AC power supply. This was caused by a short to ground within the operate range level recorder for OTSG #2.

During the recovery from the trip, it was discovered the Main Steam Line No. 2 Isolation Valve was allowing some steam flow even though the valve appeared to be fully closed. A unit cooldown was initiated.

REFUELING INFORMATION

DATE: January, 1983

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling shutdown: September 3, 1983
3. Scheduled date for restart following refueling: October 29, 1983
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what in general will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Ans: Expect the Reload Report to require standard reload fuel design Technical Specification changes (3/4.1 Reactivity Control Systems and 3/4.2 Power Distribution Limits).

5. Scheduled date(s) for submitting proposed licensing action and supporting information: July, 1983
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Ans: None identified to date.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 177 (b) 92 - Spent Fuel Assemblies

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present: 735 Increase size by: 0 (zero)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date: 1992 - assuming ability to unload the entire core into the spent fuel pool is maintained.

COMPLETED FACILITY CHANGE REQUEST

FCR NO: 77-228

SYSTEM: Emergency Ventilation System and Control Room Emergency Ventilation System

COMPONENT: Fans C-30-1 and 2, C-21-1 and 2

CHANGE, TEST OR EXPERIMENT: FCR 77-228 provided timing devices which were added to the breakers for Emergency Ventilation Fans C-30-1 and 2 and Control Room Emergency Ventilation Fans C-21-1 and 2. These devices measure the total time the breakers are closed, keeping an accurate record of run times for these systems. Work was completed March 27, 1981.

REASON FOR CHANGE: An accurate method is required to measure and record run times to insure that the 720 hour time limits on the filter units of these systems are not exceeded.

SAFETY EVALUATION: Electric timers were added to the systems for the purpose of providing an accurate method of recording run times on these systems. No functional change was made to the basic systems.