



**Commonwealth Edison**

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50-254

NJK-83-78

March 2, 1983

Mr. Edson G. Case, Deputy Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Mr. Case:

Enclosed please find a listing of those changes, tests, and experiments completed during the month of February, 1983, for Quad-Cities Station Units 1 and 2, DPR-29 and DPR-30. A summary of the safety evaluation is being reported in compliance with 10 CFR 50.59.

Thirty-nine copies are provided for your use.

Very truly yours,

COMMONWEALTH EDISON COMPANY  
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis  
Station Superintendent

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Enclosure

cc: T. J. Rausch

IE2A

## SPECIAL TEST 1-61

On March 1, 1983, Special Test 1-61 was completed.

Visual inspection and special measurements (NDT - eddy current testing, ultrasonic testing, etc.) were performed on selected Unit One fuel. Gamma scanning was performed on selected bundles and individual fuel rods. Neutron radiography of selected control blades and individual fuel rods was performed. Replacement of one fuel rod from each of two Barrier Lead Test Assemblies and retrieval of a set of spacers from a high residence time 7 x 7 fuel bundle was completed.

### Summary of Safety Evaluation

The probability of an occurrence or the consequence of an accident, or malfunction of equipment important to safety as previously evaluated in the Final Safety Analysis Report is not increased, because all fuel handling will be performed within the confines of the spent fuel storage pool using standard, reviewed and approved fuel handling procedures. Generic procedures will also be used by General Electric personnel that have been reviewed under the Special Test Review and are found acceptable.

The possibility for an accident or malfunction of a different type than any previously evaluated in the Final Safety Analysis Report is not created, because all fuel maneuvers will be standard operations such as bundle transfers between racks, dechanneling and channeling and are within the assumptions of the FSAR evaluation. The other activities do not create accident potentials greater than the dropped fuel bundle accident considered in the FSAR.

The margin of safety, as defined in the basis for any Technical Specification, is not reduced because no activities during this test would cause release rates greater than those assumed for the Technical Specification basis.

M-4-1-7-47

## RCIC Automatic Initiation Pushbutton

### Description

The installation of the RCIC Automatic Initiation Pushbutton will provide a manual fast start capability of the RCIC System, should the need arise, decreasing the time involved from 10 minutes to 20 seconds. The installation consisted of wiring a pushbutton actuator on the 901-4 panel, in parallel with the existing logic for Reactor Vessel Low Low Water Level.

### Evaluation

This modification will provide a marked decrease in the system initiation time to allow faster manual actuation of the system. To prevent accidental initiation, a plexiglass enclosure was installed around the pushbutton and the switch was wired with a double set of contacts in series to prevent D.C. arcing. Since existing components of the RCIC System will not be altered, the margins of safety as defined in the Technical Specifications will not be affected.

M-4-1-81-26

#### Containment Spray Header Drain Lines

##### Description

This modification installed drain lines in low points in the header and header inlet line. It was initiated to allow drainage of accumulated water from these low spots prior to the Drywell Spray Header and Nozzle Air Test to prevent spraying test personnel with any contaminated water.

##### Evaluation

This new drain line can be considered similar to a spray nozzle on the spray header. The function and operation of the spray header will not be affected by this modification. Containment integrity and isolation will not be adversely affected since all new piping will be installed inside the Drywell.

M-4-1-77-46

#### HPCI Automatic Initiation Pushbutton

##### Description

The installation of the HPCI Automatic Initiation Pushbutton will provide a manual fast start capability of the HPCI System should the need arise, decreasing the time involved from 10 minutes to 20 seconds. The installation consisted of wiring a pushbutton actuator on the 901-3 panel, in parallel with the existing logic for Reactor Vessel Low Low Water Level.

##### Evaluation

This modification will provide a marked decrease in the system initiation time to allow faster manual actuation of the system. To prevent accidental initiation, a plexiglass enclosure was installed around the pushbutton and the switch was wired with a double set of contacts in series to prevent D.C. arcing. Since existing components of the HPCI System will not be altered, the margins of safety as defined in the Technical Specifications will not be affected.

M-4-1-79-23

### Pressure Suppression Wide Range Pressure Instrumentation

#### Description

This modification designed and installed instrumentation for the Suppression Chamber capable of measuring pressures ranging from 5 psi vacuum to four times Containment design pressure. The modification was designed to meet the NRC Commitment NUREG 0578.

#### Evaluation

The new instrumentation will provide a wider range monitoring capability than the present instrumentation, which will remain in place for normal operation. The modification will provide monitoring capability only and design requirements are unchanged. Therefore, the new instrumentation will exceed existing capabilities and existing equipment will remain in place to meet any Technical Specification requirements.

M-4-1-79-24

### Suppression Chamber Level Instrumentation (Wide Range)

#### Description

This modification designed and installed instrumentation for the Suppression Chamber capable of measuring water level ranging from the bottom of the Suppression Chamber to 5 feet above normal water level. The modification was designed to meet the NRC Commitment NUREG 0578.

#### Evaluation

The new instrumentation will provide a wider range monitoring capability than the present instrumentation, which will remain in place for normal operation. The modification will provide monitoring capability only and design requirements are unchanged. Therefore, the new instrumentation will exceed existing capability and existing equipment will remain in place to meet any Technical Specification requirements.

M-4-2-79-23

### Pressure Suppression Wide Range Pressure Instrumentation

#### Description

This modification designed and installed instrumentation for the Suppression Chamber capable of measuring pressures ranging from 5 psi vacuum to four times Containment design pressure. The modification was designed to meet the NRC Commitment NUREG 0578.

#### Evaluation

The new instrumentation will provide a wider range monitoring capability than the present instrumentation, which will remain in place for normal operation. The modification will provide monitoring capability only and design requirements are unchanged. Therefore, the new instrumentation will exceed existing capabilities and existing equipment will remain in place to meet any Technical Specification requirements.

M-4-2-79-24

### Suppression Chamber Level Instrumentation (Wide Range)

#### Description

This modification designed and installed instrumentation for the Suppression Chamber capable of measuring water level ranging from the bottom of the Suppression Chamber to 5 feet above normal water level. The modification was designed to meet the NRC Commitment NUREG 0578.

#### Evaluation

The new instrumentation will provide a wider range monitoring capability than the present instrumentation, which will remain in place for normal operation. The modification will provide monitoring capability only and design requirements are unchanged. Therefore, the new instrumentation will exceed existing capability and existing equipment will remain in place to meet any Technical Specification requirements.