LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JANUARY, 1983

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle Nuclear Power Station Unit One is a Boiling Water Reactor with a designed electrical output of 1078 MWe net, located in Marseilles, Illinois. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. The plant is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit has not commenced commercial generation of power.

This report was compiled by John Ullrich, telephone number (815)357-6761, extension 481.

II. SUMMARY OF UNIT OPERATING EXPERIENCE FOR UNIT ONE

January 1-31 The unit was shutdown due to the "B" RHR Pump being declared inoperative due to excessive pump vibrations.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications.

There were no amendment to facility license or technical specification during the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

C. Tests and Experiments Requiring NRC Approval.

There were no tests or experiments requiring NRC approval during the reporting period.

D. Corrective Maintenance of Safety Related Equipment.

The following tables present a summary of safety-related maintenance completed on Unit One during the reported period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Component Name, Cause of Malfunctions, Results and Effects on Safe Operation, and Corrective Action.

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L15388	--	VC 'B' Train Steam Generator	Bad heating element	Steam generator will not Start-up trips breaker.	Replaced heating element
L20676	--	'OB' VC Odor Eater Inlet Damper	Bad upper seal in actuation	Damper would not close.	Replaced upper seal.
L20716	--	Charcoal Filter Inlet Damper	Bad 'O' ring on solenoid extension	Damper indicates open at all times	Replaced 'O' ring
L20994	--	A VC Air Intake Rad Monitor D	Input coupling capacitor and input discriminator chip A101 defective	Rad monitor failed down-scale	Replaced input coupling capacitor and discriminator chip
L21028	82-174/03L-0	'B' IRM	Loose high voltage connector	Indication is erratic	Replaced high voltage connector
L21215	--	+20 VDC APRM Power Supply	Bad SCR in power supply	Power supply does not work	Replaced SCR
L21414	--	'A' IRM	Bad pre-regulator	IRM reading high on range one.	Replaced pre-regulator
L21436	82-173/03L-0	Outboard Reactor Water Sample Isolation Valve	Dirty valve and solenoid	Solenoid valve intermittently sticks in the open position	Cleaned valve and solenoid
L21438	--	'B' Rx Recirc Suction Valve	Bad valve	Valve won't cycle	Replaced valve
L21460	--	Pressure Transmitter for 0PL58JA Flow Sensing	Bad transducer	Readings from pressure transmitter and pressure indicator do not agree	Replaced transducer
L21461	--	Pressure Transmitter for 0PL58JB Flow Sensing	Bad transducer	Readings from pressure transmitter and pressure indicator do not agree	Replaced transducer
L21474	--	Make-up and Blowdown Sample Lines	Lines plugged	Not available for NPDES sampling and daily radioactivity measurements	Cleaned lines

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L21557	--	VC Rad Monitor B	Bad G-M tube	B PRM won't stay reset	Replaced G-M tube
L21690	--	RHR 'A' Inboard Drywell Spray Valve	Bad packing	Valve packing leaks	Replaced packing
L21726	--	Drywell Particulate Rad Monitor	Bad low voltage connector	Input signal cable has a loose connector	Replaced ~~low~~ voltage connector
L21768	--	'1B' D/G Governor	Bad cams, micro switch arm and sync motor	Speed could not be controlled either locally or remotely using the raise/lower switch	Replaced cams, micro switch and sync motor
L21861	--	D/G 1B Motor-Driven Air Compressor Discharge Relief Valve	Bad Valve	Valve lifts at low pressure	Replaced valve
L21865	--	'B' Air Compressor for 2A D/G	Bad 'M' contactor	Placing control switch to off does not stop the air compressor	Replaced 'M' contactor
L21892	--	A VC Filter Train Out	Upper piston seal bad	Does not close from control room	Replaced upper piston seal
L21945	--	Control Room HVAC Charcoal Filter Bypass Damper	Fuse pulled	Damper did not close	Reinstalled fuse
L21947	--	Aux Electric Room HVAC Exhaust Air Outlet Damper	Fuse pulled	Damper did not open	Reinstalled fuse
L21949	--	Control Room HVAC O.A. Damper	Fuse pulled	Damper did not open	Reinstalled fuse
L21950	--	Aux Electric Room Charcoal Filter Inlet Damper 0VE07YA	Actuator limits out-of-adjustment	Dual position indication	Set limits on actuator

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L21951	--	Auz Electric Room Charcoal Filter Inlet Damper 0VE09YA	Bad seals on piston shaft	Dual position indication	Replaced seals
L21952	--	Aux Electric Room HVAC Outside Air Isolation Damper	Fuse pulled	Damper did not open	Reinstalled fuse
L21962	--	Main Steam Line Drain Valve	Torque switch contacts dirty	Can't seal-in close position	Cleaned contacts
L22014	--	Safety Relief Valve	Solenoid valve was grounded	Positive ground on Div II power supply could cause valve to go open	Replaced solenoid valve

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, January 1, to January 31, 1983. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

Licensee Event Report Number	Date	Title of Occurrence
82-180/03L-0	12/30/82	Mechanical Snubbers not Installed
83-002/03L-0	1/21/83	Failure to Perform Functional LIS-HP-09
83-003/03L-0	1/18/83	Missed Calibration and Functional Test of LIS-MS-06
83-004/03L-0	1/21/83	Overdue Calibration of Unit 2 Hydrogen Recombiner Instrumentation

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

ATTACHMENT D

LTP-300-7
Revision 2
November 15, 1979
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OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LaSalle One
DATE 2/2/82
COMPLETED BY John Ullrich
TELEPHONE (815) 357-6761 x481

OPERATING STATUS

1. REPORTING PERIOD: January 1983 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 100% MAX. DEPEND. CAPACITY (MWe-Net): 0
   DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): _____
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	0	0	2747.4
6. REACTOR RESERVE SHUTDOWN HOURS	0	0	0
7. HOURS GENERATOR ON LINE	0	0	1857.6
8. UNIT RESERVE SHUTDOWN HOURS	0	0	0
9. GROSS THERMAL ENERGY GENERATED (MWH)	0	0	2140579
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0	0	520399
11. NET ELECTRICAL ENERGY GENERATED (MWH)	0	0	460775
12. REACTOR SERVICE FACTOR	NA	NA	NA
13. REACTOR AVAILABILITY FACTOR	NA	NA	NA
14. UNIT SERVICE FACTOR	NA	NA	NA
15. UNIT AVAILABILITY FACTOR	NA	NA	NA
16. UNIT CAPACITY FACTOR (Using MDC)	NA	NA	NA
17. UNIT CAPACITY FACTOR (Using Design MWe)	NA	NA	NA
18. UNIT FORCED OUTAGE RATE	NA	NA	NA

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 2/2/83

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	6/21/82
INITIAL ELECTRICITY	_____	3/4/82
COMMERCIAL OPERATION	4/1/83	_____

ATTACHMENT A

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-373

UNIT LaSalle One

DATE February 2, 1983

COMPLETED BY John Ullrich

TELEPHONE (815) 357-6761 x481

MONTH January 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	0	17.	0
2.	0	18.	0
3.	0	19.	0
4.	0	20.	0
5.	0	21.	0
6.	0	22.	0
7.	0	23.	0
8.	0	24.	0
9.	0	25.	0
10.	0	26.	0
11.	0	27.	0
12.	0	28.	0
13.	0	29.	0
14.	0	30.	0
15.	0	31.	0
16.	0		

LTP-300-7
Revision 2
November 13, 1979
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ATTACHMENT B
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE Feb. 2, 1983
COMPLETED BY John Ullrich
TELEPHONE (815) 357-6761
x-481

REPORT MONTH January 1983

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
1	12-31-82*	F	744.0	A	4	Normal shutdown due to "Unusual Event" due to "B" RHR Pump being inop. because of high vibration

* Refer to the December 1982 report.

VI. UNIQUE REPORTING REQUIREMENTS

A. Main Steam Relief Valve Operations for Unit 1

There were no main steam relief valve operations during this reporting period.

B. ECCS Systems Outages

There were no ECCS System Outages during this reporting period.

C. Off-Site Dose Calculation Manual

There were no changes to the Off-Site Dose Calculations Manual during this reporting period.

D. Radioactive Waste Treatment System

There were no changes to the Radioactive Waste Treatment System during this reporting period.

E. Process Control Program

There were no changes to the Process Control Program during this reporting period.