

50.55(e) Report

Arizona Public Service Company

P.O. BOX 21666 • PHOENIX, ARIZONA 85036

1983 MAR -7 PM 2:25

February 28, 1983
ANPP-23120-GHD/BSK

U. S. Nuclear Regulatory Commission
Region V
Creskide Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

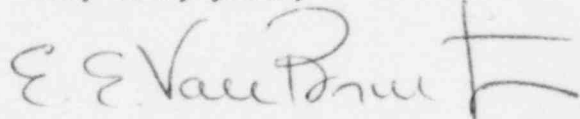
Subject: Final Report - DER 83-6
A 50.55(e) Report Relating to
Undersize Shock Suppressor Installed And Accepted By QC In
Unit 1 Containment
File: 83-019-026
D.4.33.2

Reference: (A) Telephone Conversation between P. Narbut and
J. Cook on February 3, 1983

Dear Sir:

Attached is our final written report of the Reportable Deficiency, under
10CFR50.55(e), referenced above.

Very truly yours,



E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects Management
ANPP Project Director

EEVBJr/GHD:db

Enclosure

cc: See Attached Page 2

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U. S. Nuclear Regulatory Commission
Attention: Mr. D. M. Sternberg, Chief
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cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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FINAL REPORT - DER 83-6
DEFICIENCY EVALUATION 50.55(e)
ARIZONA PUBLIC SERVICE COMPANY (APS)
PVNGS UNIT 1

I. DESCRIPTION OF DEFICIENCY

During system walkdown, it was discovered that a mechanical shock suppressor, item 52, of the reactor coolant system was installed per Rev. 1 of drawing 13-RC-089-H-00F, which specified a #1/2 suppressor with a design load of 600 pounds. This suppressor was accepted by QC per Rev. 2 of drawing 13-RC-089-H-00F which had been revised to require a #1 suppressor with a design load of 2,166 pounds. Final QC inspection did not identify the #1/2 suppressor as being incorrect per the inspection drawing.

II. ANALYSIS OF SAFETY IMPLICATIONS

This condition is evaluated as reportable. The deficiency, if left uncorrected, could lead to a potentially safety significant condition. The installed #1/2 suppressor has inadequate load capacity (600 pounds) based upon the design load of 2,166 pounds, compromising the integrity of the reactor coolant system. A detailed analysis of the safety implications was not performed, as the condition is being corrected.

III. CORRECTIVE ACTION

Nonconformance Report PC-5360 will be dispositioned to replace the existing #1/2, item 52, suppressor with a correct #1, item 52, as per Rev. 2 of drawing 13-RC-089-H-00F. QC inspection and acceptance will be required.

To preclude reoccurrence of this type of problem, a formal training session was held for all QC pipe support engineers on 2/3/83. This class stressed the necessity of QC reviewing the pipe support Bill of Materials and comparing this to the vendor ID plate for correctness of size and type. This training session is documented in a training session report and is on file in QC's permanent files. Also, the project has modified the engineering design release practice by requiring that all late design changes of this type be transmitted to Construction for implementation using Design Change Packages which highlight that a design change was made.