

**LICENSEE EVENT REPORT**

CONTROL BLOCK: 

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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	N	J	S	G	S	2	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5
7	8	14						15	25										26	30				57	CAT	58			
		LICENSEE CODE							LICENSE NUMBER											LICENSE TYPE									

CON'T

0 1 7 8

REPORT SOURCE

60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

DOCKET NUMBER

EVENT DATE

REPORT DATE

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 On February 10 and 11, 1983 during routine surveillance testing, two steam generator  
0 3 snubbers failed to meet requirements for both the acceleration or bleed rate tests.  
0 4 Similar degradation had been discovered in testing of Salem Unit 1 snubbers in  
0 5 October 1982. An engineering evaluation at that time had shown that, in the design  
0 6 basis event, no failure of the Reactor Coolant System piping would occur. The snubber  
0 7 degradation involved the potential failure of components which could prevent performance  
0 8 of safety equipment as analyzed in the FSAR in accordance with Technical Specification  
7 8 9 6.9.1.8e. 80

SYSTEM CODE C B (11)		CAUSE CODE B (12)		CAUSE SUBCODE Z (13)		COMPONENT CODE S U P P O R T (14)		COMP. SUBCODE D (15)		VALVE SUBCODE Z (16)	
7 8		9 10		11 12		13 18		19 20			
(17) LER RO REPORT NUMBER		EVENT YEAR 8 3 (21) (22)		SEQUENTIAL REPORT NO. 0 0 7 (24) (25) (26)		OCCURRENCE CODE / 0 1 (27) (28) (29)		REPORT TYPE T (30)		REVISION NO. 0 (31) (32)	
ACTION TAKEN B (18)		FUTURE ACTION F (19)		EFFECT ON PLANT Z (20)		SHUTDOWN METHOD Z (21)		HOURS 0 0 0 0 (22) (23) (24) (25)		ATTACHMENT SUBMITTED Y (26)	
33 34		35 36		37 38 39 40		41 42		43 44		45 46 47 48	
NPRD-4 FORM SUB. Y (24)		PRIME COMP. SUPPLIER N (25)		COMPONENT MANUFACTURER R 2 3 6 (26) (27) (28) (29)							

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Investigation of Unit 1 problems had revealed that the snubber piston seal rubber

1 1 material had flattened due to aging. The snubbers had been assembled in 1974; shelf

1 2 life of the seals is now estimated to be 7 years. The seals will be rebuilt using

1 3 original parts. Seals with improved materials are presently being investigated.

1	4											80		
7	8	9												
FACILITY STATUS			% POWER			OTHER STATUS			METHOD OF DISCOVERY			DISCOVERY DESCRIPTION		
1	5	G	28	0	0	0	29	NA	B	31	Surveillance Testing			32
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21

ACTIVITY CONTENT  
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 2 (33) 2 (34) NA NA

7 8 9 10 11 44 45 80

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	(37)	2	(38)	NA	(39)

PERSONNEL INJURIES		DESCRIPTION
NUMBER		
1	8	NA

1	9	Z	42	NA	8303140243 830304 PDR ADQCK 05000311 S PDR
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7 8 9 10 80

PUBLICITY

ISSUED DESCRIPTION (45)

2 0 N (44) NA

NRC USE ONLY

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NAME OF PREPARER R. Frahm

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