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Director
Office of Nuclear Reactor Regulation
U S Nuclear Regulatory Commission
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Unit 1 Inservice Inspection Summary Report

Attached for your review are four copies of the report, "Inservice Inspection-Examination Summary, Prairie Island Nuclear Generating Plant - Unit 1, November 15, 1982 to January 10, 1983." This is the second inservice inspection conducted for inspection period three.

This report identifies the components examined, the examination methods used, the examination number and summarizes the examination results of each of the following areas:

1. Plant Systems

- a) Pressure retaining components and supports of the reactor coolant and associated systems classified as ASME Class I and ASME Class 2.
- b) Seismic Bolting Program.

2. Eddy Current Examination of Steam Generator Tubing.

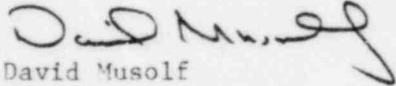
The evaluation of the results from the inservice examinations indicated that the integrity of the systems has been maintained.

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This report is being submitted in accordance with the Prairie Island ASME Code Section XI Inservice and Testing Program submitted to the Commission for review of February 1, 1978 (revised through Revision 7 dated December 23, 1981). As noted in that program, submittal of the attached summary is intended to satisfy the inspection reporting requirements contained in IWA-6220 of Section XI of the ASME Code.



David Musolf
Manager - Nuclear Support Services

DMM/KNC/bd

Attachment

cc: Regional Admin-III, NRC (2 copies of attachment)
NRR Project Manager, NRC (1 copy of attachment)
NRC Resident Inspector (w/o attachment)
G Charnoff
H Baron, Chief Boiler Insp., State of MN (1 copy of attachment)
S Tack, Hartford Ins. (1 copy of attachment)