

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 ALB R F 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CONT

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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 Following a reactor scram on 2/5/83, MSRV 1-1-22 was manually opened and failed

03 to close. The unit was brought to cold shutdown. The pilot cartridge was

04 replaced. On 2/8/83 during startup from cold shutdown, MSRV 1-1-22 apparently

05 opened at 178 psig. The reactor was placed in cold shutdown. There was no

06 effect on public health or safety. The remaining MSRVs were operable.

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08

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

0 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

17 LER/RO REPORT NUMBER 8 3 1 0 0 6 1 0 1 T 0

18 ACTION TAKEN 19 FUTURE ACTION 20 EFFECT ON PLANT 21 SHUTDOWN METHOD 22 HOURS 23 ATTACHMENT SUBMITTED 24 NPD-4 FORM SUB. 25 PRIME COMP. SUPPLIER 26 COMPONENT MANUFACTURER

1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Investigation revealed excessive pilot valve leakage and the bracket for the pilot

11 inlet tube had broken and the tube was lodged in the MSRV seat, preventing closure.

12 The Target Rock Model 67F relief valve was replaced. During the next refueling

13 outage for each unit, the inlet tube bracket for each MSRV will be inspected

14 and repaired if necessary.

15 FACILITY STATUS 16 POWER 17 OTHER STATUS 18 METHOD OF DISCOVERY 19 DISCOVERY DESCRIPTION

1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

16 ACTIVITY CONTENT RELEASED OF RELEASE 17 AMOUNT OF ACTIVITY 18 LOCATION OF RELEASE

1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

17 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION

1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

18 PERSONNEL INJURIES NUMBER DESCRIPTION

1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION

1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

20 PUBLICITY ISSUED DESCRIPTION

1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

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NRC USE ONLY

NAME OF PREPARER E. Holder

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Tennessee Valley Authority  
Browns Ferry Nuclear Plant

Form BF 17  
BF 15.2  
2/12/82

## LER SUPPLEMENTAL INFORMATION

BFRO-50-259 / 83006 Technical Specification Involved 3.6.0

Reported Under Technical Specification 6.7.2.a.(9)\* Date Due NRC 2/21/83

Event Narrative:

On February 5, 1983, unit 1 was operating at 100% power, unit 2 was in a refueling outage, and unit 3 was operating at 100% power. Only unit 1 was affected by this event. While testing the main turbine overspeed trip, the reactor scrammed. MSRV 1-1-22 was manually opened to control reactor pressure and failed to close. The reactor was brought to cold shutdown. Based on past experiences, the problem was attributed to a malfunction of the pilot valve, which was replaced. MSRV 1-1-22 was manually actuated with no reactor pressure and appeared to have closed. During startup of the unit on 2/8/83, MSRV 1-1-22 opened at 178 psig and failed to close (before the reactor reached 250 psig, where the valve was to be tested). The reactor was placed in cold shutdown to investigate. The Target Rock Model 67F, serial no. 1070, was replaced with serial no. 1076.

An investigation revealed that the pilot inlet tube mounting bracket had broken, permitting the inlet tube to get under the seat of the MSRV. During the next refueling outages, these brackets will be inspected, the bracket welds will be liquid penetrant inspected and any necessary repairs will be made.

On both of these events a Notification of Unusual Event (Radiological Emergency Plan) was initiated. For information regarding related events, BFRO 50-259/83007 is being submitted.

\* Previous Similar Events:

None with the two stage valves now installed. Prior to installation of the two stage valves, there are five events of relief valves opening and failing to reseal, which are as follows: 259/77013, 260/78003, 260/78004, 296/78008, and 296/78021. The root cause of these events is unrelated.

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

\*Revision: JRP