NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

Update Report
Previous Report Date - 12-17-81

CONTROL BLOCK: | | | | | | | (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	LICENSEE CODE (2): G A E I H 1	LICENSE NUMBER (3): 0 0 - 0 0 0 0 0 - 0 0	LICENSE TYPE (4): 4 1 1 1 1	CAT (5):

CON'T 0 1	REPORT SOURCE (6): L	DOCKET NUMBER (7): 0 5 0 0 0 3 2 1	EVENT DATE (8): 1 2 0 3 8 1	REPORT DATE (9): 0 2 1 5 8 3

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 Personnel noted that a Unit 2 torus to reactor building vacuum relief
0 3 isolation valve failed to close. This is a potential loss of primary
0 4 containment integrity (Unit 2 Tech. Specs. 3.6.1.1). During subsequent
0 5 work, personnel discovered conditions of potential inoperability of the
0 6 torus to reactor building dP switches for both units (Unit 1 Tech. Specs.
0 7 3.7.A.3 and Unit 2 Tech. Specs. 3.6.4.2). The health and safety of the
0 8 public were not affected by this non-repetitive event.

	SYSTEM CODE (11)	CAUSE CODE (12)	CAUSE SUBCODE (13)	COMPONENT CODE (14)	COMP. SUBCODE (15)	VALVE SUBCODE (16)
0 9	S H	B	C	I N S T R U	S	Z

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 1	—	1 2 9	/	0 1	X	—	1

ACTION TAKEN (18)	FUTURE ACTION (19)	EFFECT ON PLANT (20)	SHUTDOWN METHOD (21)	HOURS (22)	ATTACHMENT SUBMITTED (23)	NPRD-4 FORM SUB. (24)	PRIME COMP. SUPPLIER (25)	COMPONENT MANUFACTURER (26)
G	F	Z	Z	0 0 0 0	Y	N	A	B 0 8 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of this event was water accumulation in the torus to reactor
1 1 building pressure sensing lines. This accumulation was due to improper
1 2 sloping of these lines during installation. Corrective action (i.e.,
1 3 installation of lines with the correct slope as per a design change) was
1 4 completed for both Unit 1 and 2 by 6-2-82.

	FACILITY STATUS (28)	% POWER (29)	OTHER STATUS (30)	METHOD OF DISCOVERY (31)	DISCOVERY DESCRIPTION (32)
1 5	E	0 9 9	NA	A	Engineering Evaluation

	ACTIVITY RELEASED (33)	CONTENT OF RELEASE (34)	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z	Z	NA	NA

PERSONNEL EXPOSURES

	NUMBER (37)	TYPE (38)	DESCRIPTION (39)
1 7	0 0 0	Z	NA

PERSONNEL INJURIES

	NUMBER (40)	DESCRIPTION (41)
1 8	0 0 0	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE (42)	DESCRIPTION
1 9	Z	NA

PUBLICITY

	ISSUED (44)	DESCRIPTION (45)	NRC USE ONLY
2 0	N	NA	

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PDR ADOCK 05000321
S PDR

NAME OF PREPARER: H. L. Sumner - Supt. Plt. Eng. Serv. PHONE: 912-367-7851

GPO 917-926

LER #: 50-321/1981-129, Rev. 1
Licensee: Georgia Power Company
Facility Name: Edwin I. Hatch
Docket #: 50-321

Narrative Report
for LER 50-321/1981-129, Rev. 1
Update Report - Previous Report Date 12-17-81

On December 1, 1981, while Unit 1 and Unit 2 were in steady state power operation, site personnel noted that a Unit 2 torus to reactor building vacuum relief isolation valve failed to close after normal drywell venting was complete. The event could possibly have resulted in a loss of primary containment integrity (Unit 2 Tech. Specs. 3.6.1.1). While performing work per a Maintenance Request, personnel discovered that the reactor building to torus pressure sensing lines on Unit 2 had low sections which would allow water to accumulate. This could prevent the reactor building to torus vacuum breakers from correctly operating (Unit 2 Tech. Specs. 3.6.4.2). Additional investigation indicated that the same situation existed on Unit 1 (Unit 1 Tech. Specs. 3.7.A.3). The health and safety of the public were not affected by this non-repetitive event.

The cause of this event has been attributed to error in the installation of the reactor building to torus pressure sensing lines. This error (incorrect sloping of these lines) was corrected by a design change; this work was completed for both units by 6-2-82.