

INDIAN POINT 3
SERVICE WATER SYSTEM

DRAFT

A. SUMMARY

A.1 INTRODUCTION

The service water system (SWS) acts as a heat transport medium, removing heat from the components that it supplies, and transferring this heat to the Hudson River. This system is necessary to remove reactor decay heat under LOCA conditions and most transient events.

This analysis is carried out under the following conditions:

- The system is quantified to the first isolation valve capable of isolating a major component. The component isolation valves are, in most cases, included with the components which are isolated; for example, the component cooling heat exchanger isolation valves are included with the component cooling heat exchangers.
- Operator interaction with the SWS to correct deficiencies or to recover the system following failure are considered for the mode selector switch; and, after the first hour, for errors in shifting the system lineup.
- Prior to the initiating event, two service water pumps were supplying the nuclear header and two service water pumps were supplying the conventional header.
- Success of the system is defined as two service water pumps supplying the nuclear header for 24 hours after the initiating event, and one service water pump supplying the conventional header for 24 hours after the initiating event.

A.2 RESULTS

Table 1 summarizes the results obtained for the service water system analysis. Two cases for electric power are presented: "No Loss of Offsite Power" and "Loss of Offsite Power." Three boundary conditions for each electric power case are analyzed: "Power at all 480V buses"; "Loss of power at a single 480V bus"; and "Loss of power at two 480V buses."

The analysis has revealed the following dominant contributors to failure of the SWS to supply water to time $t = 24$ hours:

- | | <u>Mean</u> |
|--|----------------------|
| • Case 1 - No Loss of Offsite Power | |
| - Power at all 480V buses. | |
| • Mispositioned mode selector switch (96.4%) | 2.2×10^{-3} |

	<u>Mean</u>
- Loss of power at a single 480V bus	
• Random failure of the pumps selected to supply the nuclear header (63%)	3.9×10^{-3}
• Mispositioned mode selector switch (37%)	2.2×10^{-3}
- Loss of power at two 480V buses	
• Failure of the pumps selected to supply the nuclear header due to insufficient power (100%)	1.0
• Case 2 - Loss of Offsite Power	
- Power at all 480V buses	
• Mispositioned mode selector switch (93%)	2.2×10^{-3}
- Loss of power at one 480V bus	
• Random failure of the pumps selected to supply the nuclear header (69.5%)	5.3×10^{-3}
• Mispositioned mode selector switch (29%)	2.2×10^{-3}
- Loss of power at two 480V buses	
• Failure of the pumps selected to supply the nuclear header due to insufficient power (100%)	1.0

No comparison is made to WASH-1400 results as there is no comparable system analysis in WASH-1400.

A.2 CONCLUSIONS

The service water system is required to support plant operation and will be operating at the time of the initiating event. For this reason, the system is unusual in relation to the plant standby emergency systems.

Failure of this system is dominated by the human error of mispositioning the mode selector switch. There is no indication in the control room which can aid the operator in determining the correct position for this switch during normal plant operation; and, under the assumptions used in this analysis, if undetected, this error leads directly to system failure.

B. SYSTEM DESCRIPTION

B.1 SYSTEM FUNCTION

The SWS provides water from the Hudson River to various plant components requiring heat removal for proper operation during normal plant operation and during abnormal plant conditions.

This analysis was performed to determine the frequency of failure of the SWS to supply sufficient cooling water to the components required during abnormal plant operations. The components requiring service water during abnormal plant conditions are: the three emergency diesel generators (EDGs); the five containment building ventilation fan cooler recirculation units (FCUs); and the two component cooling heat exchangers.

The SWS consists of three groups of three pumps and two cooling water headers. One group of three pumps normally supplies essential plant services (the nuclear header) which, for this analysis, consists of the EDGs and the containment FCUs. The second group of three pumps supplies nonessential plant services (the conventional header) which, for this analysis, consists of the component cooling heat exchangers. The third group of pumps, backup service water pumps numbers 37, 38, and 39, are provided to supply essential plant services in the event of loss of the normal water supply. The third set of pumps cannot be selected for automatic starting, are not considered a part of the engineered safeguards system and, for these reasons, are excluded from this analysis.

Nuclear plant services and conventional plant services can be supplied from either of the first two groups of pumps. Operator action to manually realign the service water headers is required in order to shift header services. The pumps selected to supply the nuclear header are selected for automatic starting by the control room operator by shifting a mode selector switch.

System failure is defined as "No or Insufficient Flow from the Service Water System" which may be further defined as failure of two of three service water pumps selected for nuclear plant service to supply water to the nuclear services header and failure of one pump selected for conventional plant service to supply water to the conventional services header.

B.2 SYSTEM DIAGRAMS

The block diagram, Figure 1, is a schematic representation of the SWS which depicts the system success requirements.

Figure 2, is a system functional diagram showing the basic system piping arrangements.

B.3 SERVICE WATER SYSTEM AND OPERATION

B.3.1 SYSTEM NORMAL OPERATION

The SWS consists of nine identical vertical, centrifugal sump-type pumps rated at 5,000 gpm at 220 ft. TDH. These pumps supply two independent supply headers. One header normally provides cooling water for the plant essential loads (nuclear header), the other header provides cooling water for the plant nonessential loads (conventional header). The backup pumps are normally lined up to both service water headers; however, these pumps are provided for loss of the normal water supply only.

During normal operation, the essential cooling loads are supplied by two of the three service water pumps aligned to the nuclear header while the nonessential loads are supplied by two of the three service water pumps aligned to the conventional header. During plant emergency conditions, any two of the three service water pumps on the nuclear header can supply all of the cooling water requirements for essential plant equipment; any one of the three service water pumps on the conventional header can supply all of the cooling loads for nonessential equipment.

The service water pumps supplying the essential cooling loads are selected for automatic starting during engineered safeguards system (ESS) actuation by means of a mode selector switch in the control room. This mode selector switch has two positions "4, 5, 6," and "1, 2, 3." The "4, 5, 6" position closes contacts in the ESS actuation control circuits and opens contacts in the safety injection recirculation control circuits for service water pumps 34, 35, and 36. The opposite conditions exist for the service water pumps controlled by the position not selected (for this analysis, pumps 31, 32, and 33). The "1, 2, 3" position performs the same functions for service water pumps 31, 32, and 33 that the "4, 5, 6" position performs for pumps 34, 35, and 36. For the purposes of this analysis, service water pumps 34, 35, and 36 are assumed to be selected to supply essential plant services.

The discharge of each service water pump passes through a self-washing, automatic blowdown strainer. These strainers function to remove particles $>1/8$ " in diameter from the service water flow stream. Cleaning and blowdown of the service water discharge strainers is automatic in response to the following signals: every 24 hours, the strainer is cleaned and blown down for five minutes; and whenever the Δp across the strainer reaches 3 psid, the strainer is cleaned and blown down until the strainer Δp is reduced to <3 psid. The strainer motors receive power from MCCs 36A and 36B which are the essential 480V MCCs for Indian Point 3.

Each service water pump discharges through a check valve and a normally open butterfly valve to its respective service water header. The butterfly valve is used to isolate a service water pump for maintenance, and the check valve is used to prevent back flow through an idle service water pump.

The essential cooling header supplies the following systems and components:

- Containment Building Fan Cooler Recirculation Units.
- Containment Building Fan Cooler Recirculation Units' Fan Motor Cooling Coils.
- Instrument Air Compressors' Closed Cooling System.
- Main Boiler Feed Pump Lube Oil Coolers.
- Main Turbine Oil Coolers.
- Generator Seal Oil Coolers.
- Emergency Diesel Generator Cooling Services.
- Control Room Air Conditioning Unit.

The conventional header supplies the component cooling heat exchangers and various conventional plant loads.

The EDGs and the containment building FCUs have automatic valves which operate in response to a safeguards actuation signal. The operation of these valves is discussed below:

- Service water for the EDGs provides lube oil cooling and jacket water cooling. Each diesel generator has a header supply isolation from both service water supply headers and an outlet isolation valve. The outlet from each diesel combines into a single outlet header and passes to a parallel flow control valve arrangement. These pneumatically operated valves maintain a total flow of 1,200 gpm through the EDGs. In the event of a high diesel generator jacket water temperature or a safeguards actuation signal, the air to the flow control valves is vented from the controller, and the valves go to the full open position. During normal plant power operation, one of the diesel generator flow control valves throttles open to maintain a total diesel generator cooling flow of 400 gpm.
- The service water system supplies cooling water to the five containment building FCUs. This water cools the air in the containment and the fan motors. The inlet and outlet isolation valves for each unit are located outside the containment building and may be operated during plant abnormal conditions. The outlet line from each fan motor cooler is combined into a single header and passes through a flow indicator and temperature indicator. Flow through the fan motor coolers is maintained at 55 gpm (total). The outlets from the FCU cooling coils are combined into a common outlet header. During normal plant operation, the service water flow from the FCUs is throttled to maintain containment ambient temperature <120°F. Two parallel bypass valves around the

temperature control valve are automatically opened in response to a safeguards actuation signal. As these valves are pneumatically closed, loss of air or loss of power to the associated solenoid valve causes the bypass valves to open. The containment building FCU automatic outlet valves are included with the containment fan cooler system description as failure of the valves to open does not result in SWS failure.

The normal service water pumps are located in a common well of the intake structure and take their suction from this well. The service water well receives river water through its own debris screen system and from either or both of the circulating water pump wells. The gates on the main circulating supply openings are normally open. The backup pumps are located in the plant discharge canal. This location allows the pumps to supply essential plant service in the event of a loss of the normal service water supply.

Table 2 lists the required flows for the SWS for various plant conditions. During normal operation, two nuclear service water pumps and two conventional service water pumps are required to support power operation. During LOCA conditions, two of the three nuclear service water pumps are required to start and run. After the shift from injection to recirculation, two nuclear service water pumps are required to continue running and one conventional service water pump is required to start and run.

B.3.2 SYSTEM OPERATION DURING EMERGENCY CONDITIONS

Four conditions of emergency operation of the SWS are discussed below. These conditions are: "Unit Trip, No Blackout, No Safety Injection"; "Unit Trip, Blackout, No Safety Injection"; "Unit Trip, No Blackout and Safety Injection"; and "Unit Trip, Blackout, and Safety Injection."

1. Unit Trip, No Blackout and No Safety Injection. When the unit trips and no blackout or safety injection occurs the system will remain in operation as it was before the event, since all power requirements will have transferred from the unit auxiliary transformer to the station auxiliary transformer, and the service water pumps will continue to operate.
2. Unit Trip, With Blackout and No Safety Injection. When this condition occurs, all service water pumps are tripped. Electrical power is reestablished using the EDGs and the essential service water header is automatically reestablished in order to support the EDG cooling requirements. The nonessential service water header is then operator reestablished to support the component cooling heat exchangers since a component cooling pump is automatically started by this event. By reestablishing these headers all other loads are then supplied with water as necessary.
3. Unit Trip, No Blackout and Safety Injection. Under this condition, the service water pumps which were running remain running, and an additional pump selected by the mode selector switch to supply the essential heater will start.

4. Unit Trip, With Blackout and With Safety Injection. When this condition occurs, all service water pumps are tripped. Electrical power is reestablished using the EDGs. The ESS safeguards sequence signal then starts the service water pumps selected by the mode selector switch (right-hand safeguards panel) to supply the essential header.

When active safety injection is completed and the recirculation phase is entered into, two pumps (with outside power or three diesels running) or one pump (with only two of the diesels running) of the nonessential header will be started by recirculation phase switches RS-2 and RS-5. RS-2 will start one pump; RS-5 will start the other pump if three diesels are running or outside power is available. Assuming pumps 34, 35, and 36 were chosen for essential loads, essential service pumps 31, 32, and 33 would be used for the recirculation phase. Positioning switch RS-2 to "On" would start pump 31; if it failed to start, pump 32 would be started; and if 32 failed to start, pump 33 would be started. Switch RS-2 is independent of power available. Positioning switch RS-5 to "On" will cause the following action only if three diesels are running or outside power is available: pump 32 will be started; if it is running and pump 31 is not running, pump 31 is started; if pump 31 is running and pump 32 fails to start, pump 33 is started.

B.4 INSTRUMENTATION AND CONTROLS

The operator controls the service water pumps from the control room. In addition to the mode selector switch which determines the service water pumps selected for automatic starting during the injection phase of an ESS actuation, the operator has individual pump breaker control switches in the control room at the cooling water and air panel. There are nine switches, one for each service water pump. Each breaker control switch for the normal service water pumps has four positions (spring-return-to-Auto):

- Pull Out. The pump is disabled from starting by any automatic start signal. With the switch in this position, the "Safeguards Equipment Locked Open" alarm will be annunciated in the control room on the safeguards panel.
- Stop. The pump breaker is tripped open.
- Auto. The pump will be started for any one of the following conditions:
 - If selected for essential service, the pump will start upon receipt of a safety injection signal or a unit trip with blackout and no safety injection signal.
 - If not selected for essential service, the pump will start when the recirculation phase switches are positioned for the blackout and safety injection condition. For the unit trip with blackout condition, operator action is necessary to start a nonessential service water pump.
- Start. The pump will be started.

while the pump is running, it may be tripped by:

- Placing the control switch to the Stop or Pull Out position
- Overload.

The backup service water pump breakers are controlled from the control room by "On-Off" switches on the cooling water and air panel. In addition to the control room breaker control switches, each normal pump may be started locally at the 480V switchgear. To start a pump locally, a "Local/Remote" selector switch is positioned to "Local" and the pump is started. With the "Local/Remote" selector switch in "Local," the pump cannot be started automatically, and all control room indication of pump breaker status is lost. This condition is annunciated in the control room.

The controls for the self-cleaning strainers are located at the intake structure or by the backup pumps, each strainer having its own control panel. Three switches are located on each panel: a disconnect switch which removes all power from the unit; a three-position (Off-Hand-Auto) condition of operation selector; and a two-position (Intermittent-Continuous) mode of operation selector. A "Service Water Strainer Trouble" alarm is indicated in the control room if strainer $\Delta p > 3$ psid or if trouble has occurred to defeat the normal operation of the strainer motor.

During operation, pressure in the service water headers is monitored and an alarm is sounded in the control room if abnormally high or low. In addition, flow indicators and temperature detectors indicate the condition of the components served by the SWS.

B.5 TECHNICAL SPECIFICATIONS

The plant technical specifications require three service water pumps on the designated essential header and two pumps on the designated nonessential and their associated piping and valves be operable at all times. If, during power operation, these requirements cannot be met within twelve hours the operators shall proceed to bring the reactor to the cold shutdown condition utilizing normal operating procedures. In addition, the isolation between the headers shall be maintained at all times except for a period of 8 hours allowed to shift service water header services.

B.6 TESTING AND MAINTENANCE

B.6.1 TESTING AND SURVEILLANCE

Periodic testing of the service water pumps is required by ASME Section XI. The required frequency of testing is monthly during normal plant operation. Plant test procedure 3PT-M35 tests all service water pumps monthly. Successful completion of this test requires that the service water pump under test run successfully for 15 minutes.

Automatic starting of the service water pumps in response to a safety injection signal is performed every refueling cycle in conjunction with EDG tests. In addition to the above tests, plant operating procedures require shifting of running service water pumps weekly.

Monitoring of the SWS is performed routinely by the plant operators to determine system status.

B.6.2 MAINTENANCE

Periodic maintenance is performed as required on the service water pumps and associated equipment. Due to the plant technical specification limits on the number of operable service water pumps on the nuclear header, periodic maintenance is not performed on a service water pump that is lined up to this header. Instead, the header services are shifted and the pump is placed out of service for maintenance. The periodic maintenance consists of a range of actions from clearing and servicing the self-cleaning strainers to pump disassembly and seal replacement.

Based upon our review of plant work permits, the probability of a service water pump being out of service for maintenance is:

Mean: 1.47×10^{-2}

Variance: 3.95×10^{-5} .

Upon the completion of pump train maintenance, testing is required to determine pump operability and to ensure correct system lineup. At the completion of this testing, the pump is returned to its normal lineup.

B.7 SUPPORT SYSTEMS

The system required for operation of the SWS is the electric power system. Table 3 identifies the service water pumps and the electrical power supplies for the pumps.

B.8 OPERATOR INTERACTION WITH THE SERVICE WATER SYSTEM

Operator action or inaction can affect the operation of the SWS. As this system is required to support normal plant operation, most operator errors that affect system operation will be annunciated or indicated in the control room within a short period of time after their occurrence. These actions include shifting pump control to Local (annunciated in the control room), inadvertently securing a running service water pump (alarmed in the control room), and mispositioning manual valves which control service water flow to individual components or groups of components. Because of the reasons stated above and because the SWS is required to be in operation to support plant operation, these types of human errors do not contribute to the failure of the SWS to start and continue running following an initiating event.

The operator may affect the automatic starting of the service water pumps selected to supply the nuclear services header by mispositioning the mode selector switch in the control room. This results in the wrong set of service water pumps starting automatically in the event of a safeguards actuation or blackout condition. If uncorrected, this condition results in failure of the EDGs due to overheating. This condition is indicated immediately after ESS actuation by low flow alarms on the containment building FCUs followed shortly by low flow alarms and high temperatures on the EDGs. This condition is rapidly corrected in the control room by starting the required service water pumps and securing the misselected service water pumps, and returning the mode selector switch to the correct position (this allows subsequent restart of the service water pumps selected to the conventional header during the shift to recirculation).

From NUREG/CR-1278*, the error of omission while using a procedure with check-off provisions is 0.001 (.0001 to .005) (Table 20-21). Using the guidance presented in the Human Error Rates section of this report, we take the upper bound as the 90th percentile value and the best estimate as the median value of a lognormal distribution. This leads to the following distribution for the error of omission when written procedures are used in nonpassive tasks:

Mean: 2.20×10^{-3}

Variance: 1.88×10^{-5} .

The probability of not discovering this error (switch position only) from Table 20-28 (Item 21) of NUREG/CR-1278 is 0.98 (.94 to .994). Again, using the guidance presented in the Human Error Rates section of this report, we take the upper bound (.994) as the 90th percentile value, and the best estimate (.98) as the median value of a lognormal distribution. This leads to the following distribution for the probability of not discovering the mispositioned switch:

Mean: 9.80×10^{-1}

Variance: 1.93×10^{-4} .

Combining the error of omission with the probability of not discovering the error, we obtain the frequency of occurrence for this mispositioned switch. The mean and variance for this event are:

Mean: 2.16×10^{-3}

Variance: 1.48×10^{-5} .

Swain, A.D. and H.E. Guttman, "Handbook of Human Reliability Analysis with Emphasis on Nuclear Power Plant Applications," *NUREG/CR-1278, Draft Report, April 1980.

On demand the operator will receive immediate indication that system operation is incorrect (low flow alarms, etc.); however, the operator is also receiving numerous indications and alarms. The operator will probably pass over the first low flow alarms and miss the status of the SWS until diesel generator temperature alarms occur. At this point, with a loss of offsite power, system failure due to loss of electric power is imminent and, for this analysis, system failure is assumed to occur.

B.9 COMMON CAUSE ANALYSIS

The normal service water pumps are located in a common well of the intake structure. The backup service water pumps are located at the discharge canal. The intake structure is designated as Seismic Class I, as is the SWS itself. The breakers for the service water pumps are located at the 480V essential switchgear at elevation 15' of the control building. The manual valves for aligning the service water supplies to the various components served are located in common locations throughout the plant (i.e., diesel generator valves are located together, containment building supply valves are located together).

Common generic components in the SWS are manufactured by the same manufacturer, are subject to common maintenance, test, and operation procedures, and have common susceptibilities to secondary causes of failure (grit, moisture, vibration). Table 4 presents a listing of the components included in the SWS analysis and the susceptibilities to the various categories of failure. Further discussion of the effect of common cause failures on system failure is presented in Section D.6 of this analysis.

C. SYSTEM MODELING

C.1 EVENT TREES

The top event for the SWS in the event trees for Indian Point 3 is "No or Insufficient Flow from the Service Water System." This is further defined as failure of the pumps selected to the nuclear header to start or run, or failure of the pumps selected to the conventional header to start or run. In the event trees, the SWS appears with the events concerning the containment fan coolers.

C.2 FAULT TREES

Figure 3 presents the system fault tree developed for the Indian Point 3 SWS. The top event, "No or Insufficient Flow (NOIF) from Service Water System" is defined as no or insufficient flow from service water header no. 1 (nuclear) or no flow from service water header no. 2 (conventional). For the purposes of this analysis, service water pump nos. 34, 35, and 36 are aligned to supply the nuclear header and pump nos. 31, 32, and 33 are aligned to supply the conventional header. The assumptions used in developing the fault tree are presented below.

- Four pumps were in operation prior to the initiating event - two on the nuclear services header, and two on the conventional services header.
- A safeguards actuation signal is present.
- The system is analyzed to the first valve which is capable of isolating a major component or group of components.
- System success requires that two pumps on the nuclear services header and one pump on the conventional services header operate for 24 hours.

C.3 BASIC EVENT CODING

Table 4 presents the basic event coding used in the SWS analysis and discusses the failure modes associated with these basic events. Also included is the failure data used in this analysis and the location and susceptibility coding for the basic events.

C.4 BLOCK DIAGRAM

Figure 1 presents the block diagram developed for the SWS. The components contained in each block of this diagram are presented below.

- Block A - Service Water Intake. The service water intake consists of the service water screens and the bypass gates. As failures in the screens and bypass gates cause failure of the nuclear and conventional service water headers, we treat the service water intake as a single failure point for the SWS.

- Block B - Service Water Pump Number 36 Train. Consists of the following components:

TPMP36-S	Pump 36 fails mechanically (start or run)
TMOF36-S	Pump 36 motor fails (start or run)
TCCP36-F	Pump 36 control circuit fails
JBS-36AD	No power from bus 6A
4BS-332D	No DC control power at bus 6A
TXVSW26C	Pump discharge valve transfers closed
TCVSW16C	Pump discharge check valve fails closed
TFLSW46G	Strainer SWN-4-6 plugs
TXJSW36L	Expansion joint SWN-3-6 ruptures.

- Block C - Service Water Pump Number 35 Train. Consists of the following components:

TPMP35-S	Pump 35 fails mechanically (start or run)
TMOF35-S	Pump 35 motor fails (start or run)
TCCP35-F	Pump 35 control circuit fails
JBS-334C	No power from bus 3A
4BS-333D	No DC control power at bus 3A
TXVSW25C	Pump discharge valve transfers closed
TCVSW15C	Pump discharge check valve fails closed
TFLSW45G	Strainer SWN-4-5 plugs
TXJSW35L	Expansion joint SWN-3-5 ruptures.

- Block D - Service Water Pump Number 34 Train. Consists of the following components:

TPMP34-S	Pump 34 fails mechanically (start or run)
TMOF34-S	Pump 34 motor fails (start or run)
TCCP34-F	Pump 34 control circuit fails
JBS-35AD	No power from bus 5A
4BS-331D	No DC control power at bus 5A
TXVSW24C	Pump discharge valve transfers closed
TCVSW14C	Pump discharge check valve fails closed
TFLSW44G	Strainer SWN-4-4 plugs
TXJSW34L	Expansion joint SWN-3-4 ruptures.

- Block E - Service Water Pump Number 33 Train. Consists of the following components:

TPMP33-S	Pump 33 fails mechanically (start or run)
TMOF33-S	Pump 33 motor fails (start or run)
TCCP33-F	Pump 33 control circuit fails
JBS-36AD	No power from bus 6A
4BS-332D	No DC control power at bus 6A
TXVSW23C	Pump discharge valve transfers closed
TCVSW13C	Pump discharge check valve fails closed
TFLSW43G	Strainer SWN-4-3 plugs
TXJSW33L	Expansion joint SWN-3-3 ruptures.

- Block F - Service Water Pump Number 32 Train. Consists of the following components:

TPMP32-S	Pump 32 fails mechanically (start or run)
TMOP32-S	Pump 32 motor fails (start or run)
TCCP32-F	Pump 32 control circuit fails
JBS-32AD	No power from bus 2A
4BS-333D	No DC control power at bus 2A
TXUSW22C	Pump discharge valve transfers closed
TCVSW12Q	Pump discharge check valve fails closed
TFLSW42G	Strainer SWN-4-2 plugs
TXJSW32L	Expansion joint SWN-3-2 ruptures.

- Block G - Service Water Pump Number 31 Train. Consists of the following components:

TPMP31-S	Pump 31 fails mechanically (start or run)
TMOP31-S	Pump motor fails (start or run)
TCCP31-F	Pump 31 control circuit fails
JBS-35AD	No power from bus 5A
4BS-331D	No DC control power at bus 5A
TXVSW21C	Pump discharge valve transfers closed
TCVSW11Q	Pump discharge check valve fails closed
TFLSW41G	Strainer SWN-4-1 plugs
TXJSW31L	Expansion joint SWN-3-1 ruptures.

- Block H - Diesel Generator Outlet. Consists of the following component:

TFVS76-Q	FCV-1176 does not open.
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- Block I - Diesel Generator Outlet. Consists of the following component:

TFVS76AQ	FCV-1176A does not open.
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- Block J - Nuclear Header Piping and valves. Consists of the following components:

TXVSW99C	Header discharge valve transfers closed
TCV100AQ	Header discharge check valve fails closed
TPP1FAIL	Nuclear header piping fails (identified in Table 6).

- Block K - Conventional Header Piping and Valves. Consists of the following components:

TXVSW98C	Header discharge valve transfers closed
TCV100BQ	Header discharge check valve fails closed
TPP2FAIL	Conventional header piping fails (identified in Table 6).

C.5 CUTSETS

Table 5 presents the minimal cutsets which lead directly to the top event of the fault tree.

D. QUANTIFICATION

Quantification of the SWS is based upon the reliability block diagram presented as Figure 1. The components on these blocks are described in Section C of this analysis and defined as a train for the purposes of this analysis.

In the following sections, quantification of the nuclear header and the conventional header contributions to system failure are presented separately. The results of the header analysis are combined to present the probability of system failure.

D.1 SINGLE FAILURES

Observation of Figure 1 reveals the service water intake (Block A) as a single contributor to SWS failure. As this train can fail the nuclear and the conventional service water headers, this failure is treated as a single failure.

This failure consists of failures of the service water intake screen and the two screen bypass gates. The bypass gates are normally open.

Because the bypass gates are normally open, and must be manually inserted to block flow, no credible failure mechanism was found which could result in these gates transferring closed. The probability of occurrence of water supply failure is assigned a value of zero.

Also included as single failures for the individual headers are Blocks J and K, which consist of: header piping failures; the header discharge isolation valve; and the header discharge check valve. These events are discussed below.

1. Pipe Failures. Table 6 identifies the system effects of pipe failure for various sections of pipe in the SWS. Failures in piping which is less than 10 inches in diameter are not presented. The table is presented in two sections; the nuclear header which is defined as the header supplied by pumps 34, 35, and 36; and the conventional header which is supplied by pumps 31, 32, and 33.

Our mean and variance for the probability of failure of a single pipe section are:

Mean: 8.60×10^{-10}

Variance: 6.00×10^{-17} .

For the nuclear header, Table 6 identifies ten sections of pipe where rupture will cause immediate failure of the SWS. For the conventional header, Table 6 identifies eight sections of pipe where rupture will cause immediate failure of the SWS. This results in the following distribution for the probability of piping failure per hour for the nuclear and conventional service water headers:

Mean (nuclear): 8.60×10^{-9}
 Variance: 4.29×10^{-15}
 Mean (conventional): 6.88×10^{-9}
 Variance: 2.75×10^{-15} .

Piping failures during plant operation are immediately detectable and result in either plant shutdown or header realignment for repair. We are interested in those piping failures which occur after the start of an initiating event. The time period of interest is the first 24 hours after the initiating event. This leads to the following distributions for failure of the nuclear or conventional service water headers due to piping failures:

Mean (nuclear): 2.06×10^{-7}
 Variance: 2.47×10^{-12}
 Mean (conventional): 1.65×10^{-7}
 Variance: 1.58×10^{-12} .

2. Isolation Valve. Each service water header contains a manual header isolation valve and a header check valve. A failure in either of these valves during plant operation is immediately detectable and would require plant shutdown for repair. We are interested in those failures which occur after initiation of the system. The mean and variance for these events are:

Manual Valve Transfers Closed (per hour)	Mean: 9.15×10^{-8} Variance: 1.01×10^{-14}
Check Valve Fails to Open (on demand)	Mean: 6.91×10^{-5} Variance: 1.03×10^{-8} .

With no loss of offsite power, the pumps selected to supply the nuclear header will continue to run and the pumps selected to supply the conventional pumps will trip. The conventional header pumps are later started by operator action. With a loss of offsite power, all pumps are initially tripped and subsequently restarted (either automatically or by operator action).

For the case of no loss of offsite power, we have the following distribution for failure of a header isolation or check valve to time, $t = 24$ hours:

Mean: 2.20×10^{-6}
 Variance: 5.51×10^{-12} .

with a loss of offsite power, or when the pumps are tripped by plant action, we have the following distribution for failure of a header isolation or check valve to time, $t = 24$ hours:

Mean: 7.13×10^{-5}

Variance: 8.93×10^{-9} .

D.2 DOUBLE FAILURES

D.2.1 NUCLEAR HEADER

Nuclear header double contributions to system failure consist of failure of two of the three pump trains and failure of the two diesel generator outlet flow control valves.

1. Service water pump train, Block B. Section C of this analysis presents the components which are included in a single pump train.. Table 4 identifies the failure rates associated with these components. Presented below is the component failure data for a single pump train.

	<u>Mean</u>	<u>Variance</u>
Pump fails (on demand)	1.36×10^{-3}	1.22×10^{-6}
(per hour)	4.68×10^{-5}	1.07×10^{-9}
Motor fails (with pump failure)	-	-
Control circuit fails (with pump failure)	-	-
Switchgear bus fails	*	-
Control power to switchgear bus fails	*	-
Expansion joint ruptures, per hour	8.60×10^{-9}	6.00×10^{-15}
Discharge strainer plugs	**	-
Discharge check valve fails, to open, demand	6.91×10^{-5}	1.03×10^{-8}
Discharge valve transfers closed, per hour	9.15×10^{-8}	1.01×10^{-14}

Blocks C, D, E, F, and G consist of similar components.

For an operating pump train, the probability of failure per hour is described by the following probability distribution (made up of pump failure to continue running, expansion joint rupture, and discharge valve transferring closed):

Mean: 4.69×10^{-5}

Variance: 9.54×10^{-10} .

*Shown for completeness only, quantified in the electric power analysis.

**Negligibly small probability of failure.

For a time $t = 24$ hours, we have the following distribution for failure of an operating pump train:

Mean: 1.13×10^{-3}

Variance: 5.51×10^{-7} .

At Indian Point 3, with no loss of offsite power, and a safeguards actuation signal, the nonvital electric loads are stripped from the 480V switchgear buses. The pumps selected by the mode selector switch to supply the nuclear header will remain operating while the pumps not selected (the conventional header pumps) will be stripped. The standby nuclear header pump will be sent a starting signal by the safeguards sequence system. With a loss of offsite power all service water pumps will be stripped from the 480V switchgear buses. When power is available, the pumps selected to supply the nuclear header will be sent starting signals by the safeguards sequencing system.

The probability of failure to start of a previously running service water pump is made up of: pump failure to start on demand; expansion joint rupture; check valve failure to open on demand; and discharge valve transferring closed. The expansion joint rupture and the discharge valve closure must have occurred while the pump was idle ($t < 1$ hour). The probability of failure on demand for the expansion joint and discharge valve with $t < 1$ hour are:

Mean (expansion joint): 8.60×10^{-9}

Variance: 6.00×10^{-15}

Mean (discharge valve): 9.15×10^{-8}

Variance: 1.01×10^{-14} .

For a single previously operating pump train, the probability of failure on demand is described by the following distribution:

Mean: 1.43×10^{-3}

Variance: 1.12×10^{-6} .

Given a successful start, the service water pump must operate for 24 hours. The total probability of failure (start and operate for 24 hours) for a single previously operating service water pump train is now:

Mean: 2.55×10^{-3}

Variance: 1.36×10^{-6} .

During plant operation, one service water pump selected to supply the nuclear header will be in standby. Pump failure to start on demand and check valve failure to open on demand remain as previously defined; however, the discharge valve and the expansion joint contributions to pump train failure on demand must be developed.

Plant procedures require shifting of the operating service water pumps weekly. Failure in either component could have occurred at any time during this period without being detected. The fault detection time for the failures is taken as one half of the test interval (84 hours). This results in the following distributions for the probability of failure on demand of an expansion joint and the discharge valve:

Mean (expansion joint): 7.22×10^{-7}

Variance: 3.03×10^{-11}

Mean (discharge valve): 7.69×10^{-6}

Variance: 8.03×10^{-11} .

This results in the following probability of failure on demand for a single standby service water pump train:

Mean: 1.44×10^{-3}

Variance: 1.11×10^{-6} .

Given a successful start, the pump must continue to operate for 24 hours, which results in the following probability distribution for failure of a standby service water pump train:

Mean: 2.56×10^{-3}

Variance: 1.36×10^{-6} .

The following expression defines the probability of failure of the nuclear service water header due to pump train failures:

$$Q_{\text{pumps}} = 2[P(\text{OP}) \times P(\text{STBY})] + [P(\text{OP})]^2 + [P(\text{OP})]^2 \times P(\text{STBY})$$

where

$P(\text{OP})$ = Probability of failure of a running (or previously running) pump to $t = 24$ hours.

$P(\text{STBY})$ = Probability of failure of a standby pump to $t = 24$ hours.

With offsite power available, and all 480V switchgear buses energized, the pump train's contribution to header failure for the pumps selected to supply the nuclear header is described by the following distribution:

Mean: 7.59×10^{-6}

Variance: 5.50×10^{-11} .

With a loss of offsite power, and all 480V switchgear buses energized, the pump train's contribution to header failure is described by the following distribution:

Mean: 2.10×10^{-5}

Variance: 2.48×10^{-10} .

2. Diesel generator outlet flow control valves, Blocks H and J. Failure of valves FCV-1176 and FCV-1176A to open on demand or to remain open for 24 hours results in failure of all three EDGs. The probability of a single valve not opening on demand is represented by the following distribution:

Mean: 4.95×10^{-6}

Variance: 4.03×10^{-7} .

For two valves in parallel we obtain the following distribution for the probability of system failure:

Mean: 6.27×10^{-7}

Variance: 1.54×10^{-11} .

D.2.2 CONVENTIONAL HEADER

There is no double failure contribution to conventional header failure as only one of three pump trains is required for system success.

D.3 TRIPLE FAILURES

D.3.1 NUCLEAR HEADER

There is no triple failure contribution to nuclear header failure.

D.3.2 CONVENTIONAL HEADER

The contribution of triple failures to failure of the conventional header consists of failure of all three pump trains.

Upon receipt of a safety injection signal, the pumps selected to supply conventional plant services are stripped from their respective 480V switchgear buses. These pumps are not restarted until the recirculation

phase of LOCA recovery. System success requires that a single pump selected to supply the conventional header start and operate for 24 hours.

The following expression defines the probability of failure for the service water pump trains selected to supply the conventional service water header.

$$Q_{\text{pumps}} = P(1S) [P(OF)^2 \times P(STBY)] + 3 \times P(OS) [P(OF)]^2$$

where

$P(OF)$ = Probability of failure of a previously running pump to $t = 24$ hours (from Section D.2.1)

$P(STBY)$ = Probability of failure of a standby pump to $t = 24$ hours (from Section D.2.1)

$P(1S)$ = Probability of having a standby pump $[1 - P(OS)]$

$P(OS)$ = Probability of maintenance of a service water pump (from Section B.6.2)

With power available at all 480V switchgear buses (with or without offsite power), the probability of conventional header failure due to pump train failure is described by the following distribution:

Mean: 3.61×10^{-7}

Variance: 1.49×10^{-13} .

D.4 TEST AND MAINTENANCE FAILURES

D.4.1 NUCLEAR HEADER

Due to the plant technical specifications and the discussion presented in Section B.6 of this report, no test or maintenance failure contribution to system failure is assigned for the nuclear header.

D.4.2 CONVENTIONAL HEADER

There is no contribution to system failure due to the testing performed upon the service water pumps selected to supply the conventional header.

The maintenance contribution to system failure is included with the quantification of the conventional pump train's effect on system failure.

D.5 HUMAN INTERACTION FAILURES

D.5.1 NUCLEAR HEADER

From Section B.8, the operator error of omission and nondiscovery of the error resulted in the following distribution for the frequency of occurrence of a mispositioned mode selector switch:

Mean: 2.16×10^{-3}

Variance: 1.46×10^{-5} .

This failure results in failure of the SWS.

D.5.2 CONVENTIONAL HEADER

As two service water pumps aligned to the conventional header are always running to support plant operations, no human error contribution is assigned for the conventional header. The human error of failing to shift to recirculation from injection is analyzed with the recirculation system description.

D.6 COMMON CAUSE FAILURES

Although there are many similar components sharing common locations in the SWS, a large portion of the components are not susceptible to most common cause failure mechanisms because they perform a passive function (items such as manual valves, check valves, piping, etc.). Active components of the system, the service water pumps, are more likely candidates for common cause failure and are discussed below. External events such as earthquakes and flooding are discussed elsewhere in this report.

1. Fire. The normal service water pumps are located at the screen well structure. The pumps are not enclosed and the fire loading in the area is low (as defined in the "Review of the Indian Point Station Fire Protection Program"). No fires are postulated in this area that could disable all service water pumps.
2. Moisture. The service water pump motors are designed for outdoor service and are protected by design from the effects of moisture.
3. Grit. Due to the design of the service water pump motors and the location of the service water pumps, airborne grit is not a common cause candidate.
4. Other Causes. Other common cause susceptibilities, such as manufacturer, test and maintenance procedures, etc., are possible contributors to common cause failure of the service water pumps. However, the plant test program, maintenance program, and technical specifications combine to 1) aid in discovery of pump problems and, 2) limit the effects of common cause failures. No quantification is performed for these causes of failure.

NOTE: The service water pump breakers are susceptible to the common cause failure mechanisms of fire, moisture, and grit due to their common location in the switchgear room. Quantification and discussion of these effects is presented in the event tree analysis as these effects would be felt throughout the plant.

D.7 SYSTEM FAILURE

Failure of the service water system is defined as failure of the nuclear header or conventional header to supply sufficient water to time $t = 24$ hours. The frequency of failure for each header is presented separately below. The failure frequency for each header is made up of the following contributors: single event, double event, and triple event random hardware failures; test and maintenance failures (which are included with the triple event failures); human errors which result in system failure; and the common cause contribution to system failure.

1. Nuclear Header Failure. The probability of failure of the nuclear header with offsite power available is characterized by the following mean and variance:

$$\begin{aligned}\text{Mean: } & \alpha_{\text{singles}} + \alpha_{\text{doubles}} + \alpha_{\text{triples}} + \alpha_{\text{test and maintenance}} \\ & + \alpha_{\text{operator error}} + \alpha_{\text{common cause}} \\ & = 2.17 \times 10^{-3}\end{aligned}$$

$$\text{Variance: } 1.41 \times 10^{-5}.$$

The probability of failure of the nuclear header with a loss of offsite power is characterized by the following mean and variance:

$$\text{Mean: } 2.25 \times 10^{-3}$$

$$\text{Variance: } 1.41 \times 10^{-5}.$$

2. Conventional Header Failure. The probability of failure of the conventional header with or without offsite power is characterized by the following mean and variance:

$$\begin{aligned}\text{Mean: } & \alpha_{\text{singles}} + \alpha_{\text{doubles}} + \alpha_{\text{triples}} + \alpha_{\text{test and maintenance}} \\ & + \alpha_{\text{operator error}} + \alpha_{\text{common cause}} + \alpha_{\text{other}} \\ & = 7.18 \times 10^{-5}\end{aligned}$$

$$\text{Variance: } 8.84 \times 10^{-9}.$$

3. System Failure. The frequency of failure of the service water system is made up of those failures in the two headers which result in header failure. For the case "Offsite Power Available" and under the condition "Power Available at All 480V Switchgear Buses," we have the following distribution for the following distribution for the frequency of failure of the service water system to time, $t = 24$ hours:

$$\text{Mean: } 2.24 \times 10^{-3}$$

$$\text{Variance: } 1.41 \times 10^{-5}.$$

For the case "Loss of Offsite Power" and under the condition "Power Available at All 480V Switchgear Buses," we have the following distribution for the frequency of failure of the service water system to time, $t = 24$ hours:

Mean: 2.33×10^{-3}

Variance: 1.41×10^{-5} .

E. SYSTEM QUANTIFICATION WITH DIFFERING BOUNDARY CONDITIONS FOR ELECTRIC POWER

E.1 LOSS OF POWER AT A SINGLE BUS

This condition is defined as loss of power to switchgear buses 5A, 6A, or 2A and 3A. Switchgear buses 2A and 3A are defined as a single bus for the following reason: a single EDG supplies both buses upon a loss of offsite power.

The loss of power at the various switchgear buses only affects the number of service water pump trains available; all other contributions to system failure remain as stated in Section D of this analysis.

For the purposes of this analysis, an operating (or previously operating) service water pump is assumed to have been powered from the switchgear bus that is now unavailable. This is a conservative assumption in that it results in a higher probability of failure for the pump trains. This assumption is used for all loss of power calculations which follow.

1. Effect on the nuclear header. The following expression defines the probability of system failure due to pump train failures for the pumps selected to supply the nuclear header:

$$Q_{\text{pumps}} = P(OP) + P(STBY)$$

where $P(OP)$ and $P(STBY)$ remain as defined in Section C.

For the condition "Offsite Power Available," this results in the following distribution for the probability of header failure due to pump train failure:

Mean: 3.69×10^{-3}

Variance: 1.88×10^{-6} .

The probability of failure of the nuclear header, with offsite power available and a loss of power at a single 480V switchgear bus is:

Mean: 5.85×10^{-3}

Variance: 1.58×10^{-5} .

With a loss of offsite power, the probability of header failure due to pump train failures with a loss of power at a single 480V switchgear bus is:

Mean: 5.11×10^{-3}

Variance: 2.68×10^{-6}

which results in the following distribution for the probability of failure of the nuclear header with a loss of offsite power and loss of power at a single 480V switchgear bus:

Mean: 7.35×10^{-3}

Variance: 1.66×10^{-5} .

2. Effect on the conventional header. The following expression defines the probability of header failure due to pump train failures for the pumps selected to supply the conventional header:

$$Q_{\text{pumps}} = P(1S) [P(OP) \times P(STBY)] + P(OS) [P(OP)]$$

where the terms of the expression remain as defined in Section D. This results in the following distribution for the probability of header failure due to pump train failures:

Mean: 4.40×10^{-5}

Variance: 5.94×10^{-10} .

The probability of conventional header failure with a loss of power at a single 480V switchgear bus (with or without offsite power) is:

Mean: 1.15×10^{-4}

Variance: 9.34×10^{-9} .

3. Effect on the system. The probability of system failure, given a loss of a single 480V switchgear bus and offsite power available is now:

Mean: 5.97×10^{-3}

Variance: 1.58×10^{-5} .

For the "Loss of Offsite Power" case, we have the following distribution for the probability of system failure with loss of a single 480V switchgear bus:

Mean: 7.46×10^{-3}

Variance: 1.66×10^{-5} .

E.2 LOSS OF POWER AT TWO BUSES

This condition is defined as loss of power to switchgear buses 5A and 6A, 5A and 2A and 3A, or 6A and 2A and 3A. The discussion for buses 2A and 3A remains as presented in Section E.1. The assumption concerning which buses are lost is modified as follows: the previously operating service water pumps are assumed to have been powered by the switchgear buses that are now unavailable.

1. Effect on the nuclear header. Loss of power to the combinations of switchgear buses identified above results in failure of the nuclear header due to insufficient flow.
2. Effect on the conventional header. Loss of power to the combinations of switchgear buses defined above leads to the following expression for the probability of pump train failures which result in header failure.

$$Q_{\text{pumps}} = P(1S) \times P(STBY) + P(OS).$$

This results in the following distribution for the probability of header failure due to pump train failures:

$$\text{Mean: } 1.72 \times 10^{-2}$$

$$\text{Variance: } 3.87 \times 10^{-5}.$$

The probability of failure of the conventional header with a loss of power at two 480V switchgear buses is:

$$\text{Mean: } 1.73 \times 10^{-2}$$

$$\text{Variance: } 3.85 \times 10^{-5}.$$

3. Effect on the system. The probability of system failure given loss of two 480V switchgear buses is 1.0 due to the effect of this condition upon the nuclear header pumps.

TABLE 1

SUMMARY OF RESULTS - SERVICE WATER SYSTEM ANALYSIS

Case 1 - Offsite Power Available
Boundary Condition: Power at All 400V Switchgear Buses

	Mean	Variance	5th Percentile	95th Percentile	Median	WASH-1400
A. Nuclear Header						
1. Single events						
a. Piping	2.1×10^{-7}	2.5×10^{-12}	4.0×10^{-10}	6.0×10^{-7}	5.3×10^{-8}	No Compar- able Analysis
b. Header valves	2.2×10^{-6}	5.5×10^{-12}	2.0×10^{-7}	5.5×10^{-6}	1.0×10^{-6}	
2. Double events						
a. Pump drains	7.6×10^{-6}	5.5×10^{-11}	1.4×10^{-6}	1.0×10^{-5}	5.3×10^{-6}	
b. IUG flow control valves	6.3×10^{-7}	1.5×10^{-11}	2.4×10^{-9}	2.0×10^{-6}	7.5×10^{-8}	
3. Human Error	2.7×10^{-3}	1.5×10^{-5}	9.7×10^{-4}	6.5×10^{-3}	1.0×10^{-3}	
4. Common Cause	*	-	-	-	-	
5. Other	*	-	-	-	-	
TOTAL	2.2×10^{-3}	1.4×10^{-5}	1.1×10^{-4}	6.2×10^{-3}	1.0×10^{-3}	
B. Conventional Header						
1. Single events						
a. Piping	1.7×10^{-7}	1.6×10^{-12}	3.0×10^{-10}	4.3×10^{-7}	4.3×10^{-8}	
b. Header valves	7.1×10^{-5}	8.9×10^{-9}	9.0×10^{-6}	2.3×10^{-4}	4.0×10^{-5}	
2. Triple events						
a. Pump drains	3.6×10^{-7}	1.5×10^{-13}	5.1×10^{-8}	9.0×10^{-7}	2.3×10^{-7}	
3. Test and maintenance (with conventional pumps)	*	-	-	-	-	
4. Common Cause	*	-	-	-	-	
5. Other	*	-	-	-	-	
TOTAL	7.2×10^{-5}	8.8×10^{-9}	1.0×10^{-6}	2.0×10^{-4}	4.1×10^{-5}	
C. System						
1. Single events						
a. Piping	3.7×10^{-7}	8.0×10^{-12}	8.0×10^{-10}	1.0×10^{-6}	9.0×10^{-8}	
b. Header valves	7.3×10^{-5}	9.0×10^{-9}	1.0×10^{-5}	2.7×10^{-4}	4.2×10^{-5}	
2. Double events						
a. "Nuclear" Pumps	7.6×10^{-6}	5.5×10^{-11}	1.4×10^{-6}	1.0×10^{-5}	5.3×10^{-6}	
b. IUG flow control valves	6.3×10^{-7}	1.5×10^{-11}	2.4×10^{-9}	2.0×10^{-6}	7.5×10^{-8}	
3. Triple events						
a. "Conventional" Pumps	3.6×10^{-7}	1.5×10^{-13}	5.1×10^{-8}	9.0×10^{-7}	2.3×10^{-7}	
4. Test and maintenance (with conventional pumps)	*	-	-	-	-	
5. Human Error	2.2×10^{-3}	1.5×10^{-5}	9.7×10^{-4}	6.5×10^{-3}	1.0×10^{-3}	
6. Common Cause	*	-	-	-	-	
7. Other	*	-	-	-	-	
SYSTEM TOTAL	2.2×10^{-3}	1.4×10^{-5}	1.9×10^{-4}	6.2×10^{-3}	1.2×10^{-3}	

*Negligibly small failure probability.

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TABLE 1 (continued)

SUMMARY OF RESULTS - SERVICE WATER SYSTEM ANALYSIS

Case 2 - Loss of Offsite Power
Boundary Condition: Power at All 480V Switchgear Buses

	Mean	Variance	5th Percentile	95th Percentile	Median	WASH-1400
A. Nuclear Header						
1. Single events						
a. Piping	2.1×10^{-7}	2.5×10^{-12}	4.0×10^{-10}	6.0×10^{-7}	5.3×10^{-8}	No Compar- able Analysis
b. Header valves	7.1×10^{-5}	8.9×10^{-9}	9.0×10^{-6}	2.1×10^{-4}	4.0×10^{-5}	
2. Double events						
a. Pump trains	2.1×10^{-5}	2.5×10^{-10}	6.1×10^{-6}	4.1×10^{-5}	1.6×10^{-5}	
b. 10% flow control valves	6.3×10^{-7}	1.5×10^{-11}	2.4×10^{-9}	2.0×10^{-6}	7.5×10^{-8}	
3. Human Error	2.2×10^{-1}	1.5×10^{-5}	9.7×10^{-5}	6.5×10^{-3}	1.0×10^{-1}	
4. Common Cause	*	-	-	-	-	
5. Other	*	-	-	-	-	
TOTAL	2.3×10^{-3}	1.4×10^{-5}	2.1×10^{-4}	6.1×10^{-3}	1.2×10^{-3}	
B. Conventional Header						
1. Single events						
a. Piping	1.7×10^{-7}	1.6×10^{-12}	3.0×10^{-10}	4.3×10^{-7}	4.3×10^{-8}	
b. Header valves	7.1×10^{-5}	8.9×10^{-9}	9.0×10^{-6}	2.1×10^{-4}	4.0×10^{-5}	
2. Triple events						
a. Pump trains	3.6×10^{-7}	1.5×10^{-13}	5.1×10^{-8}	9.0×10^{-7}	2.3×10^{-7}	
3. Test and maintenance (with conventional pumps)	*	-	-	-	-	
4. Common Cause	*	-	-	-	-	
5. Other	*	-	-	-	-	
TOTAL	7.2×10^{-5}	8.8×10^{-9}	1.0×10^{-6}	2.0×10^{-4}	4.1×10^{-5}	
C. System						
1. Single events						
a. Piping	3.7×10^{-7}	8.0×10^{-12}	8.0×10^{-10}	1.0×10^{-6}	9.8×10^{-8}	
b. Header valves	1.4×10^{-4}	3.6×10^{-8}	1.8×10^{-5}	4.4×10^{-4}	7.9×10^{-5}	
2. Double events						
a. "Nuclear" pumps	2.1×10^{-5}	2.5×10^{-10}	6.1×10^{-6}	4.1×10^{-5}	1.6×10^{-5}	
b. 10% flow control valves	6.3×10^{-7}	1.5×10^{-11}	2.4×10^{-9}	2.0×10^{-6}	7.5×10^{-8}	
3. Triple events						
a. "Conventional" pumps	3.6×10^{-7}	1.5×10^{-14}	5.1×10^{-8}	9.0×10^{-7}	2.3×10^{-7}	
4. Test and maintenance (with conventional pumps)	*	-	-	-	-	
5. Human Error	2.2×10^{-3}	1.5×10^{-5}	9.7×10^{-5}	6.5×10^{-3}	1.0×10^{-1}	
6. Common Cause	*	-	-	-	-	
7. Other	*	-	-	-	-	
SYSTEM TOTAL	2.3×10^{-3}	1.4×10^{-5}	2.9×10^{-4}	6.1×10^{-3}	1.3×10^{-3}	

*Negligibly small failure probability.
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TABLE 1 (continued)

SUMMARY OF RESULTS - SERVICE WATER SYSTEM ANALYSIS

Case 1 - Offsite Power Available
Boundary Condition: Power Lost at One 480V Switchgear Bus

	Mean	Variance	5th Percentile	95th Percentile	Median	WASH-1400
A. Nuclear Header						
1. Single events						
a. Piping	2.1×10^{-7}	2.5×10^{-12}	4.0×10^{-10}	6.0×10^{-7}	5.1×10^{-8}	No Compar- able Analysis
b. Header valves	2.2×10^{-6}	5.5×10^{-12}	2.8×10^{-7}	5.5×10^{-6}	1.0×10^{-6}	
2. Double events						
a. Pump trains	3.7×10^{-3}	1.9×10^{-6}	1.9×10^{-3}	5.6×10^{-3}	3.4×10^{-3}	
b. ENG flow control valves	6.3×10^{-7}	1.5×10^{-11}	2.4×10^{-9}	2.0×10^{-6}	7.5×10^{-8}	
3. Human Error	2.2×10^{-3}	1.5×10^{-5}	9.7×10^{-5}	6.5×10^{-1}	1.0×10^{-3}	
4. Common Cause	*	-	-	-	-	
5. Other	*	-	-	-	-	
TOTAL	5.9×10^{-3}	1.6×10^{-5}	2.6×10^{-3}	1.0×10^{-2}	5.0×10^{-3}	
B. Conventional Header						
1. Single events						
a. Piping	1.7×10^{-7}	1.6×10^{-12}	3.0×10^{-10}	4.3×10^{-7}	4.3×10^{-8}	
b. Header valves	7.1×10^{-5}	8.9×10^{-9}	9.0×10^{-6}	7.3×10^{-4}	4.0×10^{-5}	
2. Triple events						
a. Pump trains	4.4×10^{-5}	5.9×10^{-10}	1.7×10^{-5}	8.2×10^{-5}	3.8×10^{-5}	
3. Test and maintenance (with conventional pumps)	*	-	-	-	-	
4. Common Cause	*	-	-	-	-	
5. Other	*	-	-	-	-	
TOTAL	1.2×10^{-4}	9.3×10^{-9}	3.7×10^{-5}	2.4×10^{-4}	8.8×10^{-5}	
C. System						
1. Single events						
a. Piping	3.7×10^{-7}	8.0×10^{-12}	8.0×10^{-10}	1.0×10^{-6}	9.8×10^{-8}	
b. Header valves	7.3×10^{-5}	9.0×10^{-9}	1.0×10^{-5}	7.2×10^{-4}	4.2×10^{-5}	
2. Double events						
a. "Nuclear" Pumps	3.7×10^{-3}	1.9×10^{-6}	1.9×10^{-3}	5.6×10^{-3}	3.4×10^{-3}	
b. ENG flow control valves	6.3×10^{-7}	1.5×10^{-11}	2.4×10^{-9}	2.0×10^{-6}	7.5×10^{-8}	
3. Triple events						
a. "Conventional" Pumps	4.4×10^{-5}	5.9×10^{-10}	1.7×10^{-5}	8.2×10^{-5}	3.8×10^{-5}	
4. Test and maintenance (with conventional pumps)	*	-	-	-	-	
5. Human Error	2.2×10^{-3}	1.5×10^{-5}	9.7×10^{-5}	6.5×10^{-1}	1.0×10^{-3}	
6. Common Cause	*	-	-	-	-	
7. Other	*	-	-	-	-	
SYSTEM TOTAL	6.0×10^{-3}	1.6×10^{-5}	2.7×10^{-3}	1.0×10^{-2}	5.1×10^{-3}	

*Negligibly small failure probability.

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TABLE 1 (continued)

SUMMARY OF RESULTS - SERVICE WATER SYSTEM ANALYSIS

Case 2 - Loss of Offsite Power
Boundary Condition: Power Lost at One 400V Switchgear Bus

	Mean	Variance	5th Percentile	95th Percentile	Median	WASH-1400
A. Nuclear Header						
1. Single events						
a. Piping	2.1×10^{-7}	2.5×10^{-12}	4.0×10^{-10}	6.0×10^{-7}	5.3×10^{-8}	No Comparable Analysis
b. Header valves	7.1×10^{-5}	8.9×10^{-9}	9.0×10^{-6}	7.3×10^{-4}	4.0×10^{-5}	
2. Double events						
a. Pump drains	5.1×10^{-3}	2.7×10^{-6}	3.0×10^{-3}	7.6×10^{-3}	5.1×10^{-3}	
b. EOG flow control valves	6.3×10^{-7}	1.5×10^{-11}	2.4×10^{-9}	2.0×10^{-6}	7.5×10^{-8}	
3. Human Error	2.2×10^{-1}	1.5×10^{-5}	9.7×10^{-5}	6.5×10^{-1}	1.0×10^{-1}	
4. Common Cause	*	-	-	-	-	
5. Other	*	-	-	-	-	
TOTAL	7.3×10^{-3}	1.7×10^{-5}	3.0×10^{-3}	1.3×10^{-2}	6.5×10^{-3}	
B. Conventional Header						
1. Single events						
a. Piping	1.7×10^{-7}	1.6×10^{-12}	3.0×10^{-10}	4.3×10^{-7}	4.3×10^{-8}	
b. Header valves	7.1×10^{-5}	8.9×10^{-9}	9.0×10^{-6}	7.3×10^{-4}	4.0×10^{-5}	
2. Triple events						
a. Pump drains	4.4×10^{-5}	5.9×10^{-10}	1.7×10^{-5}	8.7×10^{-5}	3.8×10^{-5}	
3. Test and maintenance (with conventional pumps)	-	-	-	-	-	
4. Common Cause	*	-	-	-	-	
5. Other	*	-	-	-	-	
TOTAL	1.2×10^{-4}	9.3×10^{-9}	3.7×10^{-5}	7.4×10^{-4}	8.8×10^{-5}	
C. System						
1. Single events						
a. Piping	3.7×10^{-7}	8.0×10^{-12}	8.0×10^{-10}	1.0×10^{-6}	9.8×10^{-8}	
b. Header valves	1.4×10^{-4}	3.6×10^{-8}	1.8×10^{-5}	4.4×10^{-4}	7.9×10^{-5}	
2. Double events						
a. "Nuclear" Pumps	5.1×10^{-3}	2.7×10^{-6}	3.0×10^{-3}	7.6×10^{-3}	5.1×10^{-3}	
b. EOG flow control valves	6.3×10^{-7}	1.5×10^{-11}	2.4×10^{-9}	2.0×10^{-6}	7.5×10^{-8}	
3. Triple events						
a. "Conventional" Pumps	4.4×10^{-5}	5.9×10^{-10}	1.7×10^{-5}	8.7×10^{-5}	3.8×10^{-5}	
4. Test and maintenance (with conventional pumps)	-	-	-	-	-	
5. Human Error	2.2×10^{-1}	1.5×10^{-5}	9.7×10^{-5}	6.5×10^{-1}	1.0×10^{-1}	
6. Common Cause	*	-	-	-	-	
7. Other	*	-	-	-	-	
SYSTEM TOTAL	7.5×10^{-3}	1.7×10^{-5}	4.0×10^{-3}	1.7×10^{-2}	6.6×10^{-3}	

*Negligibly small failure probability.

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TABLE 1 (continued)

SUMMARY OF RESULTS - SERVICE WATER SYSTEM ANALYSIS

Case 1 - Offsite Power Available
Boundary Condition: Power Lost at Two 400V Switchgear Buses

	Mean	Variance	5th Percentile	95th Percentile	Median	WASH-1400
A. Nuclear Header						
1. Single events						
a. Piping	2.1×10^{-7}	2.5×10^{-12}	4.0×10^{-10}	6.0×10^{-7}	5.3×10^{-8}	No Compar- able Analysis
b. Header valves	2.2×10^{-6}	5.5×10^{-12}	2.0×10^{-7}	5.5×10^{-6}	1.0×10^{-6}	
2. Double events						
a. Pump trains	1.0	0.0				
b. EHG flow control valves	6.3×10^{-7}	1.5×10^{-11}	2.4×10^{-9}	2.0×10^{-6}	7.5×10^{-8}	
3. Human Error	2.2×10^{-3}	1.5×10^{-5}	9.7×10^{-5}	6.5×10^{-3}	1.0×10^{-1}	
4. Common Cause	*	-	-	-	-	
5. Other	*	-	-	-	-	
TOTAL	1.0	0.0	-	-	1.0	
B. Conventional Header						
1. Single events						
a. Piping	1.7×10^{-7}	1.6×10^{-12}	3.0×10^{-10}	4.3×10^{-7}	4.3×10^{-8}	
b. Header valves	7.1×10^{-5}	8.9×10^{-9}	9.0×10^{-6}	7.3×10^{-4}	4.0×10^{-5}	
2. Triple events						
a. Pump trains	1.7×10^{-2}	3.9×10^{-5}	9.0×10^{-3}	2.7×10^{-2}	1.6×10^{-2}	
3. Test and maintenance (with conventional pumps)	*	-	-	-	-	
4. Common Cause	*	-	-	-	-	
5. Other	*	-	-	-	-	
TOTAL	1.7×10^{-2}	3.9×10^{-5}	9.1×10^{-3}	2.7×10^{-2}	1.6×10^{-2}	
C. System						
1. Single events						
a. Piping	3.7×10^{-7}	8.0×10^{-12}	8.0×10^{-10}	1.0×10^{-6}	9.8×10^{-8}	
b. Header valves	7.3×10^{-5}	9.0×10^{-9}	1.0×10^{-5}	7.7×10^{-4}	4.7×10^{-5}	
2. Double events						
a. "Nuclear" Pumps	1.0	0.0				
b. EHG flow control valves	6.3×10^{-7}	1.5×10^{-11}	2.4×10^{-9}	2.0×10^{-6}	7.5×10^{-8}	
3. Triple events						
a. "Conventional" Pumps	1.7×10^{-2}	3.9×10^{-5}	9.0×10^{-3}	2.7×10^{-2}	1.6×10^{-2}	
4. Test and maintenance (with conventional pumps)	*	-	-	-	-	
5. Human Error	2.2×10^{-3}	1.5×10^{-5}	9.7×10^{-5}	6.5×10^{-3}	1.0×10^{-1}	
6. Common Cause	*	-	-	-	-	
7. Other	*	-	-	-	-	
SYSTEM TOTAL	1.0	0.0	-	-	1.0	

*Negligibly small failure probability.

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TABLE 1 (continued)

SUMMARY OF RESULTS - SERVICE WATER SYSTEM ANALYSIS

Case 2 - Loss of Offsite Power
Boundary Condition: Power Lost at Two 480V Switchgear Buses

	Mean	Variance	5th Percentile	95th Percentile	Median	MASH-140H
A. Nuclear Header						
1. Single events						
a. Piping	2.1×10^{-7}	2.5×10^{-12}	4.0×10^{-10}	6.0×10^{-7}	5.3×10^{-8}	No Compar- able Analysis
b. Header valves	7.1×10^{-5}	0.9×10^{-9}	9.0×10^{-6}	2.1×10^{-4}	4.0×10^{-5}	
2. Double events						
a. Pump Trains	1.0	0.0	-	-	-	
b. EDG flow control valves	6.3×10^{-7}	1.5×10^{-11}	2.4×10^{-9}	2.0×10^{-6}	7.5×10^{-8}	
3. Human Error	2.2×10^{-3}	1.5×10^{-5}	9.7×10^{-5}	6.5×10^{-3}	1.0×10^{-1}	
4. Common Cause	*	-	-	-	-	
5. Other	*	-	-	-	-	
TOTAL	1.0	0.0	-	-	1.0	
B. Conventional Header						
1. Single events						
a. Piping	1.7×10^{-7}	1.6×10^{-12}	3.0×10^{-10}	4.3×10^{-7}	4.3×10^{-8}	
b. Header valves	7.1×10^{-5}	0.9×10^{-9}	9.0×10^{-6}	2.3×10^{-4}	4.0×10^{-5}	
2. Triple events						
a. Pump Trains	1.7×10^{-7}	3.9×10^{-5}	9.0×10^{-3}	2.7×10^{-2}	1.6×10^{-7}	
3. Test and maintenance (with conventional pumps)	*	-	-	-	-	
4. Common Cause	*	-	-	-	-	
5. Other	*	-	-	-	-	
TOTAL	1.7×10^{-2}	3.9×10^{-5}	9.1×10^{-3}	2.7×10^{-2}	1.6×10^{-2}	
C. System						
1. Single events						
a. Piping	3.7×10^{-7}	8.0×10^{-12}	8.0×10^{-10}	1.0×10^{-6}	9.8×10^{-8}	
b. Header valves	1.4×10^{-4}	3.6×10^{-8}	1.0×10^{-5}	4.4×10^{-4}	7.9×10^{-5}	
2. Double events						
a. "Nuclear" Pumps	1.0	0.0	-	-	-	
b. EDG flow control valves	6.3×10^{-7}	1.5×10^{-11}	2.4×10^{-9}	2.0×10^{-6}	7.5×10^{-8}	
3. Triple events						
a. "Conventional" Pumps	1.7×10^{-2}	3.9×10^{-5}	9.0×10^{-3}	2.7×10^{-2}	1.6×10^{-2}	
4. Test and maintenance (with conventional pumps)	*	-	-	-	-	
5. Human Error	2.2×10^{-3}	1.5×10^{-5}	9.7×10^{-5}	6.5×10^{-3}	1.0×10^{-1}	
6. Common Cause	*	-	-	-	-	
7. Other	*	-	-	-	-	
SYSTEM TOTAL	1.0	0.0	-	-	1.0	

*Negligibly small failure probability.

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TABLE 2

FLOW REQUIREMENTS FOR THE SERVICE WATER SYSTEM

Essential header (Nuclear)	Design Flow Each (gpm)	Normal Operation	Injection Phase	Recirculation Phase
Containment Fan Cooler Units and Motor Coolers (5)	2,000	2,850	10,000 (5 of 5)	6,000 (3 of 5)
Diesel Generators (3)	400	Not Required	1,200 (3 of 3)	500 (2 of 3)
Instrument Air Compressor Cooling System (2)	65	65	65	65
Strainer Blowdown (3)	100	100	300	200
Component Cooling Heat Exchangers (2)	3,500	Not Supplied	Not Required	3,500 (1 of 2)
Radiation Shield Mixing Nozzle	80	80	80	80
Other Miscellaneous	610	1,480	<u>1,480</u>	<u>40</u>
Total			15,045	10,685

NonEssential header (Conventionally)	Design Flow Each (gpm)	Normal Operation	Injection Phase	Recirculation Phase
Diesel Generator	400	Not Required	Not Required	400 (1 of 3)
Component Cooling Heat Exchangers (2)	3,500	7,000 (2 of 2)	Not Required	3,500 (1 of 2)
Strainer Blowdown (3)	100	200	Not Required	100
Other Miscellaneous	4,690	<u>4,650</u>	<u>0</u>	<u>40</u>
Total		11,850	0	4,040

() Number of like components cooled.

TABLE 3

ELECTRICAL SUPPLY BUSES FOR
INDIAN POINT 3 SERVICE WATER PUMPS

Pump Number	480V Bus
31	5A
32	2A
33	6A
34	5A
35	3A
36	6A
37	5A
38	3A
39	6A

TABLE 4

BASIC EVENT DATA - INDIAN POINT 3 SERVICE WATER SYSTEM

Event Description and Failure Mode	Fault Tree Coding	Failure Data			Common Cause Data			Comments
		Mean	W/D	Variance	MTH	Ref. (1)	Location	
No power at switchgear bus 2A	JBS-32AD	-	-	-	-	-	See EP Analysis	See EP analysis
No power at switchgear bus 3A	JBS-33AD	-	-	-	-	-	-	See EP analysis
No power at switchgear bus 5A	JBS-35AD	-	-	-	-	-	-	See EP analysis
No power at switchgear bus 6A	JBS-36AD	-	-	-	-	-	-	See EP analysis
No DC control power at switchgear bus 2A	4BS-333D	-	-	-	-	-	-	See EP analysis
No DC control power at switchgear bus 3A	4BS-333D	-	-	-	-	-	-	See EP analysis
No DC control power at switchgear bus 5A	4BS-333D	-	-	-	-	-	-	See EP analysis
No DC control power at switchgear bus 6A	4BS-332D	-	-	-	-	-	-	See EP analysis
SW pump no. 31 does not start does not continue to run	IPMP31-S	1.36 x 10 ⁻³ 4.68 x 10 ⁻⁵	0 11	1.22 x 10 ⁻⁶ 1.07 x 10 ⁻⁹	- see text	11 17	Intake well	I, V
SW pump no. 32 does not start does not continue to run	IPMP32-S	1.36 x 10 ⁻³ 4.68 x 10 ⁻⁵	0 11	1.22 x 10 ⁻⁶ 1.07 x 10 ⁻⁹	- see text	11 17	-	I, V
SW pump no. 33 does not start does not continue to run	IPMP33-S	1.36 x 10 ⁻³ 4.68 x 10 ⁻⁵	0 11	1.22 x 10 ⁻⁶ 1.07 x 10 ⁻⁹	- see text	11 17	-	I, V
SW pump no. 34 does not start does not continue to run	IPMP34-S	1.36 x 10 ⁻³ 4.68 x 10 ⁻⁵	0 11	1.22 x 10 ⁻⁶ 1.07 x 10 ⁻⁹	- see text	11 17	-	I, V

(1) Reference - refers to item numbers in the plant failure data section of this report.

TABLE 4 (continued)

BASIC EVENT DATA - INDIAN POINT 3 SERVICE WATER SYSTEM

Event Description and Failure Mode	Fault Tree Coding	Failure Data					Common Cause Data		Comments
		Mean	H/D	Variance	MTTR	Ref. (1)	Location	Susceptibility	
SW pump no. 35 does not start does not continue to run	TPMP35-S	1.36×10^{-3} 4.60×10^{-5}	0 H	1.22×10^{-6} 1.07×10^{-9}	- see text	11 17	"	I, V	
SW pump no. 36 does not start does not continue to run	TPMP36-S	1.36×10^{-3} 4.60×10^{-5}	0 H	1.22×10^{-6} 1.07×10^{-9}	- see text	11 17	"	I, V	
SW pump no. 31 motor does not start/ run	TMOP31-S	-	-	-	-	-	Intake structure	I, V, M	with pump
SW pump no. 32 motor does not start/ run	TMOP32-S	-	-	-	-	-	"	"	with pump
SW pump no. 33 motor does not start/ run	TMOP33-S	-	-	-	-	-	"	"	with pump
SW pump no. 34 motor does not start/ run	TMOP34-S	-	-	-	-	-	"	"	with pump
SW pump no. 35 motor does not start/ run	TMOP35-S	-	-	-	-	-	"	"	with pump
SW pump no. 36 motor does not start/ run	TMOP36-S	-	-	-	-	-	"	"	with pump
SW pump no. 31 control circuit fails	TCCP31-F	-	-	-	-	-	Cont. Rm/480V Swgr	"	with pump
SW pump no. 32 control circuit fails	TCCP32-F	-	-	-	-	-	"	"	with pump
SW pump no. 33 control circuit fails	TCCP33-F	-	-	-	-	-	"	"	with pump
SW pump no. 34 control circuit fails	TCCP34-F	-	-	-	-	-	"	"	with pump
SW pump no. 35 control circuit fails	TCCP35-F	-	-	-	-	-	"	"	with pump
SW pump no. 36 control circuit fails	TCCP36-F	-	-	-	-	-	"	"	with pump
Expansion joint SWN-3-1 ruptures	TXJWS31L	0.60×10^{-9}	H	6.00×10^{-15}	see text	47	Intake well	I, V	
Expansion joint SWN-3-2 ruptures	TXJWS32L	0.60×10^{-9}	H	6.00×10^{-15}	see text	47	"	"	

(1) Reference - Refers to item numbers in the plant failure data section of this report.

TABLE 4 (continued)

BASIC EVENT DATA - INDIAN POINT 3 SERVICE WATER SYSTEM

Event Description and Failure Mode	Fault Tree Coding	Failure Data				Common Cause Data			Comments
		Mean	H/D	Variance	MIR	Ref. (1)	Location	Susceptibility	
Expansion joint SWN-3-3 ruptures	TXJSM33	8.60×10^{-9}	H	6.00×10^{-15}	see text	47	"	"	
Expansion joint SWN-3-4 ruptures	TXJSM34L	8.60×10^{-9}	H	6.00×10^{-15}	see text	47	"	"	
Expansion joint SWN-3-5 ruptures	TXJSM35L	8.60×10^{-9}	H	6.00×10^{-15}	see text	47	"	"	
Expansion joint SWN-3-6 ruptures	TXJSM36L	8.60×10^{-9}	H	6.00×10^{-15}	see text	47	"	"	
Check valve SWN-1-1 fails to open	TCVSW11Q	6.91×10^{-5}	D	1.03×10^{-8}	-	3	Strainer enclosure	1	
Check valve SWN-1-2 fails to open	TCVSW12Q	6.91×10^{-5}	D	1.03×10^{-8}	-	1	"	"	
Check valve SWN-1-3 fails to open	TCVSW13Q	6.91×10^{-5}	D	1.03×10^{-8}	-	1	"	"	
Check valve SWN-1-4 fails to open	TCVSW14Q	6.91×10^{-5}	D	1.03×10^{-8}	-	3	"	"	
Check valve SWN-1-5 fails to open	TCVSW15Q	6.91×10^{-5}	D	1.03×10^{-8}	-	3	"	"	
Check valve SWN-1-6 fails to open	TCVSW16Q	6.91×10^{-5}	D	1.03×10^{-8}	-	3	"	"	
Discharge valve SWN-2-1 transfers closed	TXVSW21C	9.15×10^{-8}	H	1.01×10^{-14}	see text	1	"	"	
Discharge valve SWN-2-2 transfers closed	TXVSW22C	9.15×10^{-8}	H	1.01×10^{-14}	see text	1	"	"	
Discharge valve SWN-2-3 transfers closed	TXVSW23C	9.15×10^{-8}	H	1.01×10^{-14}	see text	1	"	"	
Discharge valve SWN-2-4 transfers closed	TXVSW24C	9.15×10^{-8}	H	1.01×10^{-14}	see text	1	"	"	
Discharge valve SWN-2-5 transfers closed	TXVSW25C	9.15×10^{-8}	H	1.01×10^{-14}	see text	1	"	"	
Discharge valve SWN-2-6 transfers closed	TXVSW26C	9.15×10^{-8}	H	1.01×10^{-14}	see text	1	"	"	

(1) Reference - Refers to item numbers in the plant failure data section of this report.

TABLE 4 (continued)

BASIC EVENT DATA - INDIAN POINT 3 SERVICE WATER SYSTEM

Event Description and Failure Mode	Fault Tree Coding	Failure Data			Common Cause Data			Comments
		Mean	H/D	Variance	MTTR	Ref. (1)	Location	Susceptibility
Discharge strainer SWH-4-1 plugged	IFLSW41G	(2)	-	-	-	-	Strainer enclosure	I
Discharge strainer SWH-4-2 plugged	IFLSW42G	(2)	-	-	-	-	-	-
Discharge strainer SWH-4-3 plugged	IFLSW43G	(2)	-	-	-	-	-	-
Discharge strainer SWH-4-4 plugged	IFLSW44G	(2)	-	-	-	-	-	-
Discharge strainer SWH-4-5 plugged	IFLSW45G	(2)	-	-	-	-	-	-
Discharge strainer SWH-4-6 plugged	IFLSW46G	(2)	-	-	-	-	-	-
FCV 1176 does not open	IFV576-Q	4.98 x 10 ⁻⁴	0	4.03 x 10 ⁻⁷	-	0	DG Building	I, V
FCV 1176A does not open	IFV576AQ	4.98 x 10 ⁻⁴	0	4.03 x 10 ⁻⁷	-	0	DG Building	I, V
SW discharge valve SWH98 transfers closed	IXVSW98C	9.15 x 10 ⁻⁸	0	1.01 x 10 ⁻¹⁴	see text	1	Valve pit	I
SW discharge valve SWH99 transfers closed	IXVSW99C	9.15 x 10 ⁻⁸	0	1.01 x 10 ⁻¹⁴	see text	1	Valve pit	I
SW discharge check valve SWH-100A fails closed	ICV100AQ	6.91 x 10 ⁻⁵	0	1.03 x 10 ⁻⁸	-	3	Valve pit	I
SW discharge check valve SWH-100B fails closed	ICV100BQ	6.91 x 10 ⁻⁵	0	1.03 x 10 ⁻⁸	-	3	Valve pit	I
SW piping nuclear fails	IPP1FAIL	8.60 x 10 ⁻¹⁰	0	6.00 x 10 ⁻¹⁷	see text	40	Various	I
SW piping conventional fails	IPP2FAIL	8.60 x 10 ⁻¹⁰	0	6.00 x 10 ⁻¹⁷	see text	40	Various	I

(1) Reference - refers to item numbers in the plant failure data section of this report.

(2) Negligibly small failure probability.

TABLE 5 (continued)

TITLE: SCALC WATER SYSTEM INDIAN POOL J COSETS

SERVICE WATER SYSTEM INDIAN POINT 3 CUTSETS
(Sheet 3 of 7)

(Sheet 3 of 1)

CONSTITUTIONAL EVENTS.

[illegible]

SERVICE WATER SYSTEM INDIAN POINT 3 CUTSETS

(Sheet 4 of 7)

CUT SETS WITH 3 BASIC EVENTS.

8101	1791	ICCP11-F	PMPP12-S	PMPP13-S	1791	PMPP11-S	PMPP12-S	PMPP13-S	1801	PMPP11-S	PMPP12-S	PMPP13-S
8111	1811	ICCP11-F	PMPP12-S	PMPP13-S	1821	PMPP11-S	ICCP12-F	PMPP13-S	1831	PMPP11-S	ICCP12-F	PMPP13-S
8161	1861	ICCP11-F	PMPP12-S	PMPP13-S	1871	PMPP11-S	JMS-12AD	ICCP13-F	1881	PMPP11-S	JMS-12AD	ICCP13-F
8191	1891	ICCP11-F	JMS-12AD	ICCP13-F	1901	PMPP11-S	PMPP12-S	ICCP13-F	1911	PMPP11-S	PMPP12-S	ICCP13-F
8261	1901	ICCP11-F	PMPP12-S	ICCP13-F	1911	PMPP11-S	PMPP12-S	ICCP13-F	1921	PMPP11-S	PMPP12-S	ICCP13-F
8451	1941	ICCP11-F	PMPP12-S	ICCP13-F	1951	PMPP11-S	ICCP12-F	ICCP13-F	1961	PMPP11-S	ICCP12-F	ICCP13-F
8481	1961	ICCP11-F	ICCP12-F	ICCP13-F	1971	ICV5W12G	PMPP11-S	PMPP13-S	1981	ICV5W12G	PMPP11-S	PMPP13-S
8511	1991	ICV5W12G	PMPP11-S	PMPP13-S	2001	ICV5W12G	PMPP11-S	PMPP13-S	2011	ICV5W12G	PMPP11-S	PMPP13-S
8541	2021	ICV5W12G	PMPP11-S	PMPP13-S	2031	ICV5W12G	PMPP11-S	PMPP13-S	2041	ICV5W12G	PMPP11-S	PMPP13-S
8571	2051	ICV5W12G	ICCP11-F	PMPP13-S	2061	ICV5W12G	ICCP11-F	PMPP13-S	2071	ICV5W12G	ICCP11-F	PMPP13-S
8601	2081	ICV5W12G	ICCP11-F	PMPP13-S	2091	ICV5W12G	ICCP11-F	PMPP13-S	2101	ICV5W12G	ICCP11-F	PMPP13-S
8631	2111	ICV5W12G	PMPP11-S	PMPP13-S	2121	ICV5W12G	PMPP11-S	PMPP13-S	2131	ICV5W12G	PMPP11-S	PMPP13-S
8661	2141	ICV5W12G	PMPP11-S	PMPP13-S	2151	ICV5W12G	PMPP11-S	PMPP13-S	2161	ICV5W12G	PMPP11-S	PMPP13-S
8691	2171	ICV5W12G	ICCP11-F	PMPP13-S	2181	ICV5W12G	ICCP11-F	PMPP13-S	2191	ICV5W12G	ICCP11-F	PMPP13-S
8721	2201	ICV5W12G	ICCP11-F	PMPP13-S	2211	ICV5W12G	ICCP11-F	PMPP13-S	2221	ICV5W12G	ICCP11-F	PMPP13-S
8751	2231	ICV5W12G	PMPP11-S	PMPP13-S	2241	ICV5W12G	PMPP11-S	PMPP13-S	2251	ICV5W12G	PMPP11-S	PMPP13-S
8781	2261	ICV5W12G	PMPP11-S	PMPP13-S	2271	ICV5W12G	PMPP11-S	PMPP13-S	2281	ICV5W12G	PMPP11-S	PMPP13-S
8811	2291	ICV5W12G	ICCP11-F	PMPP13-S	2301	ICV5W12G	ICCP11-F	PMPP13-S	2311	ICV5W12G	ICCP11-F	PMPP13-S
8841	2321	ICV5W12G	ICCP11-F	PMPP13-S	2331	ICV5W12G	ICCP11-F	PMPP13-S	2341	ICV5W12G	ICCP11-F	PMPP13-S
8871	2351	ICV5W12G	ICCP11-F	PMPP13-S	2361	ICV5W12G	ICCP11-F	PMPP13-S	2371	ICV5W12G	ICCP11-F	PMPP13-S
8901	2381	ICV5W12G	ICCP11-F	PMPP13-S	2391	ICV5W12G	ICCP11-F	PMPP13-S	2401	ICV5W12G	ICCP11-F	PMPP13-S
8931	2411	ICV5W12G	ICCP11-F	PMPP13-S	2421	ICV5W12G	ICCP11-F	PMPP13-S	2431	ICV5W12G	ICCP11-F	PMPP13-S
8961	2441	ICV5W12G	ICCP11-F	PMPP13-S	2451	ICV5W12G	ICCP11-F	PMPP13-S	2461	ICV5W12G	ICCP11-F	PMPP13-S
8991	2471	ICV5W12G	ICCP11-F	PMPP13-S	2481	ICV5W12G	ICCP11-F	PMPP13-S	2491	ICV5W12G	ICCP11-F	PMPP13-S
9021	2501	ICV5W12G	ICCP11-F	PMPP13-S	2511	ICV5W12G	ICCP11-F	PMPP13-S	2521	ICV5W12G	ICCP11-F	PMPP13-S
9051	2531	ICV5W12G	ICCP11-F	PMPP13-S	2541	ICV5W12G	ICCP11-F	PMPP13-S	2551	ICV5W12G	ICCP11-F	PMPP13-S
9081	2561	ICV5W12G	ICCP11-F	PMPP13-S	2571	ICV5W12G	ICCP11-F	PMPP13-S	2581	ICV5W12G	ICCP11-F	PMPP13-S
9111	2591	ICV5W12G	ICCP11-F	PMPP13-S	2601	ICV5W12G	ICCP11-F	PMPP13-S	2611	ICV5W12G	ICCP11-F	PMPP13-S
9141	2621	ICV5W12G	ICCP11-F	PMPP13-S	2631	ICV5W12G	ICCP11-F	PMPP13-S	2641	ICV5W12G	ICCP11-F	PMPP13-S
9171	2651	ICV5W12G	ICCP11-F	PMPP13-S	2661	ICV5W12G	ICCP11-F	PMPP13-S	2671	ICV5W12G	ICCP11-F	PMPP13-S
9201	2681	ICV5W12G	ICCP11-F	PMPP13-S	2691	ICV5W12G	ICCP11-F	PMPP13-S	2701	ICV5W12G	ICCP11-F	PMPP13-S
9231	2711	ICV5W12G	ICCP11-F	PMPP13-S	2721	ICV5W12G	ICCP11-F	PMPP13-S	2731	ICV5W12G	ICCP11-F	PMPP13-S
9261	2741	ICV5W12G	ICCP11-F	PMPP13-S	2751	ICV5W12G	ICCP11-F	PMPP13-S	2761	ICV5W12G	ICCP11-F	PMPP13-S
9291	2771	ICV5W12G	ICCP11-F	PMPP13-S	2781	ICV5W12G	ICCP11-F	PMPP13-S	2791	ICV5W12G	ICCP11-F	PMPP13-S
9321	2801	ICV5W12G	ICCP11-F	PMPP13-S	2811	ICV5W12G	ICCP11-F	PMPP13-S	2821	ICV5W12G	ICCP11-F	PMPP13-S
9351	2831	ICV5W12G	ICCP11-F	PMPP13-S	2841	ICV5W12G	ICCP11-F	PMPP13-S	2851	ICV5W12G	ICCP11-F	PMPP13-S
9381	2861	ICV5W12G	ICCP11-F	PMPP13-S	2871	ICV5W12G	ICCP11-F	PMPP13-S	2881	ICV5W12G	ICCP11-F	PMPP13-S
9411	2891	ICV5W12G	ICCP11-F	PMPP13-S	2901	ICV5W12G	ICCP11-F	PMPP13-S	2911	ICV5W12G	ICCP11-F	PMPP13-S
9441	2921	ICV5W12G	ICCP11-F	PMPP13-S	2931	ICV5W12G	ICCP11-F	PMPP13-S	2941	ICV5W12G	ICCP11-F	PMPP13-S
9471	2951	ICV5W12G	ICCP11-F	PMPP13-S	2961	ICV5W12G	ICCP11-F	PMPP13-S	2971	ICV5W12G	ICCP11-F	PMPP13-S
9501	2981	ICV5W12G	ICCP11-F	PMPP13-S	2991	ICV5W12G	ICCP11-F	PMPP13-S	3001	ICV5W12G	ICCP11-F	PMPP13-S
9531	3011	ICV5W12G	ICCP11-F	PMPP13-S	3021	ICV5W12G	ICCP11-F	PMPP13-S	3031	ICV5W12G	ICCP11-F	PMPP13-S
9561	3041	ICV5W12G	ICCP11-F	PMPP13-S	3051	ICV5W12G	ICCP11-F	PMPP13-S	3061	ICV5W12G	ICCP11-F	PMPP13-S
9591	3071	ICV5W12G	ICCP11-F	PMPP13-S	3081	ICV5W12G	ICCP11-F	PMPP13-S	3091	ICV5W12G	ICCP11-F	PMPP13-S
9621	3101	ICV5W12G	ICCP11-F	PMPP13-S	3111	ICV5W12G	ICCP11-F	PMPP13-S	3121	ICV5W12G	ICCP11-F	PMPP13-S
9651	3131	ICV5W12G	ICCP11-F	PMPP13-S	3141	ICV5W12G	ICCP11-F	PMPP13-S	3151	ICV5W12G	ICCP11-F	PMPP13-S
9681	3161	ICV5W12G	ICCP11-F	PMPP13-S	3171	ICV5W12G	ICCP11-F	PMPP13-S	3181	ICV5W12G	ICCP11-F	PMPP13-S
9711	3191	ICV5W12G	ICCP11-F	PMPP13-S	3201	ICV5W12G	ICCP11-F	PMPP13-S	3211	ICV5W12G	ICCP11-F	PMPP13-S
9741	3221	ICV5W12G	ICCP11-F	PMPP13-S	3231	ICV5W12G	ICCP11-F	PMPP13-S	3241	ICV5W12G	ICCP11-F	PMPP13-S
9771	3251	ICV5W12G	ICCP11-F	PMPP13-S	3261	ICV5W12G	ICCP11-F	PMPP13-S	3271	ICV5W12G	ICCP11-F	PMPP13-S
9801	3281	ICV5W12G	ICCP11-F	PMPP13-S	3291	ICV5W12G	ICCP11-F	PMPP13-S	3301	ICV5W12G	ICCP11-F	PMPP13-S
9831	3311	ICV5W12G	ICCP11-F	PMPP13-S	3321	ICV5W12G	ICCP11-F	PMPP13-S	3331	ICV5W12G	ICCP11-F	PMPP13-S
9861	3341	ICV5W12G	ICCP11-F	PMPP13-S	3351	ICV5W12G	ICCP11-F	PMPP13-S	3361	ICV5W12G	ICCP11-F	PMPP13-S
9891	3371	ICV5W12G	ICCP11-F	PMPP13-S	3381	ICV5W12G	ICCP11-F	PMPP13-S	3391	ICV5W12G	ICCP11-F	PMPP13-S
9921	3401	ICV5W12G	ICCP11-F	PMPP13-S	3411	ICV5W12G	ICCP11-F	PMPP13-S	3421	ICV5W12G	ICCP11-F	PMPP13-S
9951	3431	ICV5W12G	ICCP11-F	PMPP13-S	3441	ICV5W12G	ICCP11-F	PMPP13-S	3451	ICV5W12G	ICCP11-F	PMPP13-S
9981	3461	ICV5W12G	ICCP11-F	PMPP13-S	3471	ICV5W12G	ICCP11-F	PMPP13-S	3481	ICV5W12G	ICCP11-F	PMPP13-S
9991	3491	ICV5W12G	ICCP11-F	PMPP13-S	3501	ICV5W12G	ICCP11-F	PMPP13-S	3511	ICV5W12G	ICCP11-F	PMPP13-S

TITLE: SERVICE WATER SYSTEM INDIAN POINT 3 CUTSETS

(Sheet 5 of 7)

CUT SETS WITH 1 BASIC EVENTS.

CUT SETS	1491	1492	1493	1494	1495	1496	1497	1498	1499	1500	1501	1502	1503	1504	1505	1506	1507	1508	1509	1510	1511	1512	1513	1514	1515	1516	1517	1518	1519	1520	1521	1522	1523	1524	1525	1526	1527	1528	1529	1530	1531	1532	1533	1534	1535	1536	1537	1538	1539	1540	1541	1542	1543	1544	1545	1546	1547	1548	1549	1550	1551	1552	1553	1554	1555	1556	1557	1558	1559	1560	1561	1562	1563	1564	1565	1566	1567	1568	1569	1570	1571	1572	1573	1574	1575	1576	1577	1578	1579	1580	1581	1582	1583	1584	1585	1586	1587	1588	1589	1590	1591	1592	1593	1594	1595	1596	1597	1598	1599	1600	1601	1602	1603	1604	1605	1606	1607	1608	1609	1610	1611	1612	1613	1614	1615	1616	1617	1618	1619	1620	1621	1622	1623	1624	1625	1626	1627	1628	1629	1630	1631	1632	1633	1634	1635	1636	1637	1638	1639	1640	1641	1642	1643	1644	1645	1646	1647	1648	1649	1650	1651	1652	1653	1654	1655	1656	1657	1658	1659	1660	1661	1662	1663	1664	1665	1666	1667	1668	1669	1670	1671	1672	1673	1674	1675	1676	1677	1678	1679	1680	1681	1682	1683	1684	1685	1686	1687	1688	1689	1690	1691	1692	1693	1694	1695	1696	1697	1698	1699	1700	1701	1702	1703	1704	1705	1706	1707	1708	1709	1710	1711	1712	1713	1714	1715	1716	1717	1718	1719	1720	1721	1722	1723	1724	1725	1726	1727	1728	1729	1730	1731	1732	1733	1734	1735	1736	1737	1738	1739	1740	1741	1742	1743	1744	1745	1746	1747	1748	1749	1750	1751	1752	1753	1754	1755	1756	1757	1758	1759	1760	1761	1762	1763	1764	1765	1766	1767	1768	1769	1770	1771	1772	1773	1774	1775	1776	1777	1778	1779	1780	1781	1782	1783	1784	1785	1786	1787	1788	1789	1790	1791	1792	1793	1794	1795	1796	1797	1798	1799	1800	1801	1802	1803	1804	1805	1806	1807	1808	1809	1810	1811	1812	1813	1814	1815	1816	1817	1818	1819	1820	1821	1822	1823	1824	1825	1826	1827	1828	1829	1830	1831	1832	1833	1834	1835	1836	1837	1838	1839	1840	1841	1842	1843	1844	1845	1846	1847	1848	1849	1850	1851	1852	1853	1854	1855	1856	1857	1858	1859	1860	1861	1862	1863	1864	1865	1866	1867	1868	1869	1870	1871	1872	1873	1874	1875	1876	1877	1878	1879	1880	1881	1882	1883	1884	1885	1886	1887	1888	1889	1890	1891	1892	1893	1894	1895	1896	1897	1898	1899	1900	1901	1902	1903	1904	1905	1906	1907	1908	1909	1910	1911	1912	1913	1914	1915	1916	1917	1918	1919	1920	1921	1922	1923	1924	1925	1926	1927	1928	1929	1930	1931	1932	1933	1934	1935	1936	1937	1938	1939	1940	1941	1942	1943	1944	1945	1946	1947	1948	1949	1950	1951	1952	1953	1954	1955	1956	1957	1958	1959	1960	1961	1962	1963	1964	1965	1966	1967	1968	1969	1970	1971	1972	1973	1974	1975	1976	1977	1978	1979	1980	1981	1982	1983	1984	1985	1986	1987	1988	1989	1990	1991	1992	1993	1994	1995	1996	1997	1998	1999	2000	2001	2002	2003	2004	2005	2006	2007	2008	2009	2010	2011	2012	2013	2014	2015	2016	2017	2018	2019	2020	2021	2022	2023	2024	2025	2026	2027	2028	2029	2030	2031	2032	2033	2034	2035	2036	2037	2038	2039	2040	2041	2042	2043	2044	2045	2046	2047	2048	2049	2050	2051	2052	2053	2054	2055	2056	2057	2058	2059	2060	2061	2062	2063	2064	2065	2066	2067	2068	2069	2070	2071	2072	2073	2074	2075	2076	2077	2078	2079	2080	2081	2082	2083	2084	2085	2086	2087	2088	2089	2090	2091	2092	2093	2094	2095	2096	2097	2098	2099	2100	2101	2102	2103	2104	2105	2106	2107	2108	2109	2110	2111	2112	2113	2114	2115	2116	2117	2118	2119	2120	2121	2122	2123	2124	2125	2126	2127	2128	2129	2130	2131	2132	2133	2134	2135	2136	2137	2138	2139	2140	2141	2142	2143	2144	2145	2146	2147	2148	2149	2150	2151	2152	2153	2154	2155	2156	2157	2158	2159	2160	2161	2162	2163	2164	2165	2166	2167	2168	2169	2170	2171	2172	2173	2174	2175	2176	2177	2178	2179	2180	2181	2182	2183	2184	2185	2186	2187	2188	2189	2190	2191	2192	2193	2194	2195	2196	2197	2198	2199	2200	2201	2202	2203	2204	2205	2206	2207	2208	2209	2210	2211	2212	2213	2214	2215	2216	2217	2218	2219	2220	2221	2222	2223	2224	2225	2226	2227	2228	2229	2230	2231	2232	2233	2234	2235	2236	2237	2238	2239	2240	2241	2242	2243	2244	2245	2246	2247	2248	2249	2250	2251	2252	2253	2254	2255	2256	2257	2258	2259	2260	2261	2262	2263	2264	2265	2266	2267	2268	2269	2270	2271	2272	2273	2274	2275	2276	2277	2278	2279	2280	2281	2282	2283	2284	2285	2286	2287	2288	2289	2290	2291	2292	2293	2294	2295	2296	2297	2298	2299	2300	2301	2302	2303	2304	2305	2306	2307	2308	2309	2310	2311	2312	2313	2314	2315	2316	2317	2318	2319	2320	2321	2322	2323	2324	2325	2326	2327	2328	2329	2330	2331	2332	2333	2334	2335	2336	2337	2338	2339	2340	2341	2342	2343	2344	2345	2346	2347	2348	2349	2350	2351	2352	2353	2354	2355	2356	2357	2358	2359	2360	2361	2362	2363	2364	2365	2366	2367	2368	2369	2370	2371	2372	2373	2374	2375	2376	2377	2378	2379	2380	2381	2382	2383	2384	2385	2386	2387	2388	2389	2390	2391	2392	2393	2394	2395	2396	2397	2398	2399	2400	2401	2402	2403	2404	2405	2406	2407	2408	2409	2410	2411	2412	2413	2414	2415	2416	2417	2418	2419	2420	2421	2422	2423	2424	2425	2426	2427	2428	2429	2430	2431	2432	2433	2434	2435	2436	2437	2438	2439	2440	2441	2442	2443	2444	2445	2446	2447	2448	2449	2450	2451	2452	2453	2454	2455	2456	2457	2458	2459	2460	2461	2462	2463	2464	2465	2466	2467	2468	2469	2470	2471	2472	2473	2474	2475	2476	2477	2478	2479	2480	2481	2482	2483	2484	2485	2486	2487	2488	2489	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TABLE 5 (continued)

SERVICE WATER SYSTEM INDIAN POINT 3 CUTSETS
(Sheet 7 of 7)

(Sheet 7 of 1)

[illegible]

TABLE 6

SYSTEM EFFECTS OF PIPE FAILURE

Pipe Section Nuclear Heater	Dis. (inches)	System Failure	Potential for Other Systems Impact	Comments
1. #402				
a. Pumps 12, 13, 14 pressure heater (to 12-13-14)	24	Yes	Loss of cooling to contain- ment building for cooling units (CBFU) and emergency diesel generators (EDG).	Operator can isolate and shift services to an intact heater. (Time required approximately one hour.)
b. Pumps 12, 13, 14 pressure heater	24	Yes	Same as 1a.	Same as 1a.
c. Pumps 12, 13, 14 pressure heater	24	Yes	Same as 1a.	Operator can isolate Sw-12 and recover EDG. (T = 15 min.)
d. Pumps 12, 13, 14 pressure heater	24	Yes	Same as 1a.	Operator can isolate Sw-12 or 40-1 and recover 3 of 3 CBFU and EDG (T = 15 min.)
e. Pumps 12, 13, 14 pressure heater	24	Yes	Same as 1a.	Operator can isolate Sw-12 and Sw-13 or 40-1 and recover 3 of 3 CBFU and EDG. (T = 15 min.)
f. Pumps 12, 13, 14 pressure heater	24	Yes	Same as 1a.	Same as 1a.
2. #401				
a. Supply to containment cooling, CDS, heat exchanger (before Sw-12)	20	Yes	Same as 1a.	Same as 1a.
3. #403				
a. Supply to CDS (after Sw-12)	10	Yes	Same as 1a.	Same as 1a.
4. #405				
a. Supply to Turbine 11 cooling, CDS, heat (before Sw-12)	10	Yes	Same as 1a.	Same as 1a.
5. #409				
a. Supply to CDFU (after Sw-12)	10	Yes	Same as 1a.	Same as 1a.

TABLE 6 (continued)

SYSTEM EFFECTS OF PIPE FAILURE

Pipe Section Nuclear Header	Dia. (inches)	System Failure	Potential for Other Systems Input	Comments
6. #406				
a. Pumps 31, 32, 33 discharge header (before Swt-33)	24	Yes	Loss of cooling to CCS heat exchangers	Same as 1a.
b. Discharge header (after Swt-33)	24	Yes	Same as 1a.	Same as 1a.
c. Supply to various conventional plant services before SW-1110	16	Yes	Same as 6a.	Same as 1a.
7. #407				
a. Supply to CCS heat exchanger (after Swt-31)	18	Yes	Loss of cooling to CCS heat exchangers.	Operator can isolate and restore cooling to one CCS heat exchanger (T ≈ 15 min.)
8. #407				
a. Supply to CCS heat exchanger (before Swt-31)	0	Yes	Same as 6a.	Same as 1a.
8. #407 (continued)				
b. Supply to CCS heat exchanger (after Swt-31) (includes 18" - 407)	18	Yes	Same as 6a.	Same as 7a.
9. #1093				
a. Supply to CCS (after Swt-30)	10	Yes	Same as 6a.	Operator can isolate at Swt-30. (T ≈ 15 min.)
10. #409				
a. Supply to turbine oil coolers etc. (before Swt-7)	10	Yes	Same as 6a.	Same as 1a.

TABLE 7

INDIAN POINT 3 SERVICE WATER SYSTEM - CAUSE TABLE

Nuclear Header
Case 1 Boundary Condition: Power at All 480V Switchgear Buses

Component	Failure Data			(Demand)			Failure Data (t = 24 hours)	
	(Hourly)			(Demand)			Mean	Variance
A. Single Events	Mean	Variance	Mean	Variance	Mean	Variance		
1. Piping failures (10)	8.60 x 10 ⁻⁹	4.29 x 10 ⁻¹⁵	-	-	-	-	2.06 x 10 ⁻⁷	2.47 x 10 ⁻¹²
2. Header isolation valves	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴	-	-	-	-	2.20 x 10 ⁻⁶	5.51 x 10 ⁻¹²
B. Double Events								
1. Pump trains								
a. Pump fails to start	4.68 x 10 ⁻⁵	1.07 x 10 ⁻⁹	1.36 x 10 ⁻³	1.22 x 10 ⁻⁶	-	-	-	-
b. Pump fails to run	8.60 x 10 ⁻⁹	6.00 x 10 ⁻¹⁵	-	-	-	-	-	-
c. Expansion joint ruptures	-	-	6.91 x 10 ⁻⁵	1.03 x 10 ⁻⁸	-	-	-	-
d. Check valve fails to open	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴	-	-	-	-	-	-
e. Discharge valve fails	4.69 x 10 ⁻⁵	9.54 x 10 ⁻¹⁰	1.43 x 10 ⁻³	1.17 x 10 ⁻⁶	-	-	-	-
Single train total	-	-	-	-	-	-	7.59 x 10 ⁻⁶	5.50 x 10 ⁻¹¹
2 of 3 pump trains	-	-	-	-	-	-	-	-
2. CBB flow control valve	-	-	4.98 x 10 ⁻⁴	4.03 x 10 ⁻⁷	-	-	6.27 x 10 ⁻⁷	1.54 x 10 ⁻¹¹
a. valve fails	-	-	-	-	-	-	-	-
2 of 2 valves	-	-	-	-	-	-	-	-
C. Human Error	-	-	-	-	-	-	-	-
1. Mispositioned switch	-	-	-	-	-	-	2.16 x 10 ⁻³	1.46 x 10 ⁻⁵
D. Common Cause	-	-	-	-	-	-	-	-
E. Other	-	-	-	-	-	-	-	-
F. Header Total	-	-	-	-	-	-	2.17 x 10 ⁻³	1.41 x 10 ⁻⁵

*Negligibly small failure probability.

0626A040/011/1

TABLE 7 (continued)

INDIAN POINT 3 SERVICE WATER SYSTEM - CAUSE TABLE

Nuclear Header
Case 2 Boundary Condition: Power at All 400V Switchgear Buses

Component	Failure Data			System Effect		
	(Hourly)		(Demand)	Failure Data (t = 24 hours)		
	Mean	Variance	Mean	Variance	Mean	Variance
A. Single Events						
1. Piping failures (10)	8.60 x 10 ⁻⁹	4.29 x 10 ⁻¹⁵				
2. Header Isolation valves	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴	6.91 x 10 ⁻⁵	1.03 x 10 ⁻⁸	2.06 x 10 ⁻⁷	2.47 x 10 ⁻¹²
B. Double Events						
1. Pump Trains						
a. Pump fails to start			1.36 x 10 ⁻³	1.22 x 10 ⁻⁶		
b. Pump fails to run	4.68 x 10 ⁻⁵	1.07 x 10 ⁻⁹				
c. Expansion joint ruptures	8.60 x 10 ⁻⁹	6.00 x 10 ⁻¹⁵				
d. Check valve fails to open			6.91 x 10 ⁻⁵	1.03 x 10 ⁻⁸		
e. Discharge valve fails	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴				
Single train total	4.69 x 10 ⁻⁵	9.54 x 10 ⁻¹⁰	1.43 x 10 ⁻³	1.12 x 10 ⁻⁶		
2. EDG Flow control valve						
a. valve fails			4.98 x 10 ⁻⁴	4.03 x 10 ⁻⁷		
2 of 2 valves						
C. Human Error						
1. Mispositioned switch						
D. Common Cause						
E. Other						
F. Header Total						
					2.10 x 10 ⁻⁵	2.48 x 10 ⁻¹⁰
					6.77 x 10 ⁻⁷	1.54 x 10 ⁻¹¹
					2.16 x 10 ⁻³	1.46 x 10 ⁻⁵
					2.25 x 10 ⁻³	1.41 x 10 ⁻⁵

*Negligibly small failure probability.

0626040281/1

TABLE 7 (continued)

INDIAN POINT 3 SERVICE WATER SYSTEM - CAUSE TABLE

Nuclear Header
Case 1 Boundary Condition: Power Lost at One 480V Switchgear Bus

Component	Failure Data			(Demand)		System Effect	Failure Prob (t = 24 hours)	
	Mean	Variance	(Hourly)	Mean	Variance		Mean	Variance
A. Single Events								
1. Piping failures (10)	8.60 x 10 ⁻⁹	4.29 x 10 ⁻¹⁵		-	-	System failure	2.06 x 10 ⁻⁷	2.47 x 10 ⁻¹²
2. Header isolation valves	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴		-	-	System failure	2.20 x 10 ⁻⁶	5.51 x 10 ⁻¹²
B. Double Events								
1. Pump trains								
a. Pump fails to start	-	-		1.36 x 10 ⁻³	1.22 x 10 ⁻⁶			
b. Pump fails to run	4.68 x 10 ⁻⁵	1.07 x 10 ⁻⁹		-	-			
c. Expansion joint ruptures	8.60 x 10 ⁻⁹	6.00 x 10 ⁻¹⁵		6.91 x 10 ⁻⁵	1.03 x 10 ⁻⁸			
d. Check valve fails to open	-	-		-	-			
e. Discharge valve fails	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴		1.43 x 10 ⁻³	1.12 x 10 ⁻⁶	System failure	3.69 x 10 ⁻³	1.88 x 10 ⁻⁶
Single train total	4.69 x 10 ⁻⁵	9.54 x 10 ⁻¹⁰		-	-			
2 of 3 pump trains	-	-		-	-			
2. EDG flow control valve	-	-		4.98 x 10 ⁻⁴	4.03 x 10 ⁻⁷	System failure	6.27 x 10 ⁻⁷	1.54 x 10 ⁻¹¹
a. valve fails	-	-		-	-			
2 of 2 valves	-	-		-	-			
C. Human Error								
1. Mispositioned switch	-	-		-	-	System failure	2.16 x 10 ⁻³	1.46 x 10 ⁻⁵
D. Common Cause	-	-		-	-			
E. Other	-	-		-	-			
F. Header Total	-	-		-	-	System failure	5.05 x 10 ⁻³	1.58 x 10 ⁻⁵

* Negligibly small failure probability.

06/6A0402/11/1

TABLE 7 (continued)

INDIAN POINT 3 SERVICE WATER SYSTEM - CAUSE TABLE

Nuclear Header
Case 2 Boundary Condition: Power Lost at One 480V Switchgear Bus

Component	Failure Data (Hourly)		Failure Data (Demand)		System Effect	Failure Data (t = 24 hours)	
	Mean	Variance	Mean	Variance		Mean	Variance
A. Single Events							
1. Piping failures (10)	8.60 x 10 ⁻⁹	4.29 x 10 ⁻¹⁵	-	-	System failure	2.06 x 10 ⁻⁷	2.47 x 10 ⁻¹²
2. Header Isolation valves	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴	-	-	System failure	7.11 x 10 ⁻⁵	8.91 x 10 ⁻⁹
B. Double Events							
1. Pump trains							
a. Pump fails to start	-	-	1.16 x 10 ⁻³	1.22 x 10 ⁻⁶			
b. Pump fails to run	4.60 x 10 ⁻⁵	1.07 x 10 ⁻⁹	-	-			
c. Expansion joint ruptures	8.60 x 10 ⁻⁹	6.00 x 10 ⁻¹⁵	-	-			
d. Check valve fails to open	-	-	6.91 x 10 ⁻⁵	1.01 x 10 ⁻⁸			
e. Discharge valve fails	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴	-	-			
Single train total	4.69 x 10 ⁻⁵	9.54 x 10 ⁻¹⁰	1.43 x 10 ⁻³	1.12 x 10 ⁻⁶	System failure	5.11 x 10 ⁻³	2.68 x 10 ⁻⁶
2 of 3 pump trains	-	-	-	-			
2. EUG flow control valve	-	-	4.90 x 10 ⁻⁴	4.03 x 10 ⁻⁷	System failure	6.27 x 10 ⁻⁷	1.54 x 10 ⁻¹¹
a. valve fails	-	-	-	-			
2 of 2 valves	-	-	-	-			
C. Human Error							
1. Mispositioned switch	-	-	-	-	System failure	2.16 x 10 ⁻³	1.46 x 10 ⁻⁵
D. Common Cause							
E. Other							
F. Header Total					System failure	7.35 x 10 ⁻³	1.66 x 10 ⁻⁵

*Negligibly small failure probability.

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TABLE 7 (continued)

INDIAN POINT 3 SERVICE WATER SYSTEM - CAUSE TABLE

Nuclear Header

Case 1 Boundary Condition: Power Lost at Two 480V Switchgear Buses

Component	Failure Data (Hourly)		Failure Data (Demand)		System Effect	Failure Data (t = 24 hours)	
	Mean	Variance	Mean	Variance		Mean	Variance
A. Single Events							
1. Piping failures (10)	8.60×10^{-9}	4.29×10^{-15}	-	-	System failure	2.06×10^{-7}	2.47×10^{-12}
2. Header isolation valves	9.15×10^{-8}	1.01×10^{-14}	-	-	System failure	2.20×10^{-6}	5.51×10^{-12}
B. Double Events							
1. Pump trains							
a. Pump fails to start	-	-	1.36×10^{-3}	1.22×10^{-6}			
b. Pump fails to run	4.68×10^{-5}	1.07×10^{-9}	-	-			
c. Expansion joint ruptures	8.60×10^{-9}	6.00×10^{-15}	-	-			
d. Check valve fails to open	-	-	6.91×10^{-5}	1.03×10^{-8}			
e. Discharge valve fails	9.15×10^{-8}	1.01×10^{-14}	-	-			
Single train total	4.69×10^{-5}	9.54×10^{-10}	1.43×10^{-3}	1.12×10^{-6}			
2 of 3 pump trains	-	-	-	-	System failure	1.0	0.0
2. EDG flow control valve							
a. valve fails	-	-	4.90×10^{-4}	4.03×10^{-7}			
2 of 2 valves	-	-	-	-	System failure	6.27×10^{-7}	1.54×10^{-11}
C. Human Error							
1. Mispositioned switch	-	-	-	-	System failure	2.16×10^{-3}	1.46×10^{-5}
D. Common Cause	*	-	-	-			
E. Other	*	-	-	-			
F. Header Total					System failure	1.0	0.0

*Negligibly small failure probability.

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TABLE 7 (continued)

INDIAN POINT 3 SERVICE WATER SYSTEM - CAUSE TABLE

Nuclear Header
Case 2 Boundary Condition: Power Lost at Two 400V Switchgear Buses

Component	Failure Data			System Effect		
	(Hourly)		(Demand)	Failure Data (t = 24 hours)		
	Mean	Variance	Mean	Variance	Mean	Variance
A. Single Events						
1. Piping failures (10)	8.60 x 10 ⁻⁹	4.29 x 10 ⁻¹⁵			System failure 2.06 x 10 ⁻⁷	2.47 x 10 ⁻¹²
2. Header isolation valves	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴	6.91 x 10 ⁻⁵	1.03 x 10 ⁻⁸	System failure 7.11 x 10 ⁻⁵	8.93 x 10 ⁻⁹
B. Double Events						
1. Pump trains						
a. Pump fails to start			1.16 x 10 ⁻¹	1.22 x 10 ⁻⁶		
b. Pump fails to run	4.60 x 10 ⁻⁵	1.07 x 10 ⁻⁹	-	-		
c. Expansion joint ruptures	8.60 x 10 ⁻⁹	6.00 x 10 ⁻¹⁵	-	-		
d. Check valve fails to open	-	-	6.91 x 10 ⁻⁵	1.03 x 10 ⁻⁸		
e. Discharge valve fails	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴	-	-		
Single train total	4.69 x 10 ⁻⁵	9.54 x 10 ⁻¹⁰	1.43 x 10 ⁻³	1.12 x 10 ⁻⁶	System failure 1.0	0.0
2. (DG Flow control valve						
a. valve fails	-	-	4.90 x 10 ⁻⁴	4.03 x 10 ⁻⁷	System failure 6.27 x 10 ⁻⁷	1.54 x 10 ⁻¹¹
2 of 2 valves	-	-	-	-	System failure 2.16 x 10 ⁻³	1.46 x 10 ⁻⁵
C. Human Error						
1. Mispositioned switch	-	-	-	-	System failure 1.0	0.0
D. Common Cause						
E. Other						
F. Header Total						

*Negligibly small failure probability.

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INDIAN POINT 3 SERVICE WATER SYSTEM - CAUSE TABLE

Conventional Header
Case's 1 and 2 Boundary Condition; Power at All 40W Switchgear Illuses

Component	Failure Data (Hourly)			Failure Data (Demand)			Failure Data (t = 24 hours)		
	Mean	Variance		Mean	Variance		Mean	Variance	
A. Single Events									
1. Piping failures (0)									
2. Header Isolation valves	6.80 × 10 ⁻⁹	2.75 × 10 ⁻¹⁵							
	9.15 × 10 ⁻⁸	1.01 × 10 ⁻¹⁴		6.91 × 10 ⁻⁵	1.03 × 10 ⁻¹¹		1.65 × 10 ⁻⁷	1.58 × 10 ⁻¹²	
B. Triple Events									
1. Pump trains									
a. Pump fails to start									
b. Pump fails to run	4.68 × 10 ⁻⁵	1.07 × 10 ⁻⁹		1.36 × 10 ⁻³	1.22 × 10 ⁻⁶				
c. Expansion joint ruptures	0.60 × 10 ⁻⁹	6.00 × 10 ⁻¹⁵							
d. Check valve fails to open									
e. Discharge valve fails	9.15 × 10 ⁻⁸	1.01 × 10 ⁻¹⁴		6.91 × 10 ⁻⁵	1.03 × 10 ⁻¹¹				
Single train total	4.69 × 10 ⁻⁵	9.54 × 10 ⁻¹⁰		1.43 × 10 ⁻³	1.17 × 10 ⁻⁶				
1 of 3 pump trains							3.61 × 10 ⁻⁷	1.49 × 10 ⁻¹³	
C. Test and Maintenance									
(with pump trains)									
D. Common Cause	*								
E. Other	*								
F. Header Total							7.18 × 10 ⁻⁵	8.84 × 10 ⁻⁹	

^anegligibly small failure probability.

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TABLE 7 (continued)

INDIAN POINT 3 SERVICE WATER SYSTEM - CAUSE TABLE

Conventional Header
Cases 1 and 2 Boundary Condition: Power Lost at One 400W Switchgear Bus

Component	Failure Data			(Demand)		System Effect	Failure Data (t = 24 hours)	
	Mean	Variance	(Hourly)	Mean	Variance		Mean	Variance
A. Single Events								
1. Piping failures (6)	6.88 x 10 ⁻⁹	2.75 x 10 ⁻¹⁵						
2. Header isolation valves	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴		6.91 x 10 ⁻⁵	1.03 x 10 ⁻⁸	System failure	1.65 x 10 ⁻⁷	1.58 x 10 ⁻¹²
						System failure	7.11 x 10 ⁻⁵	8.93 x 10 ⁻⁹
B. Triple Events								
1. Pump trains								
a. Pump fails to start				1.36 x 10 ⁻³	1.22 x 10 ⁻⁶			
b. Pump fails to run	4.68 x 10 ⁻⁵	1.07 x 10 ⁻⁹						
c. Expansion joint ruptures	8.60 x 10 ⁻⁹	6.00 x 10 ⁻¹⁵						
d. Check valve fails to open				6.91 x 10 ⁻⁵	1.03 x 10 ⁻⁸			
e. Discharge valve fails	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴						
Single train total	4.69 x 10 ⁻⁵	9.54 x 10 ⁻¹⁰		1.43 x 10 ⁻³	1.12 x 10 ⁻⁶	System failure	4.40 x 10 ⁻⁵	5.94 x 10 ⁻¹⁰
1 of 3 pump trains								
C. Test and Maintenance								
(with pump trains)								
D. Common Cause								
E. Other								
F. Header total						System failure	1.15 x 10 ⁻⁴	9.34 x 10 ⁻⁹

*Negligibly small failure probability.

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TABLE 7 (continued)

INDIAN POINT 3 SERVICE WATER SYSTEM - CAUSE TABLE

Conventional Header
Cases 1 and 2 Boundary Condition: Power Lost at Two 400W Switchgear Buses

Component	Hourly		Failure Data		(Demand)		System Effect		Failure Data (t = 24 hours)	
	Mean	Variance	Mean	Variance	Mean	Variance	Mean	Variance	Mean	Variance
A. Single Events										
1. Piping failures (8)	6.88 x 10 ⁻⁹	2.75 x 10 ⁻¹⁵					System failure	1.65 x 10 ⁻⁷	1.58 x 10 ⁻¹²	
2. Header isolation valves	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴	6.91 x 10 ⁻⁵		1.03 x 10 ⁻⁸		System failure	7.11 x 10 ⁻⁵	8.91 x 10 ⁻⁹	
B. Triple Events										
1. Pump trains										
a. Pump fails to start			1.36 x 10 ⁻³		1.22 x 10 ⁻⁶					
b. Pump fails to run	4.60 x 10 ⁻⁵	1.07 x 10 ⁻⁹								
c. Expansion joint ruptures	8.60 x 10 ⁻⁹	6.00 x 10 ⁻¹⁵								
d. Check valve fails to open			6.91 x 10 ⁻⁵		1.03 x 10 ⁻⁸					
e. Discharge valve fails	9.15 x 10 ⁻⁸	1.01 x 10 ⁻¹⁴								
Single train total	4.69 x 10 ⁻⁵	9.54 x 10 ⁻¹⁰	1.43 x 10 ⁻³		1.12 x 10 ⁻⁶		System failure	1.72 x 10 ⁻²	3.87 x 10 ⁻⁵	
1 of 3 pump trains										
C. Test and Maintenance										
(with pump trains)										
D. Common Cause										
E. Other										
F. Header Total							System failure	1.73 x 10 ⁻²	3.85 x 10 ⁻⁵	

*Negligibly small failure probability.

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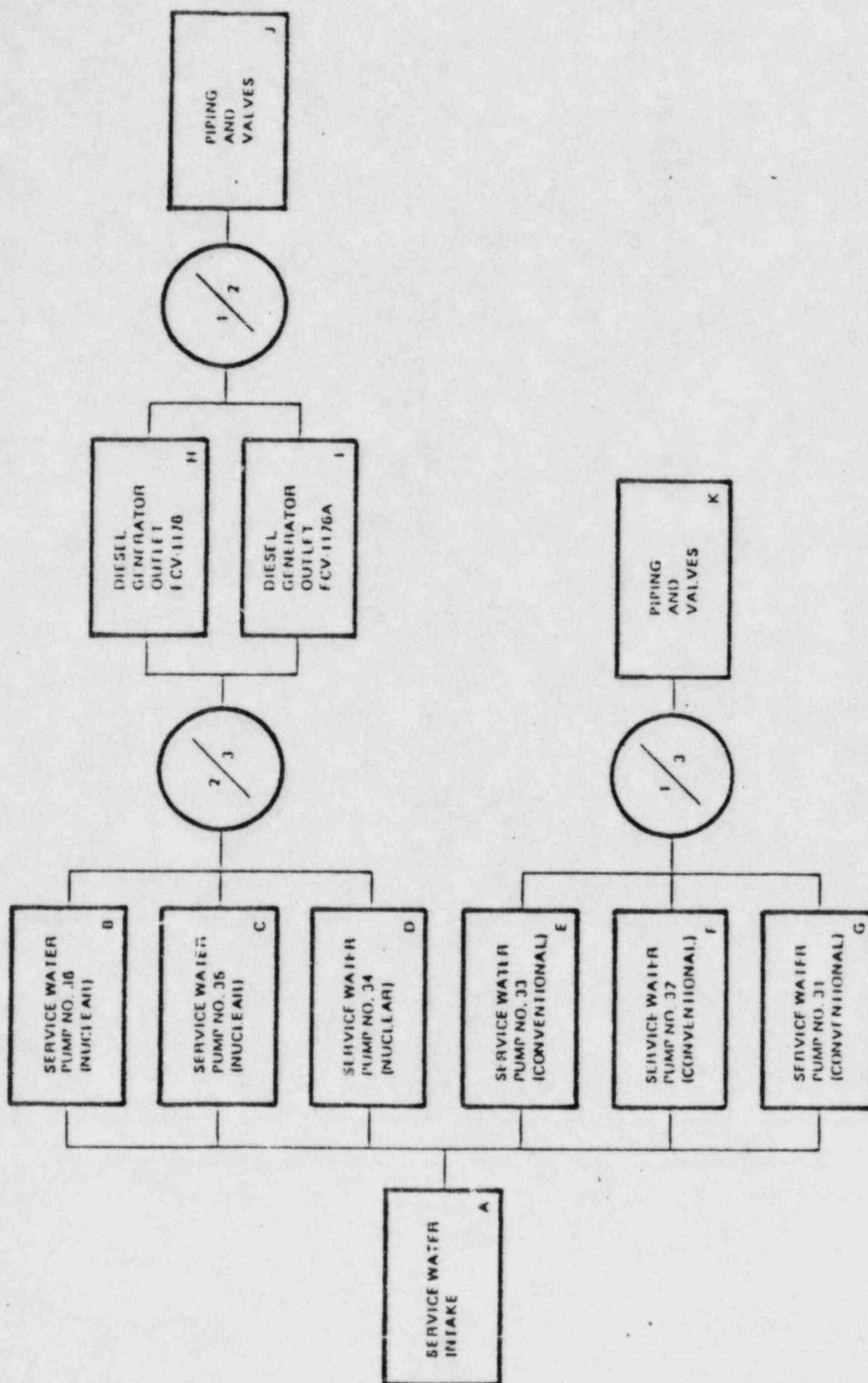


Figure 1. Service Water System Block Diagram

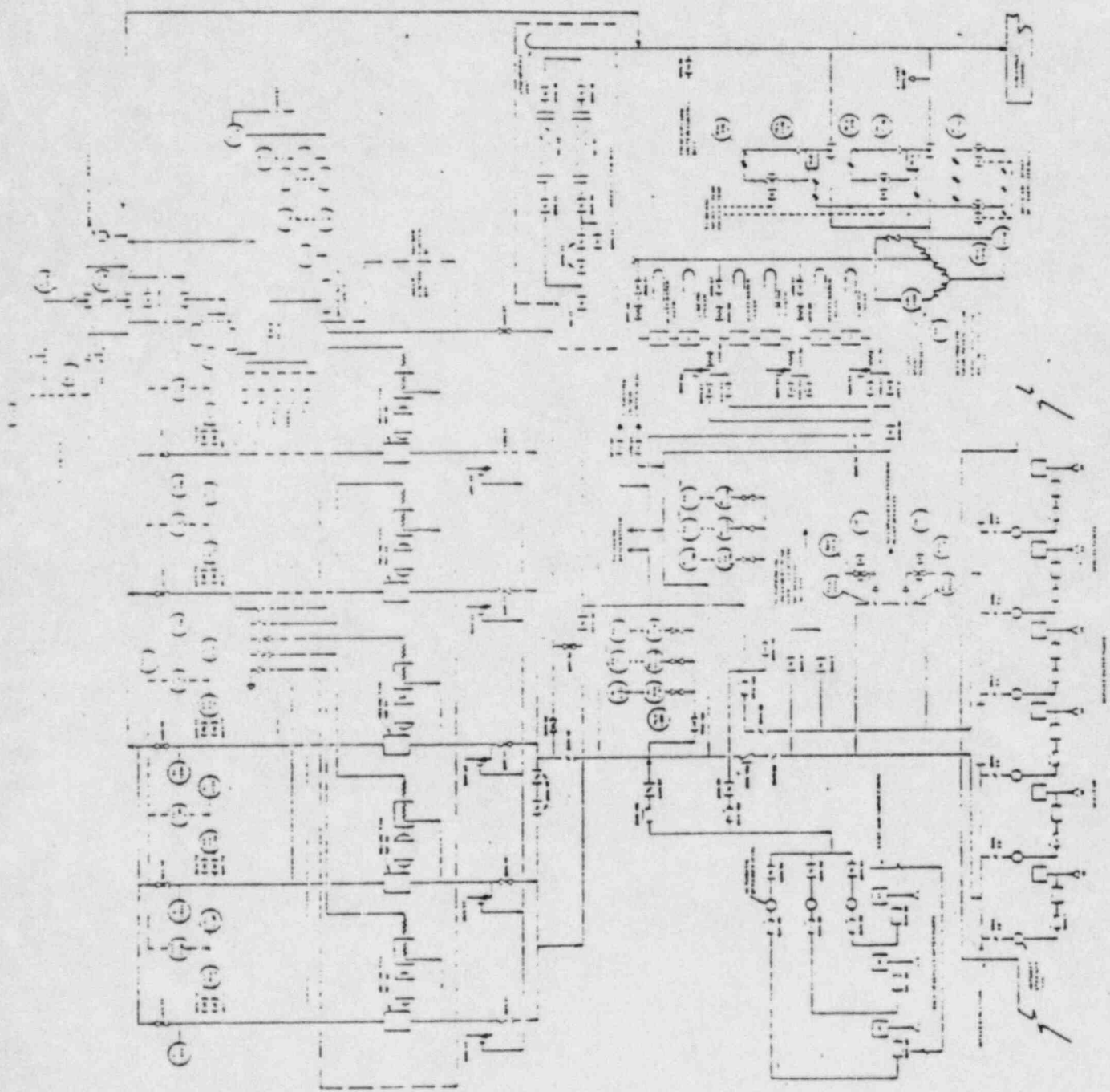


Figure 2. Simplified Schematic of Service Water System

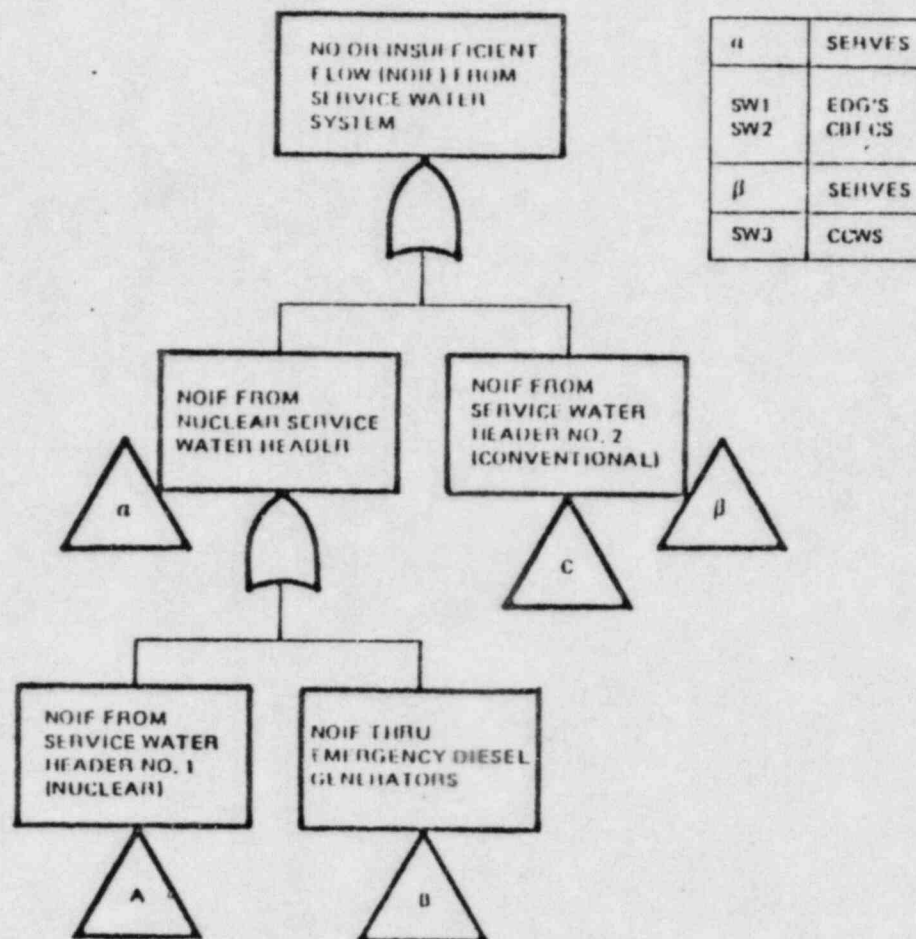


Figure 3. Service Water System Fault Trees
(Sheet 1 of 6)

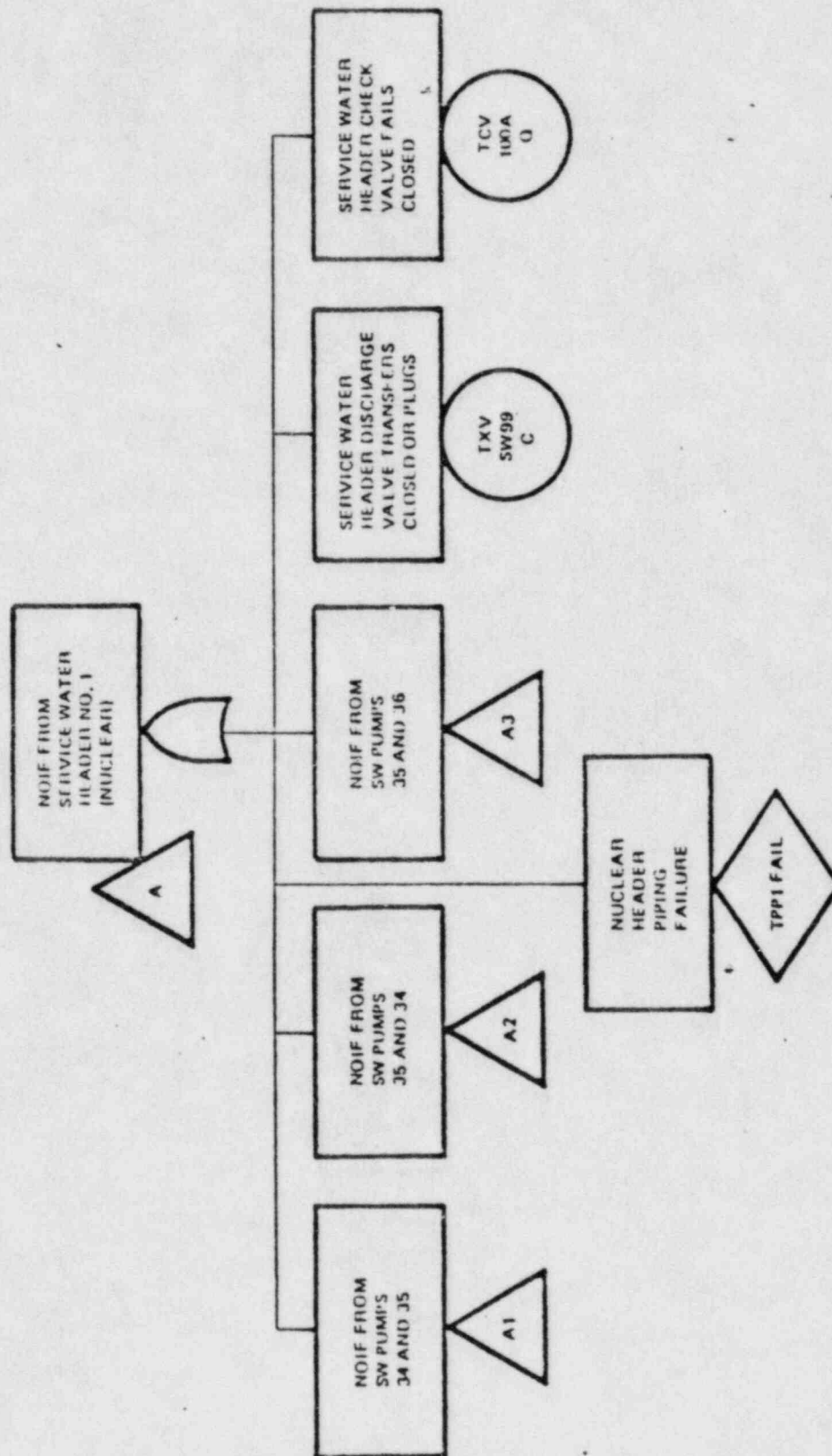


Figure 3. (Sheet 2 of 6)

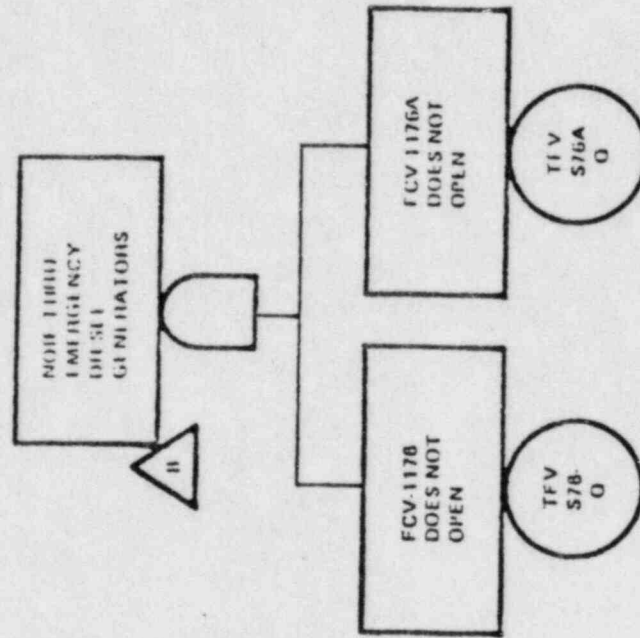


Figure 3. (Sheet 3 of 6)

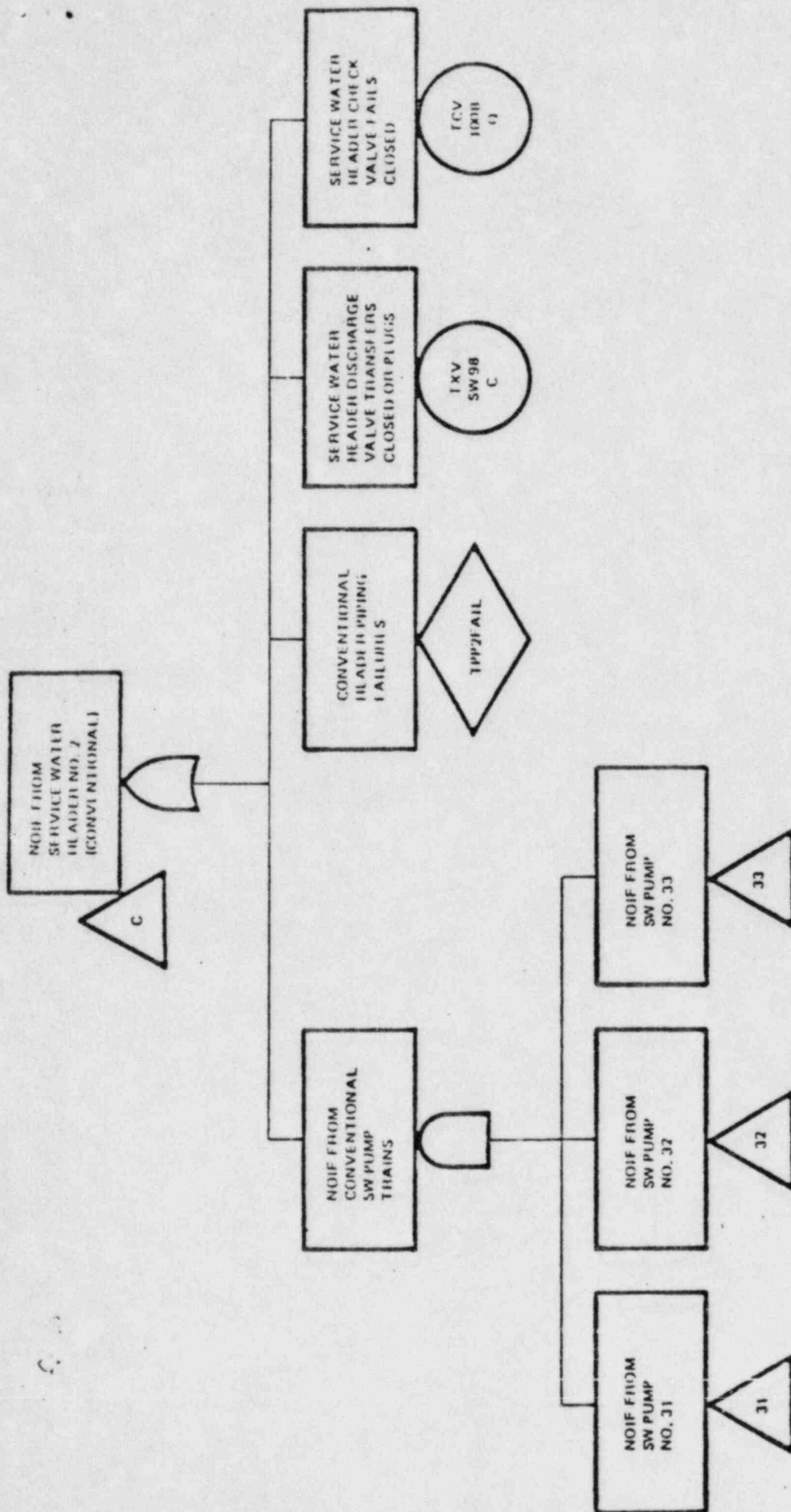


Figure 3. (Sheet 4 of 6)

α	γ	δ	ϵ	ϕ	θ
A1	34	35	3A	5	33
A2	36	34	5A	4	31
A3	35	36	6A	6	32

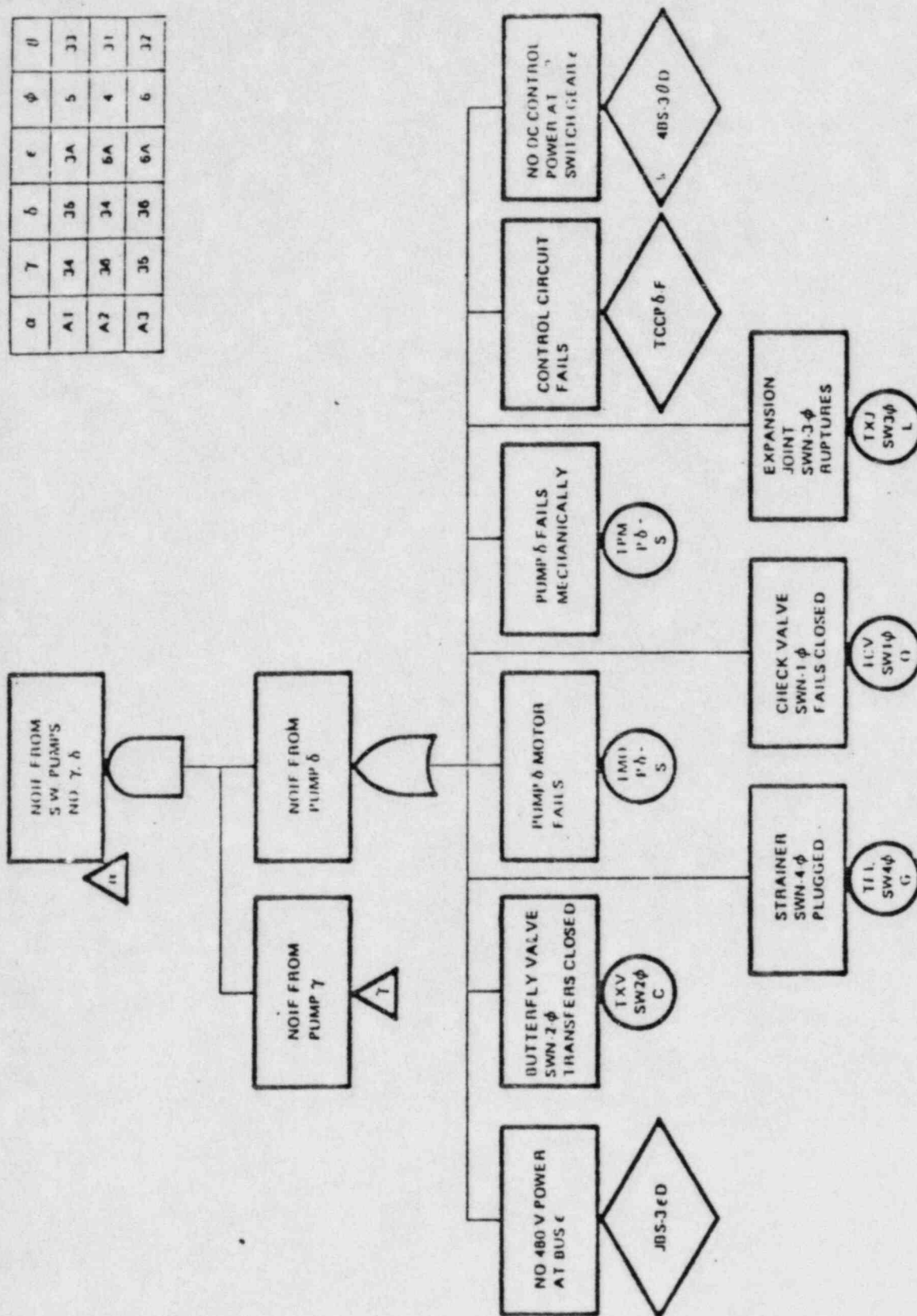


Figure 3. (Sheet 5 of 6)

7	1	0	0
34	6A	4	31
36	6A	6	32
35	3A	6	33
31	6A	1	31
33	6A	3	12
32	3A	2	31

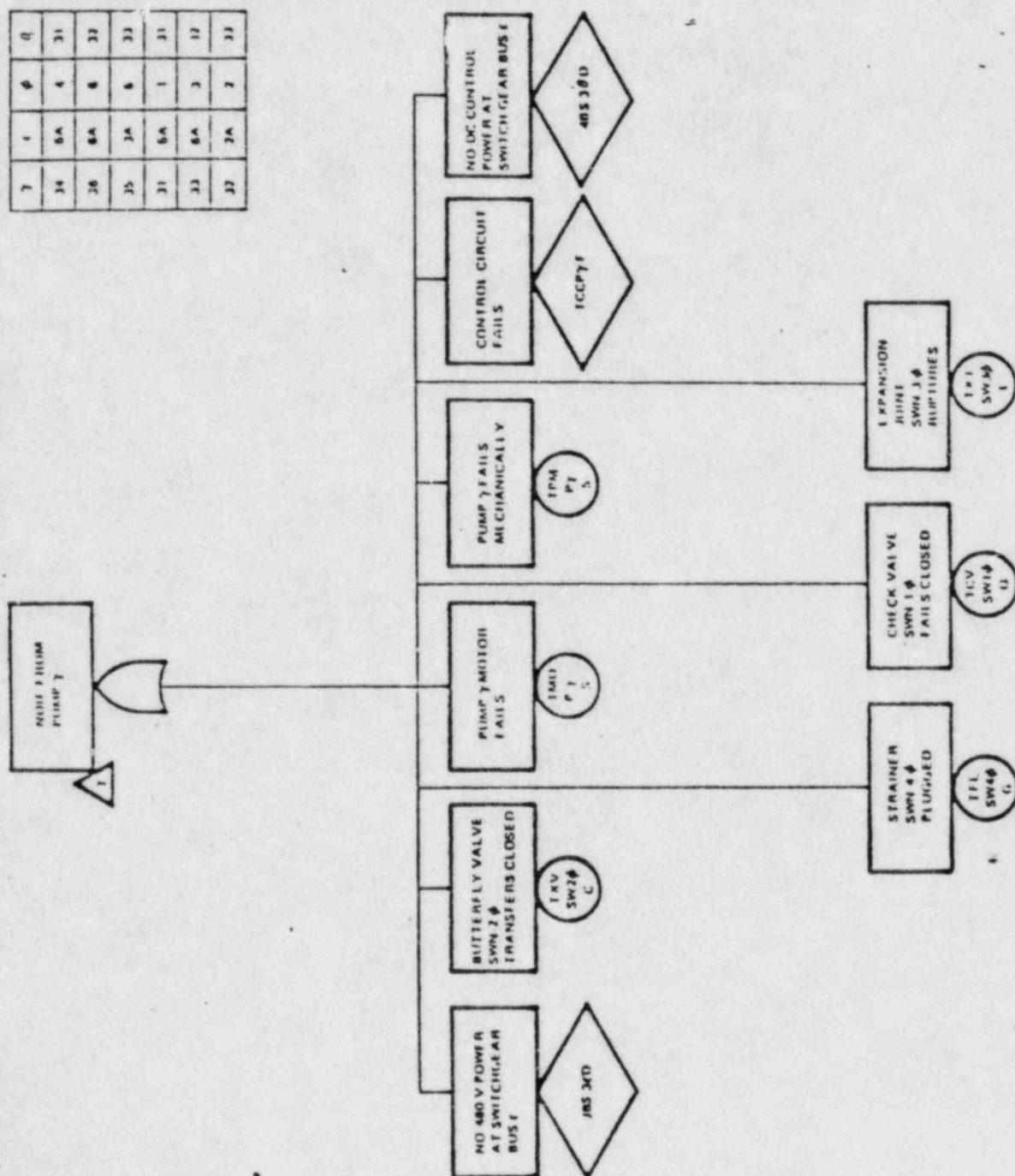


Figure 3. (Sheet 6 of 6)