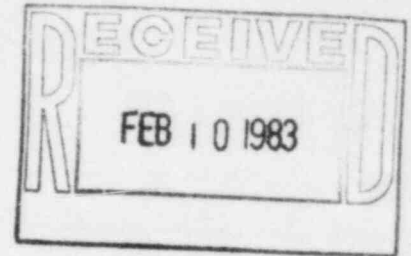




Public Service Company of Colorado

16805 Road 19 1/2, Platteville, Colorado 80651-9298

February 7, 1983
Fort St. Vrain
Unit No. 1
P-83047



Mr. John T. Collins, Regional Administrator
Region IV
Nuclear Regulatory Commission
611 Ryan Plaza Drive
Suite 1000
Arlington, Texas 76011

Reference: Facility Operating License
No. DPR-34

Docket No. 50-267

Dear Mr. Collins:

Enclosed please find a copy of Reportable Occurrence Report No. 50-267/82-052, Revised Final, submitted per the requirements of Technical Specification AC 7.5.2(b)2.

Also, please find enclosed one copy of the Licensee Event Report for Reportable Occurrence Report No. 50-267/82-052.

Very truly yours,

Don Warembourg
Don Warembourg
Manager, Nuclear Production

DW/clh

Enclosure

cc: Director, MIPC

H005

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PDR ADOCK 05000267
S PDR

REPORT DATE: February 7, 1983

REPORTABLE OCCURRENCE 82-052

ISSUE 1

OCCURRENCE DATE: December 29, 1982

Page 1 of 4

FORT ST. VRAIN NUCLEAR GENERATING STATION
PUBLIC SERVICE COMPANY OF COLORADO
16805 WELD COUNTY ROAD 19 1/2
PLATTEVILLE, COLORADO 80651-9298

REPORT NO. 50-267/82-052/03-L-1

Revised Final

IDENTIFICATION OF
OCCURRENCE:

During the period from December 29, 1982, through January 11, 1983, eight Class I hydraulic shock suppressors (snubbers) were found to be inoperable. These events constitute operation in a degraded mode of LCO 4.3.10 and are reportable per Fort St. Vrain Technical Specification AC 7.5.2(b)2.

EVENT
DESCRIPTION:

Event Nos. 1, 2, and 3 occurred while the reactor was shutdown, but the reactor had been operated at power prior to the occurrence and subsequent to the most recent check for operability. Therefore, the associated shock suppressors are assumed to have been inoperable during power operation.

Event No. 4 occurred at approximately 35% reactor power during a rise to 70% reactor power.

Event Nos. 5, 6, 7, and 8 occurred during steady state operation at 70% reactor power.

Each event is described in the following table:

<u>Event No.</u>	<u>Date</u>	<u>Snubber Identification</u>	<u>Event Description</u>
1	12-29-82	VSS-104	Improper reservoir orientation
2	12-29-82	VSS-105	Improper reservoir orientation
3	12-29-82	HRS-69	No visible oil level.
4	1-4-83	BFS-268E	No visible oil level.
5	1-6-83	BFS-216E	No visible oil level.
6	1-7-83	HRS-324	No visible oil level.
7	1-8-83	BFS-711	No visible oil level.
8	1-10-83	HRS-98	No visible oil level.

CAUSE
DESCRIPTION:

The cause of each event is described in the following table:

<u>Event No.</u>	<u>Snubber Identification</u>	<u>Cause Description</u>
1	VSS-104	Pipe movement caused reservoir to invert.
2	VSS-105	Pipe movement caused reservoir to invert.
3	HRS-69	Leaking seals and O-rings caused a loss of reservoir oil.
4	BFS-268E	Leaking seals and O-rings caused a loss of reservoir oil.
5	BFS-216E	Leaking seals and O-rings caused a loss of reservoir oil.
6	HRS-324	Leaking seals and O-rings caused a loss of reservoir oil.
7	BFS-711	Leaking seals and O-rings caused a loss of reservoir oil.
8	HRS-98	Leaking seals and O-rings caused a loss of reservoir oil.

CORRECTIVE
ACTION:

The corrective action taken for each event is described in the following table:

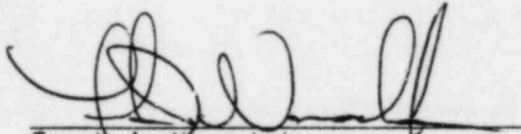
<u>Event No.</u>	<u>Snubber Identification</u>	<u>Corrective Action</u>
1	VSS-104	Rebuilt entire snubber. Tested and re-installed in system.
2	VSS-105	Rebuilt entire snubber. Installed new valve. Tested and re-installed in system.
3	HRS-69	Rebuilt entire snubber. Installed new valve. Tested and re-installed in system.
4	BFS-268E	Rebuilt entire snubber. Installed new valve block and cylinder. Tested and re-installed in system.
5	BFS-216E	Replaced with qualified spare. Tested and re-installed in system.
6	HRS-324	Rebuilt entire snubber. Tested and re-installed in system.
7	BFS-711	Replaced with qualified spare. Tested and re-installed in system.
8	HRS-98	Replaced with qualified spare. Tested and re-installed in system.

In each event, the snubbers were returned to service within the 72 hour grace period allowed by LCO 4.3.10

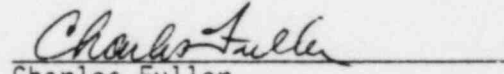
| Since eight snubbers were found inoperable during the inspection
| interval, the next inspection interval has been shortened to 31 days
| \pm 25% per SR 5.3.8(a).

No further corrective action is anticipated or required.

Prepared By:


Frank J. Novachek
Senior Plant Engineer

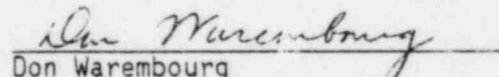
Reviewed By:


Charles Fuller
Technical Services Engineering Supervisor

Reviewed By:


Edwin D. Hill
Station Manager

Approved By:


Don Warembourg
Manager, Nuclear Production