

OPERATIONS

NOVEMBER

At the beginning of the report period, the OC Station was operating at 335 MWe with load limited by core reactivity.

The Plant is still experiencing problems with intermittent salt leaks on "A" Main Condenser and problems with the Plant's air compressors. Corrective Maintenance was required on two of the Plant's air compressors at various times within the report period. Air Compressor 1-2 has been out of service since November 19, 1982. The plant is also experiencing problems with the operation of the air dryers causing periodic low air pressure conditions during swapping of units.

A trip of the Fuel Pool Cooling Pump was experienced due to low-low level in the Fuel Pool Surge Tank. Water from the Fuel Pool (approximately 8000 to 10,000 gallons) was transferred to the Condensate Storage Tank (CST), (there is only one path for the Fuel Pool water to get to the CST). The Plant could not simulate the conditions that caused the Fuel Pool level to decrease without operation of the air operated isolation valve in the line from the Fuel Pool to the CST. The isolation valve is operated from the 75' Elev. of the Reactor Building. Contractors working in this area were questioned on Operation of the valve; however, they stated they did not touch the valve operator switch. The Fuel Pool System was returned to service within the day and contractor work has been suspended in this area pending further evaluation by Operations Management.

At 1510 hours on November 20, 1982, a Reactor shutdown was commenced when the "A" 24 Volt DC System was lost during battery equalizing charge. Loss of "A" 24 Volt System caused a downscale failure of all Intermediate Range Monitors (IRM's) in System I and both Stack Gas Sample Systems. Subsequently, the DC output breaker on Charger "A2" was found tripped at 1525 hours. The breaker was reset and the shutdown terminated at 1535 hours. Load was increased from 300 MWe to 313 MWe. The ensuing investigation revealed that the DC over voltage breaker trip setpoint was too low. The overload trip setpoint was reset.

Reactor Recirculation Pump "A" was removed from service on November 24, 1982 due to a defective pump seal. Plant load was initially reduced from 306 MWe to 280 MWe for removal of the pump from service. Load was increased from 280 MWe to 300 MWe over a ten hour period after securing the pump. As a result of this event the Drywell Unidentified Leak Rate has exhibited an increasing trend.

The following maintenance activities were performed during the report period:

1. The impellers on Emergency Service Water Pump 52C and 52D were adjusted.
2. Replaced the brushes on "A" Reactor Recirculation Pump MG Set.
3. Repaired Stack Gas Sample Pump "B".
4. Condensate Transfer Pump No. 2 was repaired and returned to service.

The following events were identified as potential Reportable Occurrences:

On November 5, 1982, the Primary Containment Isolation ball valve for TIP Machine No. 3 failed to close.

On November 15, 1982, "B" Stack Gas Sample Pump tripped causing an interruption in the Stack Gas Sample Monitoring System.

On November 16, 1982, Dilution Pump No. 3 tripped thus leaving the Plant operating without the minimum number of dilution pumps required by the Technical Specifications for a period of 21 minutes.

On November 18, 1982, the Backseating Terminal Board for Isolation Condenser Valve V-14-37 failed, causing inoperability of the valve.

On November 20, 1982, both Stack Gas Sample Systems were lost due to the DC output breaker on 24 Volt DC Charger "A-2" tripping.

On November 22, 1982, Dilution Pump No. 3 was removed from service with intake temperature less than 60°F to repair a ground in the vacuum breaker limit switch, thus leaving only one (1) pump running.

On November 28, 1982, V-1-106 and V-1-110 did not fully close during the MSIV Closure Test.

On November 30, 1982, Dilution Pump No. 2 was removed from service with intake temperature less than 60°F to clean clogged trash grates, thus leaving only one pump running.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH November 1982

DOCKET NO. 50-219
 UNIT NAME Oyster Creek
 DATE 12-1-82
 COMPLETED BY R. Baran
 TELEPHONE 971-4640

No	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
28	11-28-82	S	0	H	4	NA	ZZ	ZZZZZZ	Power Reduction of 80 MWe for a period of 2.5 hours to perform the Main Steam Isolation Valve Closure Test.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

AVERAGE DAILY POWER LEVEL

Net MWe

Docket #50219
 UnitO.C. #1
 Report Date. . . .December 6, 1982
 Compiled by. . . .Robert J. Frick
 Telephone.609-971-4637

Month: November, 1982

Day	MW	Day	MW
1	309	16	291
2	307	17	290
3	303	18	290
4	300	19	290
5	300	20	284
6	301	21	285
7	300	22	283
8	299	23	281
9	296	24	268
10	297	25	276
11	297	26	274
12	295	27	271
13	292	28	265
14	293	29	270
15	292	30	268

UNIT NAME: OYSTER CREEK

1. DOCKET: 50-219
2. REPORTING PERIOD: 11/82
3. UTILITY CONTACT: ROBERT J. FRICK 609-971-4637
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe): $637.5 * 0.8$
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 650
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS:
NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): 300
11. REASON FOR RESTRICTION, IF ANY:
FUEL DEPLETION

	MONTH	YEAR	CUMULATIVE
12. REPORT PERIOD HRS	720.0	8016.0	113424.0
13. HOURS RX CRITICAL	720.0	5084.9	84812.0
14. RX RESERVE SHUTDOWN HRS	0.0	0.0	468.2
15. HRS GENERATOR ON-LINE	720.0	4961.1	82001.6
16. UT RESERVE SHUTDOWN HRS	0.0	0.0	0.0
17. GROSS THERM ENER (MWH)	726700.0	6306000.0	135863309.0
18. GROSS ELEC ENER (MWH)	221490.0	1990830.0	45907050.0
19. NET ELEC ENER (MWH)	207940.0	1878160.0	44119000.0

20. UT SERVICE FACTOR	100.0	62.1	72.3
21. UT AVAIL FACTOR	100.0	62.1	72.3
22. UT CAP FACTOR (MDC NET)	46.6	37.8	62.7
23. UT CAP FACTOR (DER NET)	44.4	36.0	59.8
24. UT FORCED OUTAGE RATE	0.0	37.9	9.6
25. FORCED OUTAGE HRS	0.0	3041.5	8674.1
26. SHUTDOWNS SCHED OVER NEXT 6 MONTHS (TYPE,DATE,DURATION): REFUELING AND MAINTENANCE, 1/15/83, 11 MONTHS			
27. IF CURRENTLY SHUTDOWN ESTIMATED STARTUP TIME: N/A			

<u>EQUIPMENT</u>	<u>MALFUNCTION</u>	<u>CORRECTIVE ACTION</u>
DG #1 Output Breaker Control	Tripped at 600 KW, Set Point is 500 ± 50 KW	Reset to 480 KW
1-2 Condensate Transfer Pump	Noisy contactor	Cleaned contactor
Fire Detection System, Diesel Generator Fuel Storage Area	Spurious yellow trouble alarms	Found bad 2N-31 module. Replaced same.
Fire Detection System, NRW Control Room	Alarm malfunction	Replaced one thermal detector
Reactor Building, SE, airlock (outside door)	Electromagnetic control plunger sticks	Cleaned plunger
Refueling Platform, Master Hoist Control Switch	Will not return to neutral	Replaced spring in control switch
CRD Room Airlock Switchbox	Loose bolts	Replaced bolts and tightened
Reactor Building NW Airlock (outside door)	Electromagnetic control plunger sticks	Cleaned plunger
Diesel Generator #2	Oscillating ± 175 KW at full load	Cleaned all potentiometers and satisfactorily performed surveillance.
Reactor Building Airlock, 51' Elevation	Not working	Repaired loose wire on interlock switch

[illegible]

[illegible]

REFUELING INFORMATION -

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: January 15, 1983

Scheduled date for restart following refueling: late - 1983

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Technical Specification Change Request No. 96 was submitted on August 31, 1982 for incorporation of GE fuel assemblies into the Cycle 10 core.

Scheduled date(s) for submitting proposed licensing action and supporting information:

June 1, 1983 - The final supplement to the reload analysis, delineating the specific core configuration for Cycle 10 operation, will be submitted.

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric fuel assemblies - fuel design and performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
2. Exxon Fuel Assemblies - No major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core - 560
(b) in the spent fuel storage pool - 781

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1,800 Planned: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

The Spring 1987 Outage.*

*NOTE: This is for a normal refueling. Full core off-load, however, can only be accommodated through about 1983 or 1984 with 1800 licensed locations.