No. 3-82-22/1X-1

Mr. R. C. Haynes
Administrator
U.S. Nuclear Regulatory
Commission
631 Park Avenue
King of Prussia, PA 19406

Dear Mr. Haynes:

This LER deals with a revision to an LER on uncontrolled release of liquid effluents, per Tech. Spec. 3.8.B.

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | PAPBS3 (2) | 0 0 - 0 0 0 0 0 0 0 0 (3) | 4 | 1 1 1 1 (4) | | (5)
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T
0 1 | REPORT SOURCE L (6) | 0 5 0 - 0 2 7 8 (7) | 1 1 0 4 8 2 (8) | 0 1 1 1 8 3 (9)
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | The 3D RHR heat exchanger had leaked slightly radioactive water
0 3 | into the unit 3 intake structure via the high pressure service water
0 4 | piping during the period of October 25 through November 2, 1982.
0 5 | Total release is estimated to be 22.8 millicuries.
0 6 |
0 7 |
0 8 | 80
7 8 9

0 9 | SYSTEM CODE C F (11) | CAUSE CODE E (12) | CAUSE SUBCODE B (13) | COMPONENT CODE H T E X C H (14) | COMP. SUBCODE G (15) | VALVE SUBCODE Z (16)
7 8 9 10 11 12 13 18 19 20

(17) LER/RO REPORT NUMBER | EVENT YEAR 8 2 | — | SEQUENTIAL REPORT NO. 0 2 2 | / | OCCURRENCE CODE 0 1 | REPORT TYPE X | — | REVISION NO. 1
21 22 23 24 26 27 28 29 30 31 32

ACTION TAKEN D (18) | FUTURE ACTION D (19) | EFFECT ON PLANT Z (20) | SHUTDOWN METHOD Z (21) | HOURS (22) 0 0 0 0 | ATTACHMENT SUBMITTED N (23) | NPRD-4 FORM SUB. N (24) | PRIME COMP. SUPPLIER N (25) | COMPONENT MANUFACTURER P 1 6 0 (26)
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The cause of the event was 3D RHR heat exchanger leak. The heat
1 1 | exchanger was taken out of service and was repaired. The cause of the
1 2 | leak was an expansion bellows between the inner floating head and
1 3 | drain. Additional routine radiation survey of HPSW inlet and outlet
1 4 | to heat exchanger will be initiated.
7 8 9 80

1 5 | FACILITY STATUS E (28) | % POWER 0 8 9 (29) | OTHER STATUS (30) NA | METHOD OF DISCOVERY C (31) | DISCOVERY DESCRIPTION (32) Test Engineer Investigation
7 8 9 10 12 13 44 45 46 80

1 6 | ACTIVITY RELEASED L (33) | CONTENT OF RELEASE M (34) | AMOUNT OF ACTIVITY (35) 22.8 millicuries | LOCATION OF RELEASE (36) From 3DHPSW pump to intake structure
7 8 9 10 11 44 45 80

1 7 | PERSONNEL EXPOSURES NUMBER 0 1 0 (37) | TYPE E (38) | DESCRIPTION (39) Max dose rate -1R/HR; 73 maint.-9.1R, 29oper-8R,4Eng.-.2R
7 8 9 11 12 13 80

1 8 | PERSONNEL INJURIES NUMBER 0 0 0 (40) | DESCRIPTION (41) NA
7 8 9 11 12 80

1 9 | LOSS OF OR DAMAGE TO FACILITY TYPE Z (42) | DESCRIPTION (43) NA
7 8 9 10 80

IE22

PUBLICITY
2 0 | ISSUED N (44) | DESCRIPTION (45) NA | NRC USE ONLY
68 69 80

8301260462 830111
PDR ADOCK 05000278
S PDR

M.J. Cooney

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NRC FORM 366