



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

March 19, 2020

Mr. John A. Krakuszeski
Vice President
Brunswick Steam Electric Plant
Duke Energy Progress, LLC
8470 River Rd. SE (M/C BNP001)
Southport, NC 28461

**SUBJECT: BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2 – CORRECTION TO
AMENDMENT NO. 304 REGARDING REQUEST TO RELOCATE SPECIFIC
SURVEILLANCE FREQUENCIES TO A LICENSEE CONTROLLED PROGRAM
(CAC NO. MF7207)**

Dear Mr. Krakuszeski:

On May 24, 2017, the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 304 to Renewed Facility Operating License No. DPR-62 for Brunswick Steam Electric Plant (Brunswick), Unit 2 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML17096A129). The amendment revised the Technical Specifications (TSs) by relocating specific surveillance frequencies to a licensee-controlled program, consistent with the NRC-approved Technical Specifications Task Force Traveler TSTF-425, Revision 3, "Relocate Surveillance Frequencies to Licensee Control - RITSTF Initiative 5b" (ADAMS Accession No. ML090850642).

By letter dated December 17, 2019 (ADAMS Accession No. ML19351E364), Duke Energy Progress, LLC (the licensee) requested NRC approval of a correction of a typographical error in the Brunswick, Unit 2 Technical Specifications. The typographical error revised the wording of Surveillance Requirement 3.3.1.1.18 from, "Adjust recirculation drive flow to conform to reactor core flow," to "Adjust the flow control trip reference card to conform to reactor flow." The licensee stated that the error had been inadvertently introduced during the processing of Amendment No. 304 and was neither addressed in the notice to the public nor reviewed by the NRC. Thus, the licensee stated that the typographical error falls within the scope of the guidance provided by SECY-96-238, "Proposed Guidance for Correction of Technical Specification Typographical Errors," dated December 17, 1996 (ADAMS Accession No. ML003754054).

The NRC staff has confirmed that Amendment No. 304 for Brunswick, Unit 2 contained a typographical error on TS page 3.3-8, as described above. Further, the NRC staff has determined that this error was inadvertently introduced and was not the subject of the amendment or the associated notice to the public. Therefore, consistent with NRC staff guidance dated January 16, 1997 (ADAMS Accession No. ML103260096), based on the NRC's policy established by SECY-96-238, this error can be corrected by a letter to the licensee from the NRC staff.

Enclosed, please find the corrected Brunswick, Unit 2, TS page 3.3-8. This correction does not change any of the conclusions in the safety evaluation associated with Amendment No. 304. If you have any questions regarding this matter, please contact me at (301) 415-8480 or by email at andrew.hon@nrc.gov.

Sincerely,

/RA/

Andrew Hon, Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-324

Enclosures: As stated

cc: Listserv

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.3.1.1.16	Verify Turbine Stop Valve—Closure and Turbine Control Valve Fast Closure, Trip Oil Pressure—Low Functions are not bypassed when THERMAL POWER is $\geq 26\%$ RTP.	In accordance with the Surveillance Frequency Control Program
SR 3.3.1.1.17	<p>-----NOTES-----</p> <p>1. Neutron detectors are excluded.</p> <p>2. For Functions 3 and 4, the sensor response time may be assumed to be the design sensor response time.</p> <p>-----</p> <p>Verify the RPS RESPONSE TIME is within limits.</p>	In accordance with the Surveillance Frequency Control Program
SR 3.3.1.1.18	Adjust recirculation drive flow to conform to reactor core flow.	Once within 7 days after reaching equilibrium conditions following refueling outage

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(CAC NO. MF7207) DATED MARCH 19, 2020

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