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SECTION 1.0

INTRODUCTION

To assure that adequate measures are available to protect the health and safety of the general public in the Coffey County area in the event of an incident at the Wolf Creek Generating Station (WCGS), it is necessary to conduct an annual emergency preparedness Exercise which requires the participation of the Emergency Response Organization (ERO) and State and County emergency response capabilities. Federal agencies will evaluate and critique the annual Exercise to assure proper response in the event of an actual emergency at WCGS.

Exercise participants do not have prior knowledge of the accident scenario or of the starting time of the Exercise. The Exercise should demonstrate that those individuals and agencies who are assigned responsibilities in a radiological emergency are adequately trained to perform according to current plans and procedures. Furthermore, this Exercise will provide training for emergency response personnel, and identify any potential problem areas in the overall emergency response system.

This manual has been prepared to assist the Exercise controllers, evaluators, and observers in the conduct and evaluation of the Exercise. It contains all of the information and data necessary to properly conduct this Exercise in an efficient and coordinated manner, and is organized as follows:

<u>Section 2.0 Objectives and Guidelines</u>

This section defines the Exercise objectives for the licensee, State of Kansas, and Coffey County, and sets forth guidelines for the conduct of the Exercise to meet those objectives.

<u>Section 3.0 Scenario and Timeline</u>

This section describes the postulated sequence of events occurring at WCGS which requires the ERO to respond.

<u>Section 4.0 Controller Messages</u>

This section contains the Exercise messages used to control the development of the Exercise scenario.

<u>Section 5.0 Plant Data</u>

This section contains information concerning designated plant parameters. These parameters are updated every 15 minutes throughout the Exercise. To assure that adequate operational data is available in the event of a simulator failure, graphs of plant parameter information have been included in this section.

<u>Section 6.0 Meteorological Data</u>

This section contains information about the meteorological conditions in the Coffey County area during the conduct of the Exercise.

Section 7.0 Onsite Radiological Data

This section contains information about radiological conditions at the various onsite monitoring locations. Also included in this section is information concerning primary and secondary systems radiochemistry, containment atmosphere radiochemistry, and in-plant radiation levels.

Section 8.0 Offsite Radiological Data

This section contains information about radiological conditions at the various offsite monitoring locations.

Section 9.0 'Controllers' Instructions

This section provides general instructions to the Exercise controllers in the conduct of the Exercise.

Section 10.0 Evaluators' Instructions

This section provides general instructions and criteria for evaluating the responses of the Exercise participants and the progress of the Exercise. Evaluator's Log Sheets are also provided in this section.

Copies of this manual will be provided to Exercise controllers, evaluators, and selected observers prior to the Exercise. Following the Exercise, copies of this manual may be distributed to key Exercise participants.

SECTION 2.0

OBJECTIVES AND GUIDELINES

OBJECTIVES

Specific objectives to be achieved during the Exercise originate from discussions among the Wolf Creek Nuclear Operating Corporation (WCNOC), State of Kansas, Coffey County, Nuclear Regulatory Commission - Region IV (NRC), and the Federal Emergency Management Agency - Region VII (FEMA).

The listing of objectives is divided into three groups: WCNOC, State, and local. The WCNOC objectives were obtained from NRC Inspection Procedure 82302. Group I objectives are to be demonstrated during each annual Exercise. Group II objectives are to be demonstrated at least once during a 5 year period. Like the State and local objectives, the Licensee's objectives are segregated by the facility or group which will demonstrate the objective.

Each objective for each WCNOC and local facility/team will be designated with a "na", "--" or a "D". Certain parts of some objectives have been enclosed in brackets. This is to indicate it is a two part objective. If both parts of the objective are to be demonstrated, a single "D" will appear in the facility/team column. Likewise, if both parts are not applicable (na) or not to be demonstrated (--) at this exercise, then only a single "na" or "--" will be present in the facility/team column. However, if only the bracketed part of the objective is to be demonstrated (D) and the part outside the brackets is not applicable (na), then in the facility/team column a "D" would appear in brackets and a "na" would appear without brackets.

A list of abbreviations used in assigning responsibilities for objectives follows.

LIST OF ABBREVIATIONS

Amb	Ambulance Service
CEOC	Coffey County Emergency Operations Center
CR	Control Room
CRBD	Coffey County Road and Bridge Department
D	Objective to be Demonstrated
DA&FT COORD	Dose Assessment and Field Team Coordination
EOF	Emergency Operations Facility
EOFS	Emergency Operations Facility, State Dose Assessment Group
Hosp	Hospital
IC	Information Clearinghouse
JRMT	Joint Radiological Monitoring Team
KCPL GO	Kansas City Power and Light General Office
MED	Medical
MI	Media Inquiry
MM	Media Monitoring
MRC	Media Release Center
na	Objective Not Applicable to Facility/Function
OMT	Offsite Monitoring Team
OSC	Operations Support Center
PASS/ERDC	Post-Accident Sampling System/Emergency Repair-Damage Control Team
PC	Public Concern
Rad Lab	Radiation Laboratory, Kansas Department of Health and Environment
RCC	Reception and Care Center
School	Coffey County Schools
Sec	Security (WCNOC)

SEOC	State Emergency Operations Center
SFSA	State Forward Staging Area
TBD	To Be Determined
TSC	Technical Support Center
--	Objective Not to be Demonstrated

1993 OBJECTIVES

WCNOC

Group I OBJECTIVES (every year)	CR	SEC	TSC	PASS/ ERDC	OSC	OMT/ JRMT	EOF	IC/ MRC	MI	PC	MM	KCPL GO	MED
ACCIDENT DETECTION AND ASSESSMENT													
1. Demonstrate the ability to: evaluate the sequence of events; keep each facility aware of operational history and status; use all available resources (e.g. offsite, ERDC, and Engineering teams) to support accident identification and mitigation.	D	na	D	na	D	na	D	na	na	na	na	na	na
EMERGENCY CLASSIFICATION													
2. Demonstrate the ability to classify emergencies promptly and according to emergency action levels.	D	na	D	na	na	na	D	na	na	na	na	na	na
NOTIFICATION OF ONSITE AND OFFSITE EMERGENCY RESPONDERS													
3. Demonstrate the ability to notify the State, County, and NRC using the proper forms within the proper time constraints; and to notify plant personnel by sounding the emergency alarms and making appropriate announcements.	D	na	D	na	na	na	D	na	na	na	na	na	na
COMMUNICATIONS													
4. Demonstrate the ability to communicate among facilities and outside groups (e.g. State, County, NRC Bethesda headquarters), and with emergency personnel in the field.	D	D	D	D	D	D	D	D	D	D	D	D	--

1993 OBJECTIVES

WCNOC

Group I OBJECTIVES (con't) (every year)	CR	SEC	TSC	PASS/ ERDC	OSC	OMT/ JRMT	EOF	IC/ MRC	MI	PC	MM	KCPL GO	MED
RADIOLOGICAL EXPOSURE CONTROL													
5. Demonstrate the ability to continuously monitor and control radiation exposure to emergency workers within the facilities and in the field.	D	na	D	D	D	D	D	na	na	na	na	na	--
PROTECTIVE ACTION RECOMMENDATIONS													
6. Demonstrate the ability to make protective action recommendations based on all available data, and to emergency workers within the facilities and in the field.	D	na	D	na	na	na	D	na	na	na	na	na	na
STAFF AUGMENTATION													
7. Demonstrate the ability to augment the Control Room staff.	na	D	D	D	D	D	D	D	D	D	D	D	na
SHIFT STAFFING													
8. Demonstrate the ability to maintain a complete shift complement throughout the emergency.	D	na	D	na	D	na	D	D	D	D	D	D	na
Group II OBJECTIVES (once every five years)													
ACTIVATION OF EMERGENCY NEWS CENTER (JOINT INFORMATION CENTER)													
1. Demonstrate the ability to develop and disseminate clear, accurate, and timely information to the news media.	na	na	na	na	na	na	na	D	na	na	na	na	na
RUMOR CONTROL													
2. Demonstrate the ability to establish and operate rumor control in a coordinated and timely manner.	na	na	na	na	na	na	na	D	D	D	D	D	na

1993 OBJECTIVES

STATE

OBJECTIVES	SEOC	EOFS: EOF	EOFS: DA&FT COORD	JRMT	SFSA	IC	MRC	RAD LAB
MOBILIZATION OF EMERGENCY PERSONNEL								
1. Demonstrate the capability to alert and fully mobilize personnel for both emergency facilities and field operations. Demonstrate the capability to activate and staff emergency facilities for emergency operations.	D	D	D	D	D	D	D	na
FACILITIES - EQUIPMENT, DISPLAYS, AND WORK ENVIRONMENT								
2. Demonstrate the adequacy of facilities, equipment, displays, and other materials to support emergency operations.	D	D	D	na	na	D	D	na
DIRECTION AND CONTROL								
3. Demonstrate the capability to direct and control emergency operations.	D	D	D	na	na	na	na	na
COMMUNICATIONS								
4. Demonstrate the capability to communicate with all appropriate emergency personnel at facilities and in the field.	D	D	D	D	D	D	D	na
EMERGENCY WORKER EXPOSURE CONTROL								
5. Demonstrate the capability to continuously monitor and control radiation exposure to emergency workers.	na	D	D	D	D	na	na	--

1993 OBJECTIVES

STATE

OBJECTIVES	SEOC	EOFS		JRMT	SFSA	IC	MRC	RAD LAB
		EOF	DA&FT COORD					
FIELD RADIOLOGICAL MONITORING - AMBIENT RADIATION MONITORING								
6. Demonstrate the appropriate use of equipment and procedures for determining field radiation measurements.	na	na	na	D	na	na	na	na
PLUME DOSE PROJECTION								
7. Demonstrate the capability to develop dose projections and protective action recommendations regarding evacuation and sheltering.	na	na	D	na	na	na	na	na
FIELD RADIOLOGICAL MONITORING - AIRBORNE RADIOIODINE AND PARTICULATE ACTIVITY MONITORING								
8. Demonstrate the appropriate use of equipment and procedures for the measurement of airborne radioiodine concentrations as low as 10 E-7 (.0000001) microcuries per cubic centimeter in the presence of noble gases and obtain samples of particulate activity in the airborne plume.	na	na	na	D	na	na	na	na
PLUME PROTECTIVE ACTION DECISION MAKING								
9. Demonstrate the capability to make timely and appropriate protective action decisions (PAD).	D	na	D	na	na	na	na	na
ALERT AND NOTIFICATION								
10. Demonstrate the capability to promptly alert and notify the public within the 10-mile plume pathway emergency planning zone (EPZ) and disseminate instructional messages to the public on the basis of decisions by appropriate State or local officials.	D	na	na	na	na	na	na	na

1993 OBJECTIVES

STATE

OBJECTIVES	SEOC	EOFS		JRMT	SFSA	IC	MRC	RAD LAB
		EOF	DA&FT COORD					
PUBLIC INSTRUCTIONS AND EMERGENCY INFORMATION								
11. Demonstrate the capability to coordinate the formulation and dissemination of accurate information and instructions to the public.	D	na	na	na	na	na	na	na
EMERGENCY INFORMATION - MEDIA								
12. Demonstrate the capability to coordinate the development and dissemination of clear, accurate, and timely information to the news media.	na	na	na	na	na	na	D	na
EMERGENCY INFORMATION - RUMOR CONTROL								
13. Demonstrate the capability to establish and operate rumor control in a coordinated and timely manner.	na	na	na	na	na	D	na	na
IMPLEMENTATION OF PROTECTIVE ACTIONS - USE OF KI FOR EMERGENCY WORKERS, INSTITUTIONALIZED INDIVIDUALS, AND THE GENERAL PUBLIC								
14. Demonstrate the capability and resources to implement potassium iodide (KI) protective actions for emergency workers, institutionalized individuals, and, if the State plan specifies, the general public.*	na	D	D	D	D	na	na	na
IMPLEMENTATION OF PROTECTIVE ACTIONS - SPECIAL POPULATIONS								
15. Demonstrate the capability and resources necessary to implement appropriate protective actions for special populations.	na	na	na	na	na	na	na	na

*The State of Kansas does not recommend the use of KI for the general public (State Plan Section 1.3.1 Tab J)

1993 OBJECTIVES

STATE

OBJECTIVES	SEOC	EOFS		JRMT	SFSA	IC	MRC	RAD LAB
		EOF	DA&FT COORD					
IMPLEMENTATION OF PROTECTIVE ACTIONS - SCHOOLS								
16. Demonstrate the capability and resources necessary to implement protective actions for school children within the plume pathway emergency planning zone (EPZ).	na	na	na	na	na	na	na	na
TRAFFIC AND ACCESS CONTROL								
17. Demonstrate the organizational capability and resources necessary to control evacuation traffic flow and to control access to evacuated and sheltered areas.	D	na	na	na	D	na	na	na
RECEPTION CENTER - MONITORING, DECONTAMINATION, AND REGISTRATION								
18. Demonstrate the adequacy of procedures, facilities, equipment, and personnel for the radiological monitoring, decontamination, and registration of evacuees.	na	na	na	na	na	na	na	na
CONGREGATE CARE								
19. Demonstrate the adequacy of facilities, equipment, supplies, personnel, and procedures for congregate care of evacuees.	na	na	na	na	na	na	na	na
MEDICAL SERVICES - TRANSPORTATION								
20. Demonstrate the adequacy of vehicles, equipment, procedures and personnel for transporting contaminated, injured or exposed individuals.	na	na	na	na	na	na	na	na

1993 OBJECTIVES

STATE

OBJECTIVES	SEOC	EOFS		JRMT	SFSA	IC	MRC	RAD LAB
		EOF	DA&FT COORD					
MEDICAL SERVICES - FACILITIES								
21. Demonstrate the adequacy of the equipment, procedures, supplies, and personnel of medical facilities responsible for treatment of contaminated, injured, or exposed individuals.	na	na	na	na	na	na	na	na
EMERGENCY WORKERS, EQUIPMENT, AND VEHICLES - MONITORING AND DECONTAMINATION								
22. Demonstrate the adequacy of procedures for the monitoring and decontamination of emergency workers, equipment, and vehicles.	na	na	na	na	na	na	na	na
SUPPLEMENTARY ASSISTANCE (FEDERAL/OTHER)								
23. Demonstrate the capability to identify the need for external assistance and to request such assistance from Federal or other support organizations.	--	na	na	na	na	na	na	na
POST-EMERGENCY SAMPLING								
24. Demonstrate the use of equipment and procedures for the collection and transportation of samples from areas that received deposition from the airborne plume.	na	na	na	--	na	na	na	na
LABORATORY OPERATIONS								
25. Demonstrate laboratory operations and procedures for measuring and analyzing samples.	na	na	na	na	na	na	na	--

1993 OBJECTIVES

STATE

OBJECTIVES	SEOC	EOFS		JRMT	SFSA	IC	MRC	RAD LAB
		EOF	DA&FT COORD					
INGESTION EXPOSURE PATHWAY - DOSE PROJECTION AND PROTECTIVE ACTION DECISION MAKING								
26. Demonstrate the capability [to project dose to the public] for the ingestion exposure pathway and to recommend protective actions.	[na] --	na	[--] --	na	na	na	na	na
INGESTION EXPOSURE PATHWAY - PROTECTIVE ACTION IMPLEMENTATION								
27. Demonstrate the capability to implement protective actions for the ingestion exposure pathway.	--	na	na	na	na	na	na	na
RECOVERY, REENTRY AND RETURN - DECISION MAKING								
28. Demonstrate the capability to develop decisions on relocation, re-entry, and return.	--	na	--	na	na	na	na	na
RELOCATION, RE-ENTRY, AND RETURN - IMPLEMENTATION								
29. Demonstrate the capability to implement appropriate measures for relocation, re-entry, and return.	--	na	na	na	--	na	na	na
CONTINUOUS, 24-HOUR STAFFING								
30. Demonstrate the capability to maintain staffing on a continuous, 24-hour basis through an actual shift change.	--	D	--	--	D	D	na	--
OFFSITE SUPPORT FOR THE EVACUATION OF ONSITE PERSONNEL								
31. Demonstrate the capability to provide offsite support for the evacuation of onsite personnel.	na	na	na	na	na	na	na	na

1993 OBJECTIVES

STATE

OBJECTIVES	SEOC	EOFS		JRMT	SFSA	IC	MRC	RAD LAB
		EOF	DA&FT COORD					
UNANNOUNCED EXERCISE OR DRILL								
32. Demonstrate the capability to carry out emergency response functions in an unannounced exercise or drill.	--	--	--	--	na	--	--	na
OFF-HOURS EXERCISE OR DRILL								
33. Demonstrate the capability to carry out emergency response functions during an off-hours exercise or drill.	--	--	--	--	na	--	--	na

1993 OBJECTIVES

LOCAL

OBJECTIVES	CEOC	JRMT	IC	MRC	CRBD	School	Amb	Hosp	Reception/Care Centers	
									Lyon	Anderson
MOBILIZATION OF EMERGENCY PERSONNEL										
1. Demonstrate the capability to alert and fully mobilize personnel for both emergency facilities and field operations. Demonstrate the capability to activate and staff emergency facilities for emergency operations.	D	D	D	D	D	na	na	na	D	--
FACILITIES - EQUIPMENT, DISPLAYS, AND WORK ENVIRONMENT										
2. Demonstrate the adequacy of facilities, equipment, displays, and other materials to support emergency operations.	D	na	D	D	D	na	na	na	na	na
DIRECTION AND CONTROL										
3. Demonstrate the capability to direct and control emergency operations.	D	na	na	na	D	na	na	na	D	--
COMMUNICATIONS										
4. Demonstrate the capability to communicate with all appropriate emergency personnal at facilities and in the field.	D	D	D	D	D	D	na	na	D	--
EMERGENCY WORKER EXPOSURE CONTROL										
5. Demonstrate the capability to continuously monitor and control radiation exposure to emergency workers.	D	D	na	na	D	D	D	D	D	--

1993 OBJECTIVES

LOCAL

OBJECTIVES	CEOC	JRMT	IC	MRC	CRBD	School	Amb	Hosp	Reception/Care Centers	
									Lyon	Anderson
FIELD RADIOLOGICAL MONITORING - AMBIENT RADIATION MONITORING										
6. Demonstrate the appropriate use of equipment and procedures for determining field radiation measurements.	na	D	na	na	na	na	na	na	na	na
PLUME DOSE PROJECTION										
7. Demonstrate the capability to develop dose projections and protective action recommendations regarding evacuation and sheltering.	na	na	na	na	na	na	na	na	na	na
FIELD RADIOLOGICAL MONITORING - AIRBORNE RADIOIODINE AND PARTICULATE ACTIVITY MONITORING										
8. Demonstrate the appropriate use of equipment and procedures for the measurement of airborne radioiodine concentrations as low as 10 E-7 (.0000001) microcuries per cubic centimeter in the presence of noble gases and obtain samples of particulate activity in the airborne plume.	na	D	na	na	na	na	na	na	na	na
PLUME PROTECTIVE ACTION DECISION MAKING										
9. Demonstrate the capability to make timely and appropriate protective action decisions (PAD).	na	na	na	na	na	na	na	na	na	na
ALERT AND NOTIFICATION										
10. Demonstrate the capability to promptly alert and notify the public within the 10-mile plume pathway emergency planning zone (EPZ) and disseminate instructional messages to the public on the basis of decisions by appropriate State or local officials.	D	na	na	na	na	na	na	na	na	na

1993 OBJECTIVES

LOCAL

OBJECTIVES	CEOC	JRMT	IC	MRC	CRBD	School	Amb	Hosp	Reception/Care Centers Lyon	Reception/Care Centers Anderson
PUBLIC INSTRUCTIONS AND EMERGENCY INFORMATION										
11. Demonstrate the capability to coordinate the formulation and dissemination of accurate information and instructions to the public.	D	na	na	na	na	na	na	na	na	na
EMERGENCY INFORMATION - MEDIA										
12. Demonstrate the capability to coordinate the development and dissemination of clear, accurate, and timely information to the news media.	na	na	na	D	na	na	na	na	na	na
EMERGENCY INFORMATION - RUMOR CONTROL										
13. Demonstrate the capability to establish and operate rumor control in a coordinated and timely manner.	na	na	D	na	na	na	na	na	na	na
IMPLEMENTATION OF PROTECTIVE ACTIONS - USE OF KI FOR EMERGENCY WORKERS, INSTITUTIONALIZED INDIVIDUALS, AND THE GENERAL PUBLIC										
14. Demonstrate the capability and resources to implement potassium iodide (KI) protective actions for emergency workers, institutionalized individuals, and, if the State plan specifies, the general public.*	D	D	na	na	D	D	D	D	na	na
IMPLEMENTATION OF PROTECTIVE ACTIONS - SPECIAL POPULATIONS										
15. Demonstrate the capability and resources necessary to implement appropriate protective actions for special populations.	D	na	na	na	D	na	na	na	na	na

*The State of Kansas does not recommend the use of KI for the general public (State Plan Section 1.3.1 Tab J)

1993 OBJECTIVES

LOCAL

OBJECTIVES	CEOC	JRMT	IC	MRC	CRBD	School	Amb	Hosp	Reception/Care Centers: Lyon	Reception/Care Centers: Anderson
IMPLEMENTATION OF PROTECTIVE ACTIONS - SCHOOLS										
16. Demonstrate the capability and resources necessary to implement protective actions for school children within the plume pathway emergency planning zone (EPZ).	D	na	na	na	na	D	na	na	na	na
TRAFFIC AND ACCESS CONTROL										
17. Demonstrate the organizational capability and resources necessary to control evacuation traffic flow and to control access to evacuated and sheltered areas.	D	na	na	na	D	na	na	na	na	na
RECEPTION CENTER - MONITORING, DECONTAMINATION, AND REGISTRATION										
18. Demonstrate the adequacy of procedures, facilities, equipment, and personnel for the radiological monitoring, decontamination, and registration of evacuees.	na	na	na	na	na	na	na	na	D	--
CONGREGATE CARE										
19. Demonstrate the adequacy of facilities, equipment, supplies personnel, and procedures for congregate care of evacuees.	na	na	na	na	na	na	na	na	D	--
MEDICAL SERVICES - TRANSPORTATION										
20. Demonstrate the adequacy of vehicles, equipment, procedures and personnel for transporting contaminated, injured or exposed individuals.	na	na	na	na	na	na	D	na	na	na

1993 OBJECTIVES

LOCAL

OBJECTIVES	CEOC	JRMT	IC	MRC	CRBD	School	Amb	Hosp	Reception/Care Centers	
									Lyon	Anderson
MEDICAL SERVICES - FACILITIES										
21. Demonstrate the adequacy of the equipment, procedures, supplies, and personnel of medical facilities responsible for treatment of contaminated, injured, or exposed individuals.	na	na	na	na	na	na	na	D	na	na
EMERGENCY WORKERS, EQUIPMENT, AND VEHICLES - MONITORING AND DECONTAMINATION										
22. Demonstrate the adequacy of procedures for the monitoring and decontamination of emergency workers, equipment, and vehicles.	na	na	na	na	--	na	na	na	D	--
SUPPLEMENTARY ASSISTANCE (FEDERAL/OTHER)										
23. Demonstrate the capability to identify the need for external assistance and to request such assistance from Federal or other support organizations.	na	na	na	na	na	na	na	na	na	na
POST-EMERGENCY SAMPLING										
24. Demonstrate the use of equipment and procedures for the collection and transportation of samples from areas that received deposition from the airborne plume.	na	na	na	na	na	na	na	na	na	na
LABORATORY OPERATIONS										
25. Demonstrate laboratory operations and procedures for measuring and analyzing samples.	na	na	na	na	na	na	na	na	na	na

1993 OBJECTIVES

LOCAL

OBJECTIVES	CEOC	JRMT	IC	MRC	CRBD	School	Amb	Hosp	Reception/Care Centers	
									Lyon	Anderson
INGESTION EXPOSURE PATHWAY - DOSE PROJECTION AND PROTECTIVE ACTION DECISION MAKING										
26. Demonstrate the capability [to project dose to the public] for the ingestion exposure pathway and to recommend protective actions.	[na] --	na	na	na	na	na	na	na	na	na
INGESTION EXPOSURE PATHWAY - PROTECTIVE ACTION IMPLEMENTATION										
27. Demonstrate the capability to implement protective actions for the ingestion exposure pathway.	na	na	na	na	na	na	na	na	na	na
RECOVERY, REENTRY AND RETURN - DECISION MAKING										
28. Demonstrate the capability to develop decisions on relocation, re-entry, and return.	na	na	na	na	na	na	na	na	na	na
RELOCATION, RE-ENTRY, AND RETURN - IMPLEMENTATION										
29. Demonstrate the capability to implement appropriate measures for relocation, re-entry, and return.	--	na	na	na	na	na	na	na	na	na
CONTINUOUS, 24-HOUR STAFFING										
30. Demonstrate the capability to maintain staffing on a continuous, 24-hour basis through an actual shift change.	--	--	D	na	--	na	na	na	D	--
OFFSITE SUPPORT FOR THE EVACUATION OF ONSITE PERSONNEL										
31. Demonstrate the capability to provide offsite support for the evacuation of onsite personnel.	D	na	na	na	na	na	na	na	na	na

1993 OBJECTIVES

LOCAL

OBJECTIVES	CEOC	JRMT	IC	MRC	CRBD	School	Amb	Hosp	Reception/Care Centers	
									Lyon	Anderson
UNANNOUNCED EXERCISE OR DRILL										
32. Demonstrate the capability to carry out emergency response functions in an unannounced exercise or drill.	--	--	--	--	na	na	na	na	na	na
OFF-HOURS EXERCISE OR DRILL										
33. Demonstrate the capability to carry out emergency response functions during an off-hours exercise or drill.	--	--	--	--	na	na	na	na	na	na

GUIDELINES

These guidelines define the participants' extent of play in demonstrating the previously listed objectives.

A. The Exercise will be conducted September 1, 1993.

B. Participants will not have prior knowledge of the scenario or time of the Exercise. However, they will receive a briefing on the guidelines for the Exercise.

C. There will be pre-staging of licensee participants in the Wolf Creek Generating Station (WCGS) Simulator and outside the Information Clearinghouse (IC) and Media Release Center (MRC) in Topeka. The Licensee IC/MRC particpants will only be allowed into those facilities after simulating an appropriate travel time (~75 minutes following the declaration of an Alert). Station operators will also be pre-staged in the computer room next to the actual Control Room on site.

D. Personnel will be notified of emergency conditions through methods normally employed in making notifications (e.g. GAI-tronics, callout or Security).

E. The following emergency response facilities/functions will be participating in the Exercise:

1. Licensee - Wolf Creek Nuclear Operating Corp.

   a. Control Room (CR)
   b. Technical Support Center (TSC)
   c. Operations Support Center (OSC)
   d. Emergency Repair/Damage Control (ERDC) Teams
   e. Onsite Survey Teams (OSTs)
   f. Security (Sec)
   g. Offsite Monitoring Teams (OMTs)
   h. Emergency Operations Facility (EOF)
   i. Media Release Center (MRC)
   k. General Office - KCP&L GO
   l. Media Inquiry/Public Concern/Media Monitoring - Wichita Office (MI/PC/MM)

---

2. State - Kansas

   a. Emergency Operations Center (SEOC) Key Personnel
   b. Emergency Operations Facility (EOFS)
      1. Radiological Assessment
      2. Emergency Preparedness
   c. Information Clearinghouse (IC)
   d. Media Release Center (MRC)
   e. State Forward Staging Area (SFSA)

3. Local

   a. Coffey County Emergency
      1. Operations Center (CEOC)
      2. County Road and Bridge Department (CRBD)
      3. Information Clearinghouse (IC)
      4. Media Release Center (MRC)
      5. Coffey County Ambulance Service* (Amb)
      6. Coffey County Hospital* (Hosp)
      7. Waverly School* (School)

   b. Lyon County
      1. Reception and Care Center* (RCC)

F. The CR will be simulated from the plant simulator. A full shift complement of operators will staff the simulator.

G. Play will be driven from the plant simulator. A full shift complement of operators will staff the simulator.

H. All communications involving the CR will be duplicated in the simulator, except for the Emergency Notification System (ENS) dedicated line and the State/County radios. A commercial telephone will be used in the simulator for the ENS.

I. Fire protection panels (KC008) are not duplicated on the plant simulator and will be simulated.

J. Operational and meteorological initial conditions will be established prior to the start of the exercise and will be distributed to those players who would, under actual conditions, be aware of this information.

K. Participation by onsite personnel directly involved with emergency response shall be carried out to the fullest extent possible without affecting plant operations or plant safety. Dispatch of the following teams into the power block will occur as required:

   1. Onsite Survey Teams (OST)
   2. Emergency Repair and Damage Control Teams (ERDC)
   3. Search and Rescue Teams
   4. First Aid Teams
   5. Security Teams
   6. Fire Brigade

* These facilities will demonstrate their objectives out-of-sequence from the Exercise.

GUIDELINES

L. The phrase, "This is a drill", will begin and end all radio and telephone transmissions performed in response to scenario events.

M. If the scenario requires that any personnel working for an organization not particpating be contacted, they shall be contacted for the purpose of checking communications only.

SECTION 3.0

SCENARIO AND TIMELINE

SCENARIO

A summary of the scenario events is provided in the following subsection.

Operational events will be conducted as written. Other events may vary from those written according to the actions of the players.

Operations

The plant has been operating normally at 100% full power for the last 14 days on the sixth cycle of a middle of life core. The WCGS electrical power system is in a very high demand situation.

Maintenance work in progress is as follows:

1. The residual heat removal (RHR) system "B" train pump and heat exchanger have been removed from service for repair of a tube to shell side leak. Work has progressed to the point of reassembling the shell and associated component cooling water (CCW) lines. Completion of repairs is estimated to take 20 hours of the 32 hours remaining before entering a technical specification action level.

2. SI Pump tagged out due to motor fault. The motor is not available locally and is being shipped in. It is not able to be repaired during the Exercise.

3. The concrete hatch in the roof of the 2026' level of the auxiliary building (aux bldg) has been removed to permit the removal of the RHR "B" train heat exchanger shell. A crane was used to remove the hatch.

4. Auxiliary building (aux bldg) ventilation control is being provided by the fuel/aux bldg normal exhaust system. A slight negative pressure is being maintained in the aux bldg to ensure outside air flow is brought into the aux bldg through the opened roof access.

5. Radiological controls at the opened roof access are being provided by a health physics technician. Security personnel are providing access controls.

Meteorological

It is an overcast and humid day with winds out of the east-northeast at 12 mph. Daytime temperatures have been ranging from the low 80's to the upper 80's.

INITIAL CONDITIONS
(Page 2 of 2)

Radiochemistry

Following is the latest isotopic analysis performed on the reactor coolant system (RCS). The analysis was performed at 0730 the day of the Exercise.

Nuclide	Activity (uCi/cc)
Kr-83	3.83E-03
Kr-85m	1.17E-02
Kr-85	4.50E-04
Kr-87	2.12E-02
Kr-88	2.90E-02
Xe-133m	1.35E-03
Xe-133	5.11E-02
Xe-135m	1.42E-02
Xe-135	4.88E-02
Xe-138	4.34E-02
I-131	2.59E-03
I-132	3.95E-03
I-133	5.81E-03
I-134	6.78E-03
I-135	5.27E-03
RB-88	1.82E-03
CE-144	9.02E-08
TE-132	2.44E-04
Cs-134	2.26E-04
Cs-137	1.69E-04
Cs-138	1.64E-02
LA-140	1.35E-06
LA-142	3.01E-07
BA-140	2.07E-06

RCS I-131 Equiv. is 4.86E-03

SCENARIO

NARRATIVE SUMMARY

This scenario is based on a loss of coolant accident, reduced reactor coolant system makeup capacity, fuel failure, and a loss of containment building integrity following a hydrogen burn.

Initial conditions establish the plant operating normally at 100% full power. Demand for electricity in the area is very high.

The residual heat removal (RHR) system "B" train has been removed from service for repair of a tube to shell side leak in the heat exchanger. Repairs are estimated to be 20 hours away from completion.

The initiating event for the scenario occurs where the operators receive indications of a fire in the switchgear room no. 1. Although the fire is extinguished within minutes by the halon fire protection system, 4160 volt bus NB01 experiences damage and deenergizes. Diesel generator "A" starts but is unable to reenergize the damaged bus. A Notification of an Unusual Event is declared based on plant shutdown per Tech Spec. 3.0.3.

Offsite notifications should be made by the Shift Clerk per EPP 01-3.1. Site personnel are notified of the situation through the activation of the plant emergency alarm and the reading of the message found in EPP 01-2.2.

Approximately thirty minutes later, a 1000 gpm LOCA occurs at normal operating pressures. The exact location of the LOCA is unidentified. Shortly after the LOCA, the reactor trips and safety injection occurs. An RCS cooldown is initiated and pressure drops, therefore the leak rate decreases. Adequate reactor coolant makeup water is supplied by the centrifugal charging pump (CCP) "B".

An Alert is declared based on the loss of the reactor coolant system boundary. Offsite notifications are made and site personnel are notified of the situation. Accountability of personnel (simulated) inside the Protected Area Boundary (PAB) also occurs at this time. Thirty minutes after the Alert is declared, the TSC is activated. Personnel will begin to staff the EOF, Wichita Rumor Control, KCP&L Rumor Control, and the Topeka Information Clearinghouse (IC).

Approximately an hour later, the size of the LOCA increases and the makeup capacity of CCP "B" is exceeded. The SI accumulators discharge and the reactor vessel level indicating system (RVLIS) indicates water levels below the top of the core. Core exit thermocouple indications are >1200^{o} F.

A Site Area Emergency should be declared based on inadequate core cooling conditions but containment closure maintained. Offsite notifications are made, and site personnel are notified. At approximately the same time, the Rumor Control offices and the IC should be activated. Personnel will continue to staff the EOF, and the County and State EOCs should declare their facilities activated. Offsite protective action recommendations should include evacuation of John Redmond Reservoir (JRR).

Over the next two hours, core exit thermocouples continue to increase to approximately 2900° F, and Containment Building (ctmt bldg) hydrogen levels exceed 4% as a zircaloy-water reaction is sustained by the elevated core temperatures. After approximately 4 hours into the Exercise, a spontaneous hydrogen burn later causes ctmt bldg pressure to spike to 47 pounds per square inch gauge (psig). The ctmt bldg breaches at a failure of the ctmt bldg mini purge exhaust penetration. A General Emergency is declared based on core uncovery with the RCS not intact and ctmt closure not set.

A filtered release path is provided to the environment through the ctmt bldg breach to the plant unit vent. The release is monitored and consists almost entirely of noble gases. Offsite protective actions should include, as a minimum, evacuation of subzone "CTR" and evacuation 2-5 miles downwind.

Conditions improve as the RHR system "B" train is energized and the reactor vessel is refilled. Ctmt bldg pressure continues dropping as it equalizes with the pressure in the auxiliary building and the release is terminated after approximately 1.5 hours.

Repairs are subsequently completed on 4160 volt bus NB01 thereby returning the emergency core cooling system "A" train to operability.

TIMELINE

A timeline summary and chart indicating times of significant scenario is provided in the following subsection.

Operational events will occur as stated in the timeline, however, times vary slightly in order to provide players "freedom of play".

TIMELINE

SUMMARY

TIME		PLANT EVENT SUMMARY
0730 (H-00:30)		- Initial conditions provided to the Shift Supervisor, the DED, AEC, DEC, REC, TSC, MEC, and the DSC Supervisor; plus the DEM, TRM, RAM and ARM.
0800 (H+00:00)		- Field Exercise Activities begin.
0815 (H+00:15)	NUE	- Fire in switchgear room no. 1.
		- Automatic activation of halon fire protection system.
		- Deenergization of 4160 volt bus NB01.
		- Loss of emergency core cooling system (ECCS) "A" train. CTMT isolation damper GT HZ-12 fails open.
0820 (H+00:20)		- Fire in switchgear room no. 1 extinguished by halon fire protection system.
0845 (H+00:45)	**ALERT**	- Unidentified loss of coolant accident (1000 gpm) (LOCA) at normal operating pressures.
		- Makeup water adequate via centrifugal charging pump (CCP) "B" due to decreasing RCS pressure causing decreasing leak rate.
0847 (H+00:47)		- Reactor/turbine trip.
0856 (H+00:56)		- Ctmt bldg pressure steadily increases. >5 psig in ctmt.
0945 (H+01:45)		- RCS leak rate rapidly increases.
		- Makeup capacity exceeded for CCP "B".
0955 (H+01:55)		- Accumulator safety injection system discharges.

NOTE: Times are approximations based on previous simulator runs.

SUMMARY

TIME		PLANT EVENT SUMMARY
1000 (H+2:00)	SAE	- Reactor vessel level indication system (RVLIS) indicates water level below top of core. Core exit thermocouples increase to >1200^{o}F.
1040 (H+2:40)		- Hydrogen concentration levels inside the ctmt bldg increase.
		- Core exit thermocouple temperatures continue to increase.
1100 (H+03:00)		- Fuel damage confirmed by post-accident sampling system (PASS).
1200 (H+04:00)		- Core exit thermocouples indicate temperatures consistent with a zircaloy-water reaction (>2900^{o}F).
		- Hydrogen concentration levels inside the ctmt bldg exceed 4%.
1215 (H+04:15)	GE	- Hydrogen burn within ctmt bldg.
		- Ctmt bldg pressure spikes to 47 psig. The shock from the spike damages ctmt bldg isolation valve GT HZ-11, failing duct work.
		- Ctmt bldg breaches at shutdown purge exhaust ducting penetration GT-V160.
		- Containment spray actuation signal (CSAS) generated. Containment spray pump "B" starts and containment spray pump "B" discharge valve EN HV-12 only opens to a 20% full open position.
1218 (H+04:18)		- Plant unit vent process radiation monitors and aux bldg area radiation monitors (ARM) rapidly trend upscale. Release in progress.

TIMELINE

SUMMARY

TIME	PLANT EVENT SUMMARY
1318 (H+05:18)	- Plant unit vent process radiation monitors indicate release rates trending down.
1340 (H+05:40)	- Repairs completed on RHR Pump "B".
1345 (H+05:45)	- RHR system started from RWST.
	- Reflooding of reactor core accomplished.
1350 (H+05:50)	- Plant unit vent process radiation monitors indicated near normal release rates. Ctmt bldg and aux bldg pressure differential nearly equalized. Release terminated.
1515 (H+07:15)	- Repairs completed on 4160 volt bus NB01.
	- Energization of the ECCS "A" train.
1530 (H+07:30)	- Adequate core cooling initiated.
	- Plant conditions stabilized.
1600 (H+08:00)	- Drill activities terminated. Players' critique begin.

TIMELINE

CHART

Initial Conditions Established.	0730 >	0730 (H-00:30)			
		0800 (H+00:00)	< 0800	Field exercise activities begin.	
Fire in switchgear room No. 1. Deenergization of 4160 volt bus NB01. loss of emergency core cooling system (ECCS) "A" train	0815 >				**NUE** 0815
			< 0820	Fire in switchgear room no. 1 extinguished by automatic (halon) fire protection system.	
Unidentified loss of coolant accident (LOCA). Safety injection initiated. Makeup adequate via Centrifugal Charging Pump (CCP) "B".	0845 >		< 0847	Reactor/turbine trip.	**ALERT** 0845
		0900 (H+01:00)			
			< 0945	Reactor Coolant System (RCS) leak rate rapidly increases. Makeup capacity exceeded CCP "B".	
Reactor Vessel Indication System (RVLIS) indicates water level below top of core. Core exit thermocouples >1200°F.	1000 >	1000 (H+02:00)			**SAE** 1000
		1100 (H+03:00)	< 1100	Post-Accident Sampling System (Pass) confirms fuel damage.	
			< 1130	Hydrogen concentrations inside ctmt bldg continue to increase.	
Hydrogen concentration inside ctmt bldg exceeds 4%.	1200 >	1200 (H+04:00)			
			< 1215	Hydrogen burn within ctmt bldg. Ctmt pressure spike to 47 PSIG. Failure of ctmt bldg purge exhaust penetration. Ctmt bldg breach. Ctmt Spray Actuation Signal (CSAS) generated. Ctmt spray pump "B" pumping 20% full flow due to partial opening of valve EN HV-12.	**GE** 1215
Plant unit vent process radiation and aux bldg Area Radiation Monitors (ARM) rapidly trend upscale. Release in progress.	1218 >				
		1300 (H+05:00)			
			< 1318	Plant unit vent process radiation monitors indicate release rates trending down.	
Repairs completed on RHR Pump "B".	1340 >				
			< 1345	RHRP "B" started on ctmt bldg sump recirculation phase. Reactor core reflooded.	
Plant unit vent process radiation monitors indicate near normal release rates. Release terminated.	1350 >				
		1400 (H+06:00)			
		1500 (H+07:00)			
Repairs completed on 4160 volt bus NB01. Energization of ECCS "A" train successful.	1515 >				
			< 1530	Adequate core cooling initiated. Plant conditions stabilized.	
Drill activities terminated. Players' critique begin.	1600 >	1600 (H+08:00)			

SECTION 4.0

CONTROLLER MESSAGES

ASSIGNMENTS

SCENARIO

Message #	Time	For	From	Location	Assignment
001	0730	SS	CR Lead Cont	CR	
002	0815	SS	CR Lead Cont	CR	
003	0820+	CR Op	CR Lead Cont	Aux Bldg	
004c	0830+	SS	CR Lead Cont	CR	
005	0845+	DED/SS	CR Lead Cont	CR	
006c	0900+	DED/SS	CR Lead Cont	CR	
007c	0945+	DED	TSC Lead Cont	TSC	
008	1100+	State	EOF Lead Cont	EOF	
009	1215+	SEC	Sec Lead Cont	Sec Bldg	
010	1230+	SS	CR Lead Cont	CR	
011c	1245+	DEM	EOF Lead Cont	EOF	
012	1340	SS	CR Lead Cont	CR	
013	1500+	SS	CR Lead Cont	CR	
014	1515	SS	CR Lead Cont	CR	
015	1600	All Part	Fac Lead Conts	All	

ASSIGNMENTS

MINI-SCENARIOS

1	0815	--	Loss of 4160 Volt Bus NB01	
2	0800+	--	RHR System 'B' Train	
3	0800+	--	Aux Bldg Concrete Roof Hatch	
4	0830+	--	Isolation Valve GT HZ-12	
5	1215	--	EN HV-12 Valve	

ASSIGNMENTS

WOLF CREEK PUBLIC INFORMATION OFFICER

Message #	Time	For	From	Location	Assignment
PIO-1	0845	WC PIO	Reporter	Wich	
PIO-2	0850	WC PIO	Reporter	Wich	
PIO-3	0910	WC PIO	Reporter	Wich	
PIO-4	0920	WC PIO	Reporter	Wich	
PIO-5	0945	WC PIO	Reporter	Wich	
PIO-6	1001	WC PIO	Congressional Aide	Wich	

ASSIGNMENTS

MEDIA INQUIRY

Message #	Time	For	From	Location	Assignment
MI-1	0940	Media Inq	Reporter	Wich	
MI-2	0945	Media Inq	Reporter	Wich	
MI-3	0950	Media Inq	Reporter	Wich	
MI-4	1000	Media Inq	Reporter	Wich	
MI-5	1005	Media Inq	Reporter	Wich	
MI-6	1010	Media Inq	Reporter	Wich	
MI-7	1015	Media Inq	Reporter	Wich	
MI-8	1020	Media Inq	Reporter	Wich	
MI-9	1025	Media Inq	Reporter	Wich	
MI-10	1030	Media Inq	Reporter	Wich	
MI-11	1035	Media Inq	Reporter	Wich	
MI-12	1040	Media Inq	Reporter	Wich	
MI-13	1045	Media Inq	Reporter	Wich	

ASSIGNMENTS

MEDIA INQUIRY

Message #	Time	For	From	Location	Assignment
MI-14	1050	Media Inq	Reporter	Wich	
MI-15	1055	Media Inq	Reporter	Wich	
MI-16	1100	Media Inq	Reporter	Wich	
MI-17	1105	Media Inq	Reporter	Wich	
MI-18	1110	Media Inq	Reporter	Wich	
MI-19	1115	Media Inq	Reporter	Wich	
MI-20	1120	Media Inq	Reporter	Wich	
MI-21	1130	Media Inq	Reporter	Wich	
MI-22	1140	Media Inq	Reporter	Wich	
MI-23	1150	Media Inq	Reporter	Wich	
MI-24	1155	Media Inq	Reporter	Wich	
MI-25	1205	Media Inq	Reporter	Wich	
MI-26	1220	Media Inq	Reporter	Wich	
MI-27	1230	Media Inq	Reporter	Wich	
MI-28	1240	Media Inq	Reporter	Wich	
MI-29	1250	Media Inq	Reporter	Wich	
MI-30	1300	Media Inq	Reporter	Wich	
MI-31	1310	Media Inq	Reporter	Wich	
MI-32	1320	Media Inq	Reporter	Wich	
MI-33	1330	Media Inq	Reporter	Wich	
MI-34	1340	Media Inq	Reporter	Wich	
MI-35	1350	Media Inq	Reporter	Wich	
MI-36	1400	Media Inq	Reporter	Wich	
MI-37	1405	Media Inq	Reporter	Wich	
MI-38	1410	Media Inq	Reporter	Wich	
MI-39	1415	Media Inq	Reporter	Wich	
MI-40	1420	Media Inq	Reporter	Wich	
MI-41	1425	Media Inq	Reporter	Wich	
MI-42	1430	Media Inq	Reporter	Wich	
MI-43	1435	Media Inq	Reporter	Wich	
MI-44	1440	Media Inq	Reporter	Wich	
MI-45	1445	Media Inq	Reporter	Wich	
MI-46	1450	Media Inq	Reporter	Wich	
MI-47	1455	Media Inq	Reporter	Wich	
MI-48	1500	Media Inq	Reporter	Wich	
MI-49	1505	Media Inq	Reporter	Wich	
MI-50	1510	Media Inq	Reporter	Wich	
MI-51	1515	Media Inq	Reporter	Wich	
MI-52	1520	Media Inq	Reporter	Wich	
MI-53	1525	Media Inq	Reporter	Wich	
MI-54	1530	Media Inq	Reporter	Wich	
MI-55	1535	Media Inq	Reporter	Wich	
MI-56	1540	Media Inq	Reporter	Wich	
MI-57	1545	Media Inq	Reporter	Wich	

ASSIGNMENTS

PUBLIC CONCERN PHONE TEAM

Message #	Time	For	From	Location	Assignment
PC-1	0940	Pub Con	Citizen	Wich	
PC-2	0945	Pub Con	Citizen	Wich	
PC-3	0950	Pub Con	Citizen	Wich	
PC-4	0955	Pub Con	Citizen	Wich	
PC-5	1000	Pub Con	Citizen	Wich	
PC-6	1005	Pub Con	Citizen	Wich	
PC-7	1010	Pub Con	Citizen	Wich	
PC-8	1015	Pub Con	Citizen	Wich	
PC-9	1020	Pub Con	Citizen	Wich	
PC-10	1030	Pub Con	Citizen	Wich	
PC-11	1040	Pub Con	Citizen	Wich	
PC-12	1045	Pub Con	Citizen	Wich	
PC-13	1050	Pub Con	Citizen	Wich	
PC-14	1055	Pub Con	Citizen	Wich	
PC-15	1100	Pub Con	Citizen	Wich	
PC-16	1105	Pub Con	Citizen	Wich	
PC-17	1110	Pub Con	Citizen	Wich	
PC-18	1120	Pub Con	Citizen	Wich	
PC-19	1130	Pub Con	Citizen	Wich	
PC-20	1140	Pub Con	Citizen	Wich	
PC-21	1150	Pub Con	Citizen	Wich	
PC-22	1200	Pub Con	Citizen	Wich	
PC-23	1210	Pub Con	Citizen	Wich	
PC-24	1220	Pub Con	Citizen	Wich	
PC-25	1230	Pub Con	Citizen	Wich	
PC-26	1240	Pub Con	Citizen	Wich	
PC-27	1250	Pub Con	Citizen	Wich	
PC-28	1300	Pub Con	KG&E Employee	Wich	
PC-29	1310	Pub Con	Citizen	Wich	
PC-30	1320	Pub Con	Citizen	Wich	
PC-31	1330	Pub Con	Citizen	Wich	
PC-32	1340	Pub Con	Citizen	Wich	
PC-33	1350	Pub Con	Citizen	Wich	
PC-34	1400	Pub Con	Citizen	Wich	
PC-35	1405	Pub Con	Citizen	Wich	
PC-36	1410	Pub Con	Citizen	Wich	
PC-37	1415	Pub Con	Citizen	Wich	
PC-38	1420	Pub Con	Citizen	Wich	
PC-39	1425	Pub Con	City of Wichita	Wich	
PC-40	1430	Pub Con	Citizen	Wich	
PC-41	1435	Pub Con	Citizen	Wich	
PC-42	1440	Pub Con	Citizen	Wich	
PC-43	1445	Pub Con	Citizen	Wich	
PC-44	1450	Pub Con	Citizen	Wich	
PC-45	1455	Pub Con	Citizen	Wich	
PC-46	1500	Pub Con	Citizen	Wich	

ASSIGNMENTS

PUBLIC CONCERN PHONE TEAM

Message #	Time	For	From	Location	Assignment
PC-47	1505	Pub Con	Citizen	Wich	
PC-48	1510	Pub Con	Citizen	Wich	
PC-49	1515	Pub Con	Reporter	Wich	
PC-50	1548	Pub Con	Citizen	Wich	
PC-51	1525	Pub Con	Citizen	Wich	
PC-52	1530	Pub Con	Citizen	Wich	
PC-53	1535	Pub Con	Citizen	Wich	
PC-54	1540	Pub Con	Citizen	Wich	
PC-55	1545	Pub Con	Citizen	Wich	

ASSIGNMENTS

MEDIA MONITORING

Message #	Time	For	From	Location	Assignment
MM-1	0900	Med Mon	Radio	Wich/KCPL	
MM-2	0910	Med Mon	TV	Wich/KCPL	
MM-3	0920	Med Mon	TV	Wich/KCPL	
MM-4	0930	Med Mon	Radio	Wich/KCPL	
MM-5	0940	Med Mon	Radio	Wich/KCPL	
MM-6	0950	Med Mon	Radio	Wich/KCPL	
MM-7	1000	Med Mon	TV	Wich/KCPL	
MM-8	1010	Med Mon	TV	Wich/KCPL	
MM-9	1020	Med Mon	Radio	Wich/KCPL	
MM-10	1030	Med Mon	TV	Wich/KCPL	
MM-11	1040	Med Mon	TV	Wich/KCPL	
MM-12	1050	Med Mon	Radio	Wich/KCPL	
MM-13	1100	Med Mon	Radio	Wich/KCPL	
MM-14	1110	Med Mon	Radio	Wich/KCPL	
MM-15	1120	Med Mon	Radio	Wich/KCPL	
MM-16	1130	Med Mon	TV	Wich/KCPL	
MM-17	1140	Med Mon	TV	Wich/KCPL	
MM-18	1150	Med Mon	TV	Wich/KCPL	
MM-19	1200	Med Mon	TV	Wich/KCPL	
MM-20	1205	Med Mon	Radio	Wich/KCPL	
MM-21	1210	Med Mon	TV	Wich/KCPL	
MM-22	1220	Med Mon	TV	Wich/KCPL	
MM-23	1225	Med Mon	TV	Wich/KCPL	
MM-24	1235	Med Mon	Radio	Wich/KCPL	
MM-25	1245	Med Mon	TV	Wich/KCPL	
MM-26	1255	Med Mon	Radio	Wich/KCPL	
MM-27	1305	Med Mon	TV	Wich/KCPL	
MM-28	1315	Med Mon	TV	Wich/KCPL	
MM-29	1325	Med Mon	Radio	Wich/KCPL	
MM-30	1335	Med Mon	TV	Wich/KCPL	
MM-31	1345	Med Mon	Radio	Wich/KCPL	
MM-32	1350	Med Mon	Radio	Wich/KCPL	
MM-33	1400	Med Mon	Radio	Wich/KCPL	
MM-34	1410	Med Mon	Radio	Wich/KCPL	
MM-35	1420	Med Mon	TV	Wich/KCPL	
MM-36	1425	Med Mon	Radio	Wich/KCPL	
MM-37	1430	Med Mon	Radio	Wich/KCPL	
MM-38	1435	Med Mon	Radio	Wich/KCPL	
MM-39	1445	Med Mon	TV	Wich/KCPL	
MM-40	1450	Med Mon	Radio	Wich/KCPL	
MM-41	1500	Med Mon	TV	Wich/KCPL	
MM-42	1505	Med Mon	Radio	Wich/KCPL	
MM-43	1510	Med Mon	Radio	Wich/KCPL	
MM-44	1520	Med Mon	TV	Wich/KCPL	
MM-45	1530	Med Mon	Radio	Wich/KCPL	
MM-46	1540	Med Mon	Radio	Wich/KCPL	

ASSIGNMENTS

MEDIA MESSENGER

Message #	Time	For	From	Location	Assignment
MS-1	1000	Med Msg	Reporter	MRC	
MS-2	1015	Med Msg	Reporter	MRC	
MS-3	1030	Med Msg	Reporter	MRC	
MS-4	1050	Med Msg	Reporter	MRC	
MS-5	1110	Med Msg	Reporter	MRC	
MS-6	1130	Med Msg	Editor	MRC	
MS-7	1150	Med Msg	Editor	MRC	
MS-8	1210	Med Msg	Citizen	MRC	
MS-9	1230	Med Msg	Reporter	MRC	
MS-10	1250	Med Msg	Reporter	MRC	
MS-11	1315	Med Msg	Reporter	MRC	
MS-12	1335	Med Msg	Reporter	MRC	
MS-13	1350	Med Msg	Reporter	MRC	
MS-14	1410	Med Msg	Reporter	MRC	
MS-15	1425	Med Msg	Reporter	MRC	
MS-16	1435	Med Msg	Reporter	MRC	
MS-17	1445	Med Msg	Reporter	MRC	
MS-18	1455	Med Msg	Editor	MRC	
MS-19	1505	Med Msg	Reporter	MRC	
MS-20	1515	Med Msg	Reporter	MRC	
MS-21	1520	Med Msg	Reporter	MRC	
MS-22	1530	Med Msg	Reporter	MRC	

ASSIGNMENTS

COFFEY COUNTY EOC

Message #	Time	For	From	Location	Assignment
CEOC-1	0850	Emrg Prep	Citizen	County EOC	
CEOC-2	0930	Commissioners	Citizen	County EOC	
CEOC-3	1000	Emrg Prep	KMBC-TV	County EOC	
CEOC-4	1030	RO	Media	County EOC	
CEOC-5	1100	County Eng	Access Ctl Personnel	County EOC	
CEOC-6	1230	RO	Fire Leader	County EOC	
CEOC-7	1300	Sheriff	Storekeeper	County EOC	
CEOC-8	1330	Commissioners	Senator's Aide	County EOC	
CEOC-9	0925	County Agent	Farmer	County EOC	
CEOC-10	1400	SSO	Allen Co EPC	County EOC	
CEOC-11	1415	HMMT	Waverly Nurs. Home	County EOC	
CEOC-12	1500	SSO	Host County	County EOC	

ASSIGNMENTS

STATE EOC

Message #	Time	For	From	Location	Assignment
SEOC-1	1030	Dir. KDEP	FEMA	State EOC	
SEOC-2	1100	KNG	KDWP	State EOC	
SEOC-3	1230	Dir. KDEP	FEMA	State EOC	
SEOC-4	1300	KDOT	Goodwin Construction	State EOC	
SEOC-5	1330	KNG	CRO	State EOC	
SEOC-6	1400	KDWP	Citizen	State EOC	
SEOC-7	1415	KHP	Governor's Office	State EOC	
SEOC-8	1430	KBOA	Governor's Press Sec.	State EOC	
SEOC-9	1500	KBOA	County Agent	State EOC	
SEOC-10	1515	KDHE	EPA	State EOC	
SEOC-11	1600	KDWP	US Fish & Wildlife	State EOC	

MESSAGES

A complete listing of messages included in this scenario is provided in the following subsection. Information necessary in making and tracking message assignments is provided for each message.

The list is categorized by message type, recipient or facility, and finally by message number. A space for assigning controllers responsible for the message is included.

The scenario messages are provided to instigate actions at various facilities. More detail for particular maintenance activities is provided in the mini-scenarios.

MESSAGES

SCENARIO

Time-related scenario messages are provided in the following subsection. These messages provide actions and responses that drive the scenario.

MESSAGE NO: 001

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Shift Supervisor

FROM: CR Lead Controller

LOCATION: Control Room

TIME: 0730 (H+00:30)

MESSAGE: Initial Conditions are as follows:

Operationally

See attached forms.

Radiochemically

RCS I-131 Equiv. is 4.86 E-03.

Meterologically

It is an overcast and humid day with winds out of the east-northeast at 12 mph. Daytime temperatures have been ranging from the low 80's to the uppr 80's.

Additional information such as work orders in progress will also be provided at this time.

THIS IS A DRILL

SHIFT SUPERVISOR RELIEF CHECKLIST

OFF-GOING SHIFT (ENTER SHIFT)	MID	DATE 9/1/93	OFF-GOING SS (PRINT): HERHOLD ON-COMING SS (PRINT):

PLANT STATUS SUMMARY -- ON-COMING SS REVIEW PRIOR TO ASSUMING SHIFT (CHECK BOX)

☐ MODE: 1

☐ EVOLUTIONS IN PROGRESS:

① 100% MOL FOR 73 DAYS

② GT HZ-12 WILL NOT CLOSE. GT HZ-11 CLOSED AND DE-ENERGIZED. REFERENCE T/S 3.6.3 b

☐ SIGNIFICANT MAINTENANCE IN PROGRESS:

① 'B' SI PUMP OOS DUE TO A MOTOR FAULT. REPLACEMENT MOTOR BEING PROCURED.

② 'B' RHR PUMP OOS TO REPAIR HEAT EXCHANGER TUBE LEAK

③ 'C' CONDENSER VACUUM PUMP OOS FOR MIC INSPECTION OF COOLER

☐ SIGNIFICANT TESTING/RESTORATION IN PROGRESS OR PENDING:

① PB03 FEEDER BREAKER PB0306 IS OOS FOR ROUTINE BREAKER TESTING. BUSES PB03 AND PB04 ARE CROSS-TIED.

☐ RADWASTE STATUS: NORMAL

☐ GENERAL:

☐ REVIEW THE FOLLOWING PRIOR TO ASSUMING SHIFT:

- ☐ SHIFT SUPERVISOR'S LOG
- ☐ STANDING/SPECIAL ORDERS
- ☐ EQUIP. OUT OF SERVICE LOG
- ☐ ACTION STATEMENT SUMMARY
- ☐ SHIFT CREW COMPOSITION (PAGE 2)
- ☐ SURVEILLANCE SCHEDULE

LICENSED OPERATOR QTR SIGN-OFF LOG UPDATED ______ /

☐ REVIEW THE FOLLOWING AFTER ASSUMING SHIFT:

- ☐ TEMP. MODIFICATION LOG
- ☐ DISCHARGE PERMITS IN EFFECT
- ☐ CLEARANCE ORDER INDEX
- ☐ CONTROL ROOM LOG
- ☐ FIRE PROTECTION IMPAIRMENTS
- ☐ IGNITION SOURCE PERMITS
- ☐ COMBUSTIBLE MATERIAL PERMITS

OPERATIONS TRACKING PROGRAM UPDATED ______ /

OFF-GOING SS LSH INITIALS	ON-COMING SS , INITIALS

SHIFT SUPERVISOR RELIEF CHECKLIST

SHIFT COMPOSITION

SHIFT SUPERVISOR

SUPERVISING OPERATOR

REACTOR OPERATOR

BALANCE OF PLANT

AUX BLDG WATCH

TURB BLDG WATCH

RWSTE BLDG WATCH

SITE WATCH

WATER TREATMENT

FIRE BRIGADE

BRIGADE LEADER

MEMBER

MEMBER

MEMBER

MEMBER

OTHER

CALL SUPERINTENDENT

HP TECHNICIAN

CHEMISTRY TECHNICIAN

SUPERVISING OPERATOR RELIEF CHECKLIST

OFF-GOING SHIFT (ENTER SHIFT)	MID	DATE 9/1/93	OFF-GOING SO (PRINT): CRAIGHEAD ON-COMING SO (PRINT):

PLANT STATUS SUMMARY -- ON-COMING SS REVIEW PRIOR TO ASSUMING SHIFT
(CHECK BOX)

☐ MODE: 1

☐ EVOLUTIONS IN PROGRESS:

① 100% MOL FOR 73 DAYS

② GT HZ-12 WILL NOT CLOSE. GT HZ-11 CLOSED AND DE-ENERGIZED. REFERENCE T/S 3.6.3 b

☐ SIGNIFICANT MAINTENANCE IN PROGRESS:

① 'B' SI PUMP OOS DUE TO A MOTOR FAULT. REPLACEMENT MOTOR BEING PROCURED.

② 'B' RHR PUMP OOS TO REPAIR HEAT EXCHANGER TUBE LEAK

③ 'C' CONDENSER VACUUM PUMP OOS FOR MIC INSPECTION OF COOLER

☐ SIGNIFICANT TESTING/RESTORATION IN PROGRESS OR PENDING:

① PB03 FEEDER BREAKER PB0306 IS OOS FOR ROUTINE BREAKER TESTING. BUSES PB03 AND PB04 ARE CROSS-TIED.

☐ RADWASTE STATUS: NORMAL

☐ GENERAL:

SUPERVISING OPERATOR RELIEF CHECKLIST

☐ REVIEW THE FOLLOWING PRIOR TO ASSUMING SHIFT:

☐ CONTROL ROOM LOG	☐ SHIFT CREW COMPOSITION
☐ STANDING/SPECIAL ORDERS	☐ RO TURNOVER CHECKLIST
☐ EQUIP. OUT OF SERVICE LOG	☐ SURVEILLANCE SCHEDULE
☐ ACTION STATEMENT SUMMARY	☐ KC-008 ALARMS NOT INHIBITED

☐ REVIEW THE FOLLOWING AFTER ASSUMING SHIFT:

☐ TEMP. MODIFICATION LOG	☐ FIRE PROTECTION IMPAIRMENTS
☐ DISCHARGE PERMITS IN EFFECT	☐ IGNITION SOURCE PERMITS
☐ CLEARANCE ORDER INDEX	☐ COMBUSTIBLE MATERIAL PERMITS
☐ LOCKED VALVE LOG	☐ CONDUCT CREW BRIEFING

PRIOR TO SHIFT TURNOVER, OFF-GOING SO

- ✓ REVIEW CONTROL ROOM READINGS
- ✓ REVIEW RADWASTE LOGS
- ✓ ENSURE COMPONENT CYCLIC OR TRANSIENT LIMIT LOG (CCOTLL) IS UPDATED

ATTACH ANY ADDITIONAL PAGES NEEDED TO DESCRIBE PLANT CONDITIONS AND MARK THE NUMBER NEEDED TO DESCRIBE PLANT OF CONTINUATION PAGES [0]

OFF-GOING SO KDC
INITIALS

ON-COMING SO ______
INITIALS

REACTOR/BALANCE OF PLANT OPERATOR TURNOVER CHECKLIST

DATE: 9/1/93	OFF-GOING SHIFT: (ENTER SHIFT)	MIDS	MODE 1
OFF-GOING: RO	SWARTZENDRUBER	ON-COMING: RO	
(PRINT) BOP	WINZENREID	(PRINT) BOP	

PLANT STATUS SUMMARY – ON-COMING RO AND BOP REVIEW PRIOR TO ASSUMING SHIFT (CHECK BOX)

RO BOP

☐ ☐ EVOLUTIONS IN PROGRESS:

① 100% MOL FOR 73 DAYS

② GT HZ-12 WILL NOT CLOSE. GT HZ-11 CLOSED AND DE-ENERGIZED. REFERENCE T/S 3.6.3 b

☐ ☐ MAINTENANCE IN PROGRESS:

① 'B' SI PUMP OOS DUE TO A MOTOR FAULT. REPLACEMENT MOTOR BEING PROCURED.

② 'B' RHR PUMP OOS TO REPAIR HEAT EXCHANGER TUBE LEAK

③ 'C' CONDENSER VACUUM PUMP OOS FOR MIC INSPECTION OF COOLER

☐ ☐ TESTING IN PROGRESS:

① PB03 FEEDER BREAKER PB0306 IS OOS FOR ROUTINE BREAKER TESTING. BUSES PB03 AND PB04 ARE CROSS-TIED.

☐ ☐ COMMENTS:

REACTOR/BALANCE OF PLANT OPERATOR TURNOVER CHECKLIST

ROD CONTROL: ☐ MANUAL ☑ AUTO

ROD POSITION

	A	B	C	D	E
SHUTDOWN BANKS	229	229	229	229	229
CONTROL BANKS	229	229	229	229	

XENON: ☐ INCREASING ☐ DECREASING ☑ EQUILIBRIUM

REACTOR POWER: 3411 MW 100 %

TURBINE POWER: 1192 MWE

T_{AVG} 588.5 °F PZR PRESS 2235 PSIG CB 660 PPM DATE/TIME 9/1/93 0430

RCS LEAKRATE: IDENT 1.1 GPM

UNIDENT .023 GPM DATE/TIME 9/1/93 0430

(Note in Comments if a method other than STS BB-004 is used)

ESF STATUS PANEL SA066X, Y, Z AND REACTOR PARTIAL TRIP/BLOCK PANEL SB069

SA066Y	'B' SI PUMP	OOS DUE TO A MOTOR FAULT.
SA066Y	'B' RHR PUMP	OOS TO REPAIR HEAT EXCHANGER TUBE LEAK

PROTECTION GRADE PERMISSIVE LIGHTS

	ON	OFF		ON	OFF
P6	✓		P10	✓	
P7	✓		P11		✓
P8	✓		P12		✓
P9	✓		P13	✓	

CONTROL GRADE PERMISSIVE LIGHTS

	ON	OFF		ON	OFF
C2		✓	C9A	✓	
C3		✓	C9B	✓	
C4		✓	C9C	✓	
C5		✓	C16		✓
C7		✓			

CHECK OPERATING EQUIPMENT

☐ A CCP
☐ B CCP
☑ PDP

☐ A MD AFP
☐ B MD AFP
☐ TD AFP

LETDOWN ORIFICE
☐ A
☐ B
☑ C

☑ A CCW PMP
☐ C CCW PMP
☐ B CCW PMP
☐ D CCW PMP

SERVICE LOOP
☑ A TRAIN
☐ B TRAIN

☐ A BAT RECIRC
☐ B BAT RECIRC

☐ BL ON RECIRC
☐ BL FILLING

REACTOR/BALANCE OF PLANT OPERATOR TURNOVER CHECKLIST

CHECK OPERATING EQUIPMENT

- [x] A MFP
- [x] B MFP
- [ ] MDMFP
- [x] A COND. PMP
- [x] B COND. PMP
- [x] C COND. PMP
- [x] A HDP
- [x] B HDP

MAIN TURBINE STATUS
- [x] OPERATING
- [ ] SECURED
- [ ] ON TURNING GEAR

AUX STEAM SOURCE
- [ ] AUX BOILER
- [x] REBOILER

GLAND STEAM SOURCE
- [ ] AUX STEAM
- [x] MAIN STEAM

- [x] A CIRC WTR PMP
- [x] B CIRC WTR PMP
- [x] C CIRC WTR PMP
- [x] A SERVICE WTR PMP
- [x] B SERVICE WTR PMP
- [ ] C SERVICE WTR PMP
- [ ] LOW FLOW PMP
- [ ] A ESW
- [ ] B ESW

CONDENSATE DEMINS STATUS
- A [x]
- B [x]
- C [ ]
- D [ ]
- E [ ]
- F [ ]

L.P. HEATERS IN SERVICE: [x] YES [ ] NO - STRING [ ] A [ ] B [ ] C

H.P. HEATERS IN SERVICE [x] YES [ ] NO - STRING [ ] A [ ] B

S/G TOTAL BLOWDOWN FLOW 120 klbm/HR TO [x] CONDENSER [ ] LAKE

[ ] AP FILLING [x] AN FILLING

LEAD AIR COMPRESSOR [ ] A [x] B [ ] C

READING SHEETS REVIEWED:
- [x] AUX BUILDING READINGS
- [x] TURBINE BUILDING READINGS
- [x] DAILY SITE READINGS (NIGHT SHIFT)

FOR TRAINING USE ONLY

ON-COMING RO AND BOP REVIEW PRIOR TO ASSUMING SHIFT

- [ ] [ ] CONTROL ROOM LOG
- [ ] [ ] SHIFT CREW COMPOSITION
- [ ] [ ] EQUIPMENT OUT OF SERVICE LOG
- [ ] [ ] ACTION STATEMENT SUMMARY
- [ ] [ ] INSTRUMENT OUT OF SERVICE LOG
- [ ] [ ] SURVEILLANCE SCHEDULE
- [ ] [ ] DISCHARGE PERMITS
- [ ] [ ] CLEARANCE ORDER LOG
- [ ] [ ] IGNITION SOURCE PERMITS
- [ ] [ ] TEMPORARY MODIFICATION LOG
- [ ] [ ] CHART RECORDERS

(THESE MAY BE REVIEWED AFTER ASSUMING SHIFT)
- [ ] [ ] LOCKED VALVE LOG
- [ ] [ ] FIRE PROTECTION IMPAIRMENTS
- [ ] [ ] COMBUSTIBLE MATERIAL PERMITS
- [ ] [ ] COMPUTER ALARM SUMMARY
- [ ] [ ] NIGHT ORDERS
- [ ] [ ] STANDING/SPECIAL ORDERS

ATTACH ANY ADDITIONAL PAGES NEEDED TO DESCRIBE PLANT CONDITIONS

MARK THE NUMBER OF ADDITIONAL PAGES 0

OFF GOING: RO CAS
INITIALS
BOP BKW
INITIALS

ON COMING: WALKED DOWN MCB, CHECK FOR BURNED OUT LIGHTS
RO ____________
INITIALS
BOP ____________
INITIALS

ATTACHMENT 1
EQUIPMENT OUT OF SERVICE LOG

1 YR EOL	2 SYS DESIG	3 DECLARED INOPERABLE			4 REQUIRED RETURN			5 APPLICABLE MODES							6 MODE RESTRAINT	7 TECHNICAL SPECIFICATIONS	8 RETURNED TO OPERABLE		
		DATE	TIME	SS	DATE	TIME	SS	1	2	3	4	5	6	ALL			DATE	TIME	SS
9 REMARKS/EQUIPMENT/MWR/TMO/CO/RETEST																			
93-156	CG	8/24/93	1543	KDR	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A			
'C' CONDENSER VACUUM PUMP OOS FOR MIC INSPECTION OF ITS COOLER																			
93-157	EM	8/31/93	0211	KDR	9/3/93	0211	KDR	✓	✓	✓					MODE 1,2 & 3	3.5.2			
'B' SAFETY INJECTION PUMP (PEJ01B) DUE TO MOTOR FAULT.																			
93-158	EJ	8/31/93	0740	KDR	9/3/93	0740	KDR	✓	✓	✓					MODE 1,2 & 3	3.5.2			
'B' RHR PUMP (PEJ01B) AND 'B' RHR HEAT EXCHANGER (EEJ01B) TO REPAIR HEAT EXCHANGER TUBE LEAK																			
93-159	PB	9/1/93	0602	KDR	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A			
PB0306 OOS FOR BREAKER TESTING. PB03 AND PB04 ARE CROSS-TIED.																			

ATTACHMENT 1
EQUIPMENT OUT OF SERVICE LOG

1 YR EOL	2 SYS DESIG	3 DECLARED INOPERABLE			4 REQUIRED RETURN			5 APPLICABLE MODES							6 MODE RESTRAINT	7 TECHNICAL SPECIFICATIONS	8 RETURNED TO OPERABLE		
		DATE	TIME	SS	DATE	TIME	SS	1	2	3	4	5	6	ALL			DATE	TIME	SS
9 REMARKS/EQUIPMENT/MWR/TMO/CO/RETEST																			
93-160	GT	9/1/93	0615	KDR	N/A	N/A	N/A	✓	✓	✓	✓				MODE 1,2,3, & 4	3.6.3			
GT HZ-12 WILL NOT CLOSE. GT HZ-11 IS CLOSED AND DE-ENERGIZED IN ACCORDANCE WITH T/S 3.6.3 b.																			

ATTACHMENT 6

ACTION STATEMENT SUMMARY LOG

E.O.L. #	TECH. SPEC. #	ENTERED			DEADLINE			REMARKS	RESTORED		
YR		DATE	TIME	SS	DATE	TIME	SS		DATE	TIME	SS
93-1	3.5.2	8/31/93	0211	KDR	9/3/93	0211	KDR	'B' SI PUMP OOS DUE TO ??????			
93-2	3.5.2	8/31/93	0211	KDR	9/3/93	0211	KDR	TO REPAIR 'B' RHR HEAT EXCHANGER TUBE LEAK			

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Shift Supervisor

FROM: CR Lead Controller

LOCATION: Control Room

TIME: 0815 (H+00:15)

MESSAGE: You have just received indications of a halon discharge on fire protection panel KC008.

NOTE: This message is only provided if the simulator fails.

THIS IS A DRILL

MESSAGE NO: 003

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Control Room Operator

FROM: Auxiliary Building Operator (CR Lead Controller)

LOCATION: Auxiliary Building (Switchgear Room No. 1)

TIME: 0820+ (H+00:20+)

MESSAGE: The fire appears to have been totally extinguished by the halon fire protection system.

NOTE: This message is only provided if the simulator fails.

THIS IS A DRILL

MESSAGE NO: 004c

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Shift Supervisor

FROM: CR Lead Controller

LOCATION: Control Room

TIME: 0830+ (H+00:30+)

MESSAGE: A Notification of an Unusual Event should have been declared based on a plant shutdown per Tech Spec. 3.0.3 (loss of both trains of ECCS).

NOTE: DO NOT pass this message without the consent of the Exercise Lead Controller.

MESSAGE NO: 005

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Duty Emergency Director/Shift Supervisor

FROM: CR Lead Controller

LOCATION: Control Room

TIME: 0845+ (H+01:45+)

MESSAGE: A 1000 gallon per minute LOCA occurs. The reactor and turbine trip.

NOTE: This message is only provided if the simulator fails.

THIS IS A DRILL

MESSAGE NO: 006c

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Duty Emergency Director/Shift Supervisor

FROM: CR Lead Controller

LOCATION: Control Room

TIME: 0900+ (H+02:00+)

MESSAGE: An Alert should have been declared based on the loss of one fission product barrier.

NOTE: **Do not pass this message without the consent of the Exercise Lead Controller.**

THIS IS A DRILL

MESSAGE NO: 007c

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Duty Emergency Director

FROM: TSC Lead Controller

LOCATION: Technical Support Center

TIME: 1000+ (H+02:00+)

MESSAGE: A Site Area Emergency should have been declared based on inadequate core cooling conditions with containment closure intact.

NOTE: DO NOT pass this message without the consent of the Exercise Lead Controller.

THIS IS A DRILL

MESSAGE NO: 008

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: State Liaison in the EOF

FROM: EOF Lead Controller

LOCATION: EOF

TIME: 1100+ (H+03:00+)

MESSAGE: Please perform a shift change at this time.

THIS IS A DRILL

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Security Personnel

FROM: Security Controller

LOCATION: Security Building

TIME: 1215+ (H+04:15+)

MESSAGE: All portal monitors start alarming.

THIS IS A DRILL

MESSAGE NO: 010

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Shift Supervisor

FROM: CR Lead Controller

LOCATION: Control Room

TIME: 1230 (H+08:30)

MESSAGE: You have just been told that all portal monitors in the Security Building have alarmed.

NOTE: DO NOT pass this message unless the SEC does not relay this information to the Control Room.

THIS IS A DRILL

MESSAGE NO: 011c

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Duty Emergency Manager

FROM: EOF Lead Controller

LOCATION: Emergency Operations Facility

TIME: 1245+ (H+04:45+)

MESSAGE: A General Emergency should have been declared based on core uncovery with the containment closure not intact.

NOTE: DO NOT pass this message without the consent of the Exercise Lead Controller.

THIS IS A DRILL

MESSAGE NO: 012

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Shift Supervisor

FROM: CR Lead Controller

LOCATION: Control Room

TIME: 1340 (H+05:40)

MESSAGE: You have completed sufficient repairs on the residual heat removal system "B" train to permit its use.

NOTE: This message is only provided if the ERDC teams do not relay the above information to the Control Room after effecting repairs.

THIS IS A DRILL

MESSAGE NO: 013

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Shift Supervisor

FROM: CR Lead Controller

LOCATION: Control Room

TIME: 1500 (H+07:00)

MESSAGE: The concrete hatch has been replaced in the roof of the 2026' level of the auxiliary building.

NOTE: This message is only provided if the ERDC teams do not relay the above information to the Control Room after effecting repairs.

THIS IS A DRILL

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Shift Supervisor

FROM: CR Lead Controller

LOCATION: Control Room

TIME: 1515 (H+07:15)

MESSAGE: You have completed repairs on the 4160 volt bus NB01. Energization of the bus can be safely accomplished.

NOTE: This message is only provided if the ERDC teams do not relay the above information to the Control Room after effecting repairs.

THIS IS A DRILL

MESSAGE NO: 015

THIS IS A DRILL

<u>DO NOT</u> initiate actions affecting normal plant operations.

TO: All Participants

FROM: Facility Lead Controllers

LOCATION: Emergency Response Facilities

TIME: 1600 (H+08:00)

MESSAGE: Exercise activities have been terminated. Collect all logs, notes, etc. and give them to Facility Lead Evaluator. Participants should prepare for a players' critique.

<u>NOTE:</u> <u>DO NOT</u> pass this message without the consent of the Exercise Lead Controller.

THIS IS A DRILL

MESSAGES

MINI-SCENARIOS

The following mini-scenarios provide instructions and supplementary information for controllers in the Control Room and with the Emergency Repair and Damage Control (ERDC) Teams.

The information provided in these mini-scenarios assumes that players will take certain actions in response to the Exercise scenario. Controllers must be cognizant of the actions of the players to which they are assigned. The information provided in this section does not preclude the possibility that controllers will be required to provide additional information to the players based on player actions.

Maintenance Mini Scenario #1

LOSS OF 4160 VOLT BUS NB01

Time: 0815 (H+00:15)

Location: Switchgear Room No. 1

Tasks:

Loose bolted connections of the main bus between NB0104 and 5 heat up, short to ground, and start a fire. The fire is almost immediately extinguished by the halon system.

Main breaker (NB0112) trips and the diesel generator "A" starts, but breaker NB0111 doesn't close. The diesle is unable to re-energize the bus.

Relay and Meter technicians find 151G/F Flag, 186/F Flag and 151/F AB&C Phase flagged on cubicle NB0112, and find the NB0112 breaker tripped. All other cubicles are all right.

Maintenance should identify bus and insulation damage at cubicles NB0104 and NB0105. The bus will need to be replaced on B phase and insulation repaired on A and C phases.

To repair NB01, a bus section should be removed from spare cubicle NB0117 and installed in damaged cubicle.

Completion:

These tasks will take about 7 hours to complete; NB01 should be returned operable to the plant by H+07:15.

Maintenance Mini Scenario #2

REPAIRS TO THE RESIDUAL HEAT REMOVAL SYSTEM "B" TRAIN

Time: Duration of the Exercise

Tasks:

Repairs are in progress to repair a tube to shell side leak in the heat exchanger. Reassembly of the system is in progress. The shell has just been disconnected from the Manitowok crane. Repairs are estimated to take an additional 20 hours of the 32 hours remaining before entering a Technical Specification action level (Ref: initial conditions).

Completion:

Sufficient repairs are completed on the heat exchanger to permit its use after H+08:00. The timely startup of the system is important to scenario play and coordinated radiological parameters.

Maintenance Mini Scenario #3

REPLACEMENT OF THE AUXILIARY BUILDING CONCRETE ROOF HATCH

Time: Duration of the Exercise

Location: Auxiliary building elevation 2047' 6", roof, in the vicinity of the residual heat removal system "B" train heat exchanger roof hatch.

Tasks:

The concrete hatch in he auxiliary building has been removed to allow for the removal of the heat exchanger shell on the residual heat removal system. The hatch was removed using the Manitowok crane.

Air flow is into the auxiliary building all the while the hatch is open. This is not a release point for radiological effluents (Ref: initial conditions).

Attempts may be made to replace the concrete hatch so that the auxiliary building ventilator system can be shut down, thereby terminating the release beginning at 1218 (H+04:18).

If the Manitowok crane is used, it stops functioning just as the concrete hatch is being replaced in the roof of the 2026' level of the auxiliary building. Foul smelling smoke is coming from the engine compartment of the crane. Some of the wiring harness appears burned with melted insulation. The concrete hatch is suspended approximately 4 feet above the roof.

Completion:

The repairs on the crane engine wiring may be completed and the crane made functional at H+07:00+.

Maintenance Mini Scenario #4

FAILURE OF VALVE GT HZ-12

Time: 0830+ (H+00:30+)

Location: Personnel Hatch outside Containment

Tasks:

When the NB01 bus is deenergized, containment isolation valve GT HZ-12 fails. Should a repair team be dispatched, the time from notification of valve not operating to the scaffolding going up is 2-3 hours.

Trouble-shooting the valve should also take about 2-3 hours. The team discovers that the wrong grease was used during the last time maintenance was performed, causing the O-rings to swell up and the actuator to stop working. Repairs could take from 6-10 hours.

Completion:

This task cannot be completed in time to isolate the release from the ductwork breach downstream of HZ-12. If an attempt is made to cover the end of the open ductwork, this will require about 1.5 hours to accomplish once the scaffolding is in place. The release to the auxiliary building and thus the plant unit vent will be terminated when the pressure inside containment equalizes with pressure inside the auxiliary building.

Maintenance Mini Scenario #5

PARTIAL OPENING OF VALVE EN HV-12

Time: 1215 (H+04:15)

Tasks:

When containment spray actuation signal (CSAS) is generated, containment spray pump "B" discharge valve EN HV-12 only opens 20% due to bent valve stem. The bent valve stem causes actuator to torque out approximately 20% open.

Several methods of repair may be attempted:

1) If maintenance jumpers torque switch, motor will lock rotor and trip breaker.
2) If Operations tries to manually declutch operator and manually open valve, they find hand wheel just freewheels due to clutch keys being broken off.
3) If Maintenance tries to remove actuator, they find stem nut lock nut bound.
4) If Maintenance tries to remove actuator by turning the entire actuator off stem, they will hit the design supports.

Completion:

This task will not be completed in time to restore the valve to its fully opened position during the Exercise.

MESSAGES

PUBLIC INFORMATION

The following section consists of Wolf Creek Public Information Officer, Media Inquiry, Public Concern, Media Monitoring, and Media Messenger messages.

These messages provide questions and responses from citizenry and the media and assumes that the responses given will provide careful and informative information.

PIO-1 TO: Wolf Creek Public Information Officer

FROM: Reporter

TIME: 0845

MESSAGE: "This is a Drill"

This is Bryon Hill at WIBW radio in Topeka. We've been told by a Wolf Creek employee that there is a fire burning inside the plant. Can you confirm this? Is any radioactivity being released from the plant? Is the fire under control? Was anyone injured? What caused the fire? Has the NRC been notified? Has the plant been shut down because of this?

PIO-2 TO: Wolf Creek Public Information Officer

FROM: Reporter

TIME: 0850

MESSAGE: "This is a Drill"

This is Kelly Waldo at KAKE-TV. Wanted some information about the fire at Wolf Creek. Is the reactor on fire, like what happened at Chernobyl? Can we fly our helicopter out there to get photos of the blaze?

PIO-3 TO: Wolf Creek Public Information Officer

FROM: Reporter

TIME: 0910

MESSAGE: "This is a Drill"

This is Scott Raust at the Eagle-Beacon. What's happening at Wolf Creek this morning? Is the plant shut down now? How long will repairs take? Does this have anything to do with the refueling operation you did a while back? How much is it going to cost to get back in operation? Is this considered an emergency situation? How many other Unusual Events have you had?

PIO-4 TO: Wolf Creek Public Information Officer

FROM: Reporter

TIME: 0920

MESSAGE: "This is a Drill"

This is Ansom Wilson of the Associated Press. We've received your news release about the Wolf Creek fire. Was anyone hurt? The release says emergency plans are being followed - what does that mean? Have you had unusual events before? How many? Is the problem now solved?

Note: Do not call until NUE release is distributed.

PIO-5 TO: Wolf Creek Public Information Officer

FROM: Reporter

TIME: 0945

MESSAGE: "This is a Drill"

Bill Baxter here with KEGS radio in Emporia. I hear there was a problem at Wolf Creek this morning? What's going on?

PIO-6 TO: Wolf Creek Public Information Officer

FROM: Congressional Aide

TIME: 1000

MESSAGE: "This is a Drill"

This is Pam Raspmueller, and I'm an aide to Congressman Don Glassman. Congressman Glassman feels it is appropriate for him to make a public statement about the disaster at Wolf Creek, and I'm helping him compile information to facilitate that endeavor. Can you tell me what caused the problem at the plant? How much do you think it will cost to clean up the damage?

MI-1 TO: Media Inquiry

FROM: Reporter

TIME: 0940

MESSAGE: "This is a Drill"

Byron Hill again from WIBS radio in Topeka. I called earlier today about the Wolf Creek fire. What's going on now? Is there any chance of this going out of control?

MI-2 TO: Media Inquiry

FROM: Reporter

TIME: 0945

MESSAGE: "This is a Drill"

I'm Martin Rosenthal at Kansas City Star. What caused the fire today at Wolf Creek? Any idea what it will cost to fix the damage? When was the last time you had an unusual event? How many have you had since the plant started up? Is this more than normal?

MI-3 TO: Media Inquiry

FROM: Reporter

TIME: 0950

MESSAGE: "This is a Drill"

I'm John Birnbaum with the Manhattan Mercury. We got a wire story saying there's an Alert at the Wolf Creek nuclear plant. Can you update me on what's going on out there? Has there been any release of radiation? If radioactivity is released, what will you do to save the people who live near the plant? Would you anticpate the need to evacuate Emporia or Topeka?

MI-4 TO: Media Inquiry

FROM: Reporter

TIME: 1000

MESSAGE: "This is a Drill"

This is Lee Atweather at WIBW in Topeka. We just had a call from a person in Lyndon who was trying to drive down to I-35. She said we couldn't get through, and that there were all kinds of people in strange plastic suits down by the intersection of Highway 75 and K-31. Is this something to do with Wolf Creek? Has there been a release of radioactivity that's been blown up to the Melvern lake area?

Note: Controller notify Lead Controller prior to placing call so rumor can be tracked.

MI-5 TO: Media Inquiry

FROM: Reporter

TIME: 1005

MESSAGE: "This is a Drill"

This is Bill McCoy with KZ-93 radio in Osage City. We've heard Highway 75 at Melvern Lake has been blocked, and there are people in funny suits all over the place there. Do you know what's going on? Is it something to do with the Wolf Creek accident?

Note: Controller notify Lead Controller prior to placing call so rumor can be tracked.

MI-6 TO: Media Inquiry

FROM: Reporter

TIME: 1010

MESSAGE: "This is a Drill"

Hello, this is Jennifer Forest of the Kansas Information Network. We are getting several calls here about the Wolf Creek emergency. Can we give reporters from outside the Kansas area an 800 number to call for information? Do you special numbers for ratepayers to call? We'll be glad to give out some phone numbers if it will help.

MI-7 TO: Media Inquiry

FROM: Reporter

TIME: 1015

MESSAGE: "This is a Drill"

This is Billy Martins from the Phoenix Sun. I'm writing a story about the reactor accident. Can you tell me about the explosion and fire? I'd like to put it in terms our readers can understand. Was anyone hurt? Have you evacuated anyone? Is Wolf Creek the same kind of plant as the Palo Verde plant in Wintersburg Arizona?

MI-8 TO: Media Inquiry

FROM: Reporter

TIME: 1020

MESSAGE: "This is a Drill"

This is Greg Roberts at KSN news. I need some information on Wolf Creek to use in our noon newscast. Exactly where is the plant located? Is there just one reactor there? Are there plans to build a second unit there? Who was the contractor that built the plant? Do you have a phone number I can call? I know there have been a lot of questions about construction quality and whether there was good enough management during construction. Do you think the accident was caused by poor construction or defective materials, or was operator error at fault? Is this acident as serious as what happened at Three Mile Island? Has the NRC taken over attempts to get the situation under control?

MI-9 TO: Media Inquiry

FROM: Reporter

TIME: 1025

MESSAGE: "This is a Drill"

Hi, I'm Byron Hill at WIBW radio in Topeka. Say, we have a tip from a Wolf Creek employee that a Site Area Emergency has been declared at the plant and that you're getting ready to evacuate some people from the area. Is that true? We haven't seen any official word about it, so we haven't put anything out on the air - but it it's true, the public has a right to know. What can we say to our listeners?

MI-10 TO: Media Inquiry

FROM: Reporter

TIME: 1030

MESSAGE: "This is a Drill"

Hello, Wayland Robinson here from KSTS-TV in Tulsa, Oklahoma. Is this the Media Release Center where we can go to get news about Wolf Creek? Where is it located? If we fly in to Topeka, how do you get to the center from the airport? What hours will it be open?

MI-11 TO: Media Inquiry

FROM: Reporter

TIME: 1035 **(WAIT UNTIL SAE STATEMENT IS ISSUED)**

MESSAGE: "This is a Drill"

I just got your news release about a Site Area Emergency and I don't understand it at all. This is Kelly Waldo at KAKE-TV, and someone there told me earlier that everything was under control. From this release it sounds like there could be radioactivity coming from the plant. We'd really like to come up there - we have a radiation monitor we got from the university on our helicopter, so we would know if we were flying into a radiation cloud. Where is the nearest place we can fly to, and can we do at least a couple passes over the plant to get something on tape?

MI-12 TO: Media Inquiry

FROM: Reporter

TIME: 1040 **(WAIT UNTIL SAE STATEMENT IS ISSUED)**

MESSAGE: "This is a Drill"

This is Martin Rosenthal with the Kansas City Star. We just got your release on the Site Emergency. Let's speculate a minute - if radiation is released, how many people would be in the path of the radioactive cloud? If a release happens, couldn't a number of people die within a matter of days, like they did at Chernobyl? What's different from Chernobyl - seems to me like you have a plant out of control, like the Russians did. And, radiation's radiation, right? So how are you going to prevent another Chernobyl, right here in Kansas?

MI-13 TO: Media Inquiry

FROM: Reporter

TIME: 1045 **(WAIT UNTIL SAE STATEMENT IS ISSUED)**

MESSAGE: "This is a Drill"

This is Greg Roberts KSN news. We'd like to get permission to go along with a team of your workers at the Wolf Creek disaster and film them doing whatever they do to try and fix the plant. We're willing to sign a release to hold your company harmless from any liability for injury to our crew. All we need is one person to show us around. Will this work? This would really be a big story for us.

MI-14 TO: Media Inquiry

FROM: Reporter

TIME: 1050

MESSAGE: "This is a Drill"

Hello, this is Terry Grote of Kansas Business. I'm writing the background story for the Wolf Creek disaster. One of our reporter said you had packets of information about the plant and the WCNOC emergency plan. If we sent someone over, can we get a couple of the packets? What kind of information is in the packet?

MI-15 TO: Media Inquiry

FROM: Reporter

TIME: 1055

MESSAGE: "This is a Drill"

I'm a reporter from San Luis Obispo, California. Can you help me with some questions about the nuclear accident? You know the wire service had Kansas City Power & Light as the owners. I called that general number and the operator didn't know where to send my call. Someone better let her know there's an emergency going on, because I bet I'm not the only one having trouble reaching you people. Now start at the beginning and tell me about WCNOC's history, about their construction problems at Wolf Creek, and about the emergency you are having. By the way, my name is Alice Owens.

MI-16 TO: Media Inquiry

FROM: Reporter

TIME: 1100

MESSAGE: "This is a Drill"

Hello. This is Lonnie Sanderson from WKPR TV in Cleveland, Ohio. I understand a Site Area Emergency has been declared at the Wolf Creek nuclear plant in Kansas. What is a Site Area Emergency exactly? I also understand that two radiation barriers have been broken? What are those barriers? Are they physical barriers? By the way - where is the plant located? How many generating units are there, and who owns the plant? Who is responsible for clean-up of the plant?

MI-17 TO: Media Inquiry

FROM: Reporter

TIME: 1105

MESSAGE: "This is a Drill"

Hello, I am Louise Simpson with KEYN Radio in Wichita. I need something cleared up before our next newscast. We have material on Wolf Creek that says it is a Pressurized Water Reactor, PWR. In one of the newscasts from a TV station, they said it was a SNUPPS plant. Which is correct and what does SNUPPS stand for? Why don't you nuclear people just say what you mean instead of making up initials to hide the facts?

MI-18 TO: Media Inquiry

FROM: Reporter

TIME: 1110

MESSAGE: "This is a Drill"

This is Jules Birnbaum with ABC News. Have you had a loss of Coolant Accident at your nuclear power plant? Can you tell me if there's danger of a hydrogen buildup in the containment building, and if this buildup could result in an explosion?

MI-19 TO: Media Inquiry

FROM: Reporter

TIME: 1115

MESSAGE: "This is a Drill"

I want to talk to someone in authority. This is Ray Watson of Channel 4 in Kansas City. We sent a news team to the Wolf Creek site to get a story about the accident. They were stopped by a barricade and the Highway Patrol refuses to let them in to get the news. What are you going to do about it? I'm sure you are familiar with the term "Freedom of the Press!"

MI-20 TO: Media Inquiry

FROM: Reporter

TIME: 1120

MESSAGE: "This is a Drill"

Hi, this is Maria Holt, with the Tri-City Herald in Kennewick, Washington. According to the news story about Wolf Creek, the plant's architect-engineer is Bechtel Power Company. The nuclear plant operating 15 miles from our office also used Bechtel Power as their architect-engineer. My question is was the accident due to poor construction and if so, has Bechtel Power issued a statement?

MI-21 TO: Media Inquiry

FROM: Reporter

TIME: 1130

MESSAGE: "This is a Drill"

Hello, this is Dennis Petersen with the Lawrence Journal World. We would like to get some comments from the Lawrence group who opposed the Wolf Creek plant during the Atomic Safety and Licensing Board hearings back in 1984. Do you have any of their names or phone numbers?

MI-22 TO: Media Inquiry

FROM: Reporter

TIME: 1140

MESSAGE: "This is a Drill"

This is Randy Grahm at the Cable News Network. I need some technical information on how a nuclear power plant works. I realize your engineers are probably busy working on the plant, but do you know any experts maybe at K-State or KU, that could give me some information?

MI-23 TO: Media Inquiry

FROM: Reporter

TIME: 1150

MESSAGE: "This is a Drill"

Hello, this is Bryan Franks of KMBC-TV, in Kansas City. We understand the FAA has closed the airspace above the Wolf Creek nuclear plant. We would like to fly our helicopter over and get some shots of the emergency teams at work. Can you get permission for us to fly over about 2:00 p.m.?

MI-24 TO: Media Inquiry

FROM: Reporter

TIME: 1155

MESSAGE: "This is a Drill"

This is Janette Rives with the Wichita Eagle. Let's assume the accident at Wolf Creek gets worse, and there's a release of radiation something on the scale of Chernobyl. Wouldn't that mean everyone within a hundred miles would need to evacuate? That could include evacuation of Emporia or even Topeka, couldn't it? Do your emergency plans consider evacuation of those cities?

MI-25 TO: Media Inquiry

FROM: Reporter

TIME: 1205

MESSAGE: "This is a Drill"

I'm Martin Rosenthal at the Kansas City Star. I need some technical information for a story we're doing on Wolf Creek. What kind of material are the fuel rods made of? What is the melting temperaure for these rods? At what temperature does fuel melt? I understand that when you have fuel damage, you have a chance of having a "hydrogen burn." What is that? Does it mean the containment building might be blown apart - is that what you're afraid of? Is there some place I can look at detailed drawings of the plant?

MI-26 TO: Media Inquiry

FROM: Reporter

TIME: 1220

MESSAGE: "This is a Drill"

This is Michael Douglas with 20th Century Fox productions in Los Angeles. We've been listening to the news about your nuclear power accident out there. I think it might just make a good movie, and I'd like to know who to talk to about getting movie rights. Is someone available now, or should I write a letter to someone?

MI-27 TO: Media Inquiry

FROM: Reporter

TIME: 1230 **(WAIT UNTIL GENERAL EMERGENCY HAS BEEN DECLARED)**

MESSAGE: "This is a Drill"

I'm Kelly Waldo with KAKE-TV. We just got your news release about the General Emergency. Can you tell me if there has actually been radioactivity released, or do you just expect it to happen? When you say a "significant" amount of radiation, what do you mean - is it enough to kill someone, or make them sick? How does this compare with the amount of radiation released from the Chernobyl reactor? How many people will have to be evacuated? Will this part of the state be usuable or will the contamination ruin the land for thousands of years?

MI-28 TO: Media Inquiry

FROM: Reporter

TIME: 1240

MESSAGE: "This is a Drill"

This is Wiley Reed at KFDI. I need some clarification on your last news release. What exactly caused you to declare this General Emergency? How many people are being evacuated and where are you sending them? Has anyone been contaminated or hurt? Does the Nuclear Regulatory Commission know all about this? have they taken over trying to fix the plant?

MI-29 TO: Media Inquiry

FROM: Reporter

TIME: 1250

MESSAGE: "This is a Drill"

Gene Overton, CBS News. We've picked up a wire report that a General Emergency has been declared at your Wolf Creek plant. Is this the most serious accident ever to occur in a nuclear power plant? Has a General Emergency ever been declared in the United States? How does this compare in severity to the Chernobyl accident? How much radiation is coming out of the plant? How many people are you going to evacuate?

MI-30 TO: Media Inquiry

FROM: Reporter

TIME: 1300

MESSAGE: "This is a Drill"

This is Julie May, KSNW-TV. We've heard that you've declared a General Emergency at Wolf Creek. According to your background information, this means radioactivity could be released outside the plant. Has this happened, and has anyone in the area been contaminated?

MI-31 TO: Media Inquiry

FROM: Reporter

TIME: 1310

MESSAGE: "This is a Drill"

This is Rick Manning with KWCH-TV. I'm following up on the last news release about a General Emergency at Wolf Creek. Does this mean you will have to evacuate people from the area? How many people live within the area which you could evacuate? Where do they go? Will they ever be able to return to their homes? Did this explosion or whatever it was that caused you to do a General Emergency actually blow a hole in the big domed building? We're going to send a camera crew up there - how close to the plant will they be able to get without being contaminated?

MI-32 TO: Media Inquiry

FROM: Reporter

TIME: 1320

MESSAGE: "This is a Drill"

This is Charles Tatcher calling from London, England, the London Daily News. We have a UPI report of an accident at your Wolf Creek nuclear power station. Can you confirm this? What's happening? Has anyone been hurt? Are you evacuating anyone? Didn't I hear that our United Kingdom Central Generating Board purchased the Wolf Creek reactor design and is operating it over here? How much did they pay? Do you think a design defect caused this accident?

MI-33 TO: Media Inquiry

FROM: Reporter

TIME: 1330

MESSAGE: "This is a Drill"

I'm Jordon Hunt with the National Enquirer. We have the UPI news report on your Wolf Creek nuke plant disaster. We're right at the deadline for this issue, but I'd like to get in a story with some pictures. Can our reporter and a photographer get over there and take some pictures? What's the nearest town? Has it been evacuated? How do we get there from Kansas City?

MI-34 TO: Media Inquiry

FROM: Reporter

TIME: 1340

MESSAGE: "This is a Drill"

I'm Sandy Chapman with KWOX radio in Coffeyville. Was there a meltdown at Wolf Creek? What's the difference between what's happening now and what happened at Chernobyl? Will people have to leave their homes? Has anyone been hurt?

MI-35 TO: Media Inquiry

FROM: Reporter

TIME: 1350

MESSAGE: "This is a Drill"

Hello, is this the Information Center for emergency information? I'm Max Watts of the San Diego Dispatch. The Governor of California, while speaking at a luncheon today, said California is fully trained and prepared to handle any emergency that could arise at the Diablo Canyon nuclear plant. Did Kansas have a plan before the accident started this morning at the Wolf Creek plant?

MI-36 TO: Media Inquiry

FROM: Reporter

TIME: 1400

MESSAGE: "This is a Drill"

This is Barb Sloan from the Clay County Dispatch in Clay Center, Kansas. I'd like to have some details on the emergency at Wolf Creek. Do you have anything you can send me about Wolf Creek and what happens during a nuclear accident?

MI-37 TO: Media Inquiry

FROM: Reporter

TIME: 1405

MESSAGE: "This is a Drill"

This is Wayne Donaldson, at the St. Louis Post Dispatch. We're working on a story about the Wolf Creek disaster, and I seem to remember that the Callaway plant in Missouri is a duplicate of Wolf Creek. Do you think there's a chance the same thing could go wrong at Callaway? Have your engineers talked to the people at Callaway to explain what went wrong?

MI-38 TO: Media Inquiry

FROM: Reporter

TIME: 1410

MESSAGE: "This is a Drill"

I'm Conny Wany with the Denver Post-Dispatch. We have the UPI wire stories on the Wolf Creek accident, but I have a question. When Chernobyl blew up, radioactivity was dispersed all over the world, and they found pretty high levels in some of the European countries hundreds of mile away. What's to keep this accident from doing the same thing - isn't it a good possibility of seeing high levels of radioactivity throughout at least the mid-western states?

MI-39 TO: Media Inquiry

FROM: Reporter

TIME: 1415

MESSAGE: "This is a Drill"

This is Bonnie Tyler at WHO news radio in Chicago. We're on the air now - can you tell our listeners what has happened at Wolf Creek nuclear plant today?

MI-40 TO: Media Inquiry

FROM: Reporter

TIME: 1420

MESSAGE: "This is a Drill"

Hello. This is Bonnie Black from the Boston Globe. It is my understanding that the Wolf Creek Nuclear Generating Station in Kansas has had a serious accident. What happens after the accident is under control? Will the evacuees ever be allowed to return to their homes? How long before the plant will be allowed to operate again, or will it ever? Who is responsible for repairs and estimating the damage? Is there an emergency fund to pay for the cost of an accident - or will the ratepayers be charged for the costs?

MI-41 TO: Media Inquiry

FROM: Reporter

TIME: 1425

MESSAGE: "This is a Drill"

I'm Kelly Waldo at KAKE-TV. We're putting together our 6:00 p.m. newscast. Is someone here in Wichita who can do an on-camera interview? We'd like to come to your office at 4:30 this afternoon.

MI-42 TO: Media Inquiry

FROM: Reporter

TIME: 1430

MESSAGE: "This is a Drill"

I'm Louise Simpson at KEYN. We just got a call from a guy who says he lives near Wolf Creek. He says he just heard another explosion at the plant. Is that true, and if so, has the containment ruptured? Is there more radioactivity leaking out?

Note: Contact Lead Controller before placing call. Rumor to track.

MI-43 TO: Media Inquiry

FROM: Reporter

TIME: 1435

MESSAGE: "This is a Drill"

I'm Wyley Reed at KFDI. We heard there's been another explosion at Wolf Creek. Is this true? Anyone hurt? Has more radiation spilled out?

Note: Contact Lead Controller before placing call. Rumor to track.

MI-44 TO: Media Inquiry

FROM: Reporter

TIME: 1440

MESSAGE: "This is a Drill"

This is Chuck Checkner, WIBW Topeka. I understand radioactive iodine may be released from Wolf Creek. Am I correct in saying this may be deadly to children? How much radioactive iodine can a child be contaminated with before its deadly? How old does someone need to be before the iodine doesn't affect them? What are you doing to protect these poor kids?

MI-45 TO: Media Inquiry

FROM: Reporter

TIME: 1445

MESSAGE: "This is a Drill"

Connie Ayer, Washington Herald. We've heard about a General Disaster or something like that at your atomic power plant. Was this an explosion or something? How many people were killed? Is this a Chernobyl-type accident? Will the radioactivity kill all the wheat and cattle your Kansas farmers grow? We'd like to send a staff writer out to interview some victims of the disaster - what's the bestway to get there from New York?

MI-46 TO: Media Inquiry

FROM: Reporter

TIME: 1450

MESSAGE: "This is a Drill"

This is Bart Starr with the Kansas City Times. I'd like to get some information out about how people can express their concerns about the nuclear accident to KG&E. Are you setting up a special "hotline" for citizens who are calling in about concerns or if their property has received radiation damage, legal problems, liability concerns and concerns about relatives, friends working at or living near the plant? If so, I'd like a brief description or what the proper procedures are for filing damage claims and what will be considered as a legal claim for damages?

MI-47 TO: Media Inquiry

FROM: Reporter

TIME: 1455

MESSAGE: "This is a Drill"

I'm Sam Davidson with ABC news in New York. We're planning to do a news update on the nuclear plant accident. Has much radiation escaped the plant? How many people were involved in the evacuation? Will the Price Anderson Act pay for all this or does your company have some liability in it?

MI-48 TO: Media Inquiry

FROM: Reporter

TIME: 1500

MESSAGE: "This is a Drill"

I'm Martin Rosenthal, Kansas City Star. Can you tell me how many billions of dollars the Wolf Creek disaster will cost KG&E ratepayers? Will the Federal government kick in funds for this? Will Western Resources and KCPL go bankrupt, and if so, what will happen to your customers - who's going to provide electricity?

MI-49 TO: Media Inquiry

FROM: Reporter

TIME: 1505

MESSAGE: "This is a Drill"

John D. Marton here with KLTM Radio in Topeka. One of the residents in the Coffey County area told us that Western Resources will be receiving General Disaster Funds to purchase homes, crops, animals and land that has been damaged by radiation. Where do residents need to go or call to file a claim? Will there be special forms for them to fill out?

Note: Controller coordinate with Lead Controller prior to placing call so misinformation can be tracked.

MI-50 TO: Media Inquiry

FROM: Reporter

TIME: 1510

MESSAGE: "This is a Drill"

This is Candy Harder from WIBW-TV in Topeka. We'd like to do a special broadcast at 6:00 p.m. regarding KG&E's offer to purchase property exposed to radiation during the accident at Wolf Creek. Where will the funds come from to pay for this property? What will KG&E pay for an acre of land, chickens, cows, homes? Will it be on an individual basis? Please provide any details you may have.

Note: Controller coordinate with Lead Controller prior to placing call so misinformation can be tracked.

MI-51 TO: Media Inquiry

FROM: Reporter

TIME: 1515

MESSAGE: "This is a Drill"

I'm Janna Goode with KVOE Radio. What's the latest on the accident?

MI-52 TO: Media Inquiry

FROM: Reporter

TIME: 1520

MESSAGE: "This is a Drill"

This is Joan Burnstorf, Wichita Eagle speaking. What will the accident at Wolf Creek do to our electric bills? What about availability of service - will KG&E have enough electricity to serve their customers? Where will the electricity come from if not Wolf Creek? Are the ratepayers going to have to still pay for the construction of Wolf Creek even if it is not producing electricity?

MI-53 TO: Media Inquiry

FROM: Reporter

TIME: 1525

MESSAGE: "This is a Drill"

Hi! Jeff Carnes from KVOE Radio. Can I interview someone in your legal or insurance department in regard to what KG&E's liability is after the Wolf Creek disaster? Can they give me information on where people can call to file claims or lawsuits?

MI-54 TO: Media Inquiry

FROM: Reporter

TIME: 1530

MESSAGE: "This is a Drill"

J. R. Rawlins here from Memphis Courier. I understand Wolf Creek has the plant under control and the radiation release has stopped. Any idea what caused the accident? Any idea when the plant will produce electricity again? When will the people be able to return home? Can I find out more information from you tomorrow? Will this special number for Media Inquiry still be the right one to call tomorrow? Who do I ask for?

MI-55 TO: Media Inquiry

FROM: Reporter

TIME: 1535

MESSAGE: "This is a Drill"

Jim Johnson with the Dallas Times. Any idea about how long this accident at Wolf Creek is going to last? How long after it is under control can people start to go back to their homes? I'm doing a story on the plant and what exactly happened inside the plant. Do you have any diagrams about what happened inside? Can you give me an overview of what caused the accident?

MI-56 TO: Media Inquiry

FROM: Reporter

TIME: 1540

MESSAGE: "This is a Drill"

This is Louise Simpson from Wichita Radio Station KEYN. Is the accident over? What happens after people are evacuated? Will they be able to return home?

MI-57 TO: Media Inquiry

FROM: Reporter

TIME: 1545

MESSAGE: "This is a Drill"

Rolland Headley, KMBC Radio. I'd like to run a tape for our 6:00 news - can you do the interview? OK, good. We've gotten word of an evacuation in the Wolf Creek area. Will these people ever be able to return to their homes? What about the cropland - how many thousands of years will it be before farmers can again grow food there? Will the cattle and livestock in the area be killed by radiation? How far away has the wind carried this deadly radioactive cloud? That's all for now.

PC-1 TO: Public Concern Phone Team

FROM: Citizen

TIME: 0940

MESSAGE: "This is a Drill"

My name's Ben Ballingston, and I live in New Strawn. I've been picking up parts of conversation on my ham radio about a "turbine trip" at Wolf Creek. What are they talking about? Are we in trouble out here?

PC-2 TO: Public Concern Phone Team

FROM: Citizen

TIME: 0945

MESSAGE: "This is a Drill"

I'm Lois Fletcher and I heard on KEYN radio that there is a fire at Wolf Creek. Will this contaminate the state, like the fire at Chernobyl did?

PC-3 TO: Public Concern Phone Team

FROM: Citizen

TIME: 0950

MESSAGE: "This is a Drill"

Don Bunker, Bulkon Chemical, here. We're scheduled to deliver a tanker of chlorine to Wolf Creek today. Heard on the radio about your fire - do you still what us to deliver this stuff?

PC-4 TO: Public Concern Phone Team

FROM: Citizen

TIME: 0955

MESSAGE: "This is a Drill"

This is Dwight Goodall and I been a KG&E customer for years. My electric bill keeps going up and up and now you'll probably have to spend all kinds more money to clean up this mess at your nuke plant. What am I 'sposed to do? I just can't pay more to you people.

PC-5 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1000

MESSAGE: "This is a Drill"

This is Mrs. Alvin Smotzer and I live in Wichita. They said on the radio there's been a problem at that nuclear power plant. Will we have to leave our homes like those people in Russia did? I've lived here 32 years, and I don't want to leave.

PC-6 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1005

MESSAGE: "This is a Drill"

Hello - This is Bob Bonner at Burlington and I have a question about the accident at Wolf Creek.

I don't really understand what is going on. Is there something wrong with the plant and are we in any danger here?

PC-7 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1010

MESSAGE: "This is a Drill"

I'm Mike Abrahanson at ABC Fire Protection Service. My company sells state-of-the-art fire protection systems for industrial use. I've heard about your fire at Wolf Creek, and wonder if I can see someone about getting one our systems installed, so you won't have to worry about fire again?

PC-8 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1015

MESSAGE: "This is a Drill"

I'm Randy Harwood, and I manage the truck stop at Beto Junction. Some truckers are asking if it's safe to drive south on highway 75 past the nuke plant. Can you tell me what I should tell them?

PC-9 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1020

MESSAGE: "This is a Drill"

I heard about the nuclear plant accident on the radio. I need to go to Ottawa on business, and I need to find out if its safe? My name is Al Kanfield, and I live in Coffeyville.

PC-10 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1030

MESSAGE: "This is a Drill"

Hello, I'm Paul Griffen, and I live in St. Joseph, Missouri. My daughter is with a group of high school students who are supposed to tour Wolf Creek power plant today. They left this morning early, and were supposed to be there by 10:00. I'm concerned about them because of the accident. Can you tell me if they will be allowed to go to the plant, or will the Sheriff's Department turn them back?

PC-11 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1040

MESSAGE: "This is a Drill"

My name's Diane Froche and my husband works in security at Wolf Creek. I've been trying to call him but I can't get through. I'm worried that he may have been hurt in the fire, because he's on the fire brigade. Can you tell me if he's okay - his name is Raymond.

PC-12 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1045

MESSAGE: "This is a Drill"

I'm David Wagner and I'm a customer and I want some answers. I tried to call Western Resources to find out what's going on at Wolf Creek but they cut me off. Who am I talking to now? What's your title? Do you have any information about the accident or can you take a message and have someone call me?

PC-13 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1050

MESSAGE: "This is a Drill"

I'm Lela Borden. We live about three miles west of Wolf Creek, and I can't find that little booklet with the emergency information in it. Can you tell me what we're supposed to do, and where to go if we have to evacuate?

PC-14 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1055

MESSAGE: "This is a Drill"

Hi - I'm not sure who I need to talk to, but I have a complaint. All I've heard this morning on the radio is news about Wolf Creek. They talk and talk but it always ends up that there isn't much information available. Why don't you people tell the reporters what's going on, so we ordinary people will know. They said the Russians wouldn't give out any information about their plant blowing up, but it sounds like you're doing the same thing.

PC-15 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1100

MESSAGE: "This is a Drill"

Hello, I'm JoAnn Greeley, and I live in Emporia. I'm very concerned about the accident at Wolf Creek. We're only 40 miles east of the plant. If the plant blows up like the one did in Russia, what are we supposed to do? I heard in Russia radiation went as far as a thousand miles from the plant. Will we lose our home like those Russian people did?

PC-16 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1105

MESSAGE: "This is a Drill"

I need to talk to someone with authority. My television set went out on me this morning and it's never given me a bit of trouble before. I was talking to my neighbor who's an electrician and he said he bet when that Wolf Creek plant went out, it caused a big power surge that blew out my TV. That TV's only 7 years old and never had a problem before, so I bet that's what happened. Who can I talk to to pay me for the damage to my TV?

PC-17 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1110

MESSAGE: "This is a Drill"

My name's Joey Littlejohn and I live just north of Lyndon. I just saw three highway patrol cars go flying past here on 75 - were they going to Wolf Creek? Will someone tell us what to do and where to go?

PC-18 TO: Public Concern Phone Team

FROM: Reporter

TIME: 1120

MESSAGE: "This is a Drill"

This is Woody Hambright, and I'm outdoor editor for the Parsons Sun. For a long time we've been trying to get you to open Wolf Creek lake for fishing. Has this accident dumped any radioactive chemicals into the lake? Will it still be fit for fishing? Do you know if there are any plans to open the lake for public fishing?

PC-19 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1130

MESSAGE: "This is a Drill"

My name's Horrace Ocram and I live a mile south of Lyndon on highway 75. When I tried to go down to I-35, the highway patrol wouldn't let me through--they said there'd been a chemical spill and it might be dangerous. Was this something to do with the accident at the nuclear plant? When will we be able to get through on the highway?

Note: Controller notify Lead Controller prior to placing call so rumor can be tracked.

PC-20 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1140

MESSAGE: "This is a Drill"

This is Dean Hanover at Kansas State University. I teach a class in nuclear physics here, and we've been studying how pressurized water reactors work. Can you tell me if the problem this morning at Wolf Creek involved a LOCA, or loss-of-coolant accident? Was this a small LOCA, or the big double-ended LOCA like the "worse case" scenarios call for? Have you seen a buildup of containment pressure in response to the LOCA? Could I be included on the distribution for the technical analysis of this when it's all over?

PC-21 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1050

MESSAGE: "This is a Drill"

Hello, I'm calling about the Wolf Creek accident, is this the right number? My husband went hunting at Melvern lake this morning with my son, and I don't have any way to reach them. Are they safe? Will the sheriff or someone notify people around the lake if they need to get away from the radiation?

Note: Your name is Lisa Michaels from El Dorado if asked.

PC-22 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1200

MESSAGE: "This is a Drill"

My name is Allen Mason and I live in Kansas City. I have a daughter who lives with her mother in Burlington. The phone exchange is tied up and I can't get through.

Can you tell me if the people in that area have had to evacuate or if any of the people are in danger? Is the State in charge of this kind of emergency? Who else can I call to find out what's going on?

PC-23 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1210

MESSAGE: "This is a Drill"

This is Dr. Frank Masetta, and I'm an oncology specialist at Wesley Hospital. I'd like to volunteer my services to you in case you have injuries out at Wolf Creek from radiation exposure. My phone number is 683-0000 - just call me and I'll be ready to go anytime.

PC-24 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1220

MESSAGE: "This is a Drill"

Yes, is this where I call to find out about the Wolf Creek disaster? I'm Saul Epskin and I'm calling from Abilene. My son and daughter-in-law live in Lebo, down near Wolf Creek. I can't reach them on the phone because the lines are all tied up. Our radio just said you upgraded the emergency, and people may be evacuated. Is there any chance the radiation could go towards Lebo?

PC-25 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1230

MESSAGE: "This is a Drill"

KG&E? My name is George Ballinger and I live in Lebo. My neighbor and I keep smelling this real strange odor. Could we be getting exposed to some of that Wolf Creek radioactivity? Should we go to Emporia to the hospital to be checked out, just in case?

PC-26 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1240

MESSAGE: "This is a Drill"

This is Mrs. Mable Horshmann, and I live up by the Wolf Creek plant. I heard on the radio that some poison cloud might come out of the plant out here and make us leave our homes. I'm 87 years old and I don't drive and my husband is gone to town. There ain't any way for me to leave, and I'm afraid I'll die here from this. Please...can you help me?

(If asked, you live in the old Simpon place - 2 miles north and a mile east of Waverly)

PC-27 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1250

MESSAGE: "This is a Drill"

Hello, I'm Bernard White, and I'm principal at the Ottawa Senior High School. We're having PE classes and I was wondering if it is safe for students to be outside, with this Wolf Creek thing going on?

PC-28 TO: Public Concern Phone Team

FROM: KG&E Employee

TIME: 1300

MESSAGE: "This is a Drill"

Yeah, this is Rock Jones, and I'm in the line department at Fort Scott. We've been hearing rumors that they're going to make us work in the switchyard at Wolf Creek while this accident is going on - we heard they're doing something with one of the emergency busses and I guess they can't figure it out. None of us wants to go get nuked at that place - will we lose our jobs if we don't go?

PC-29 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1310

MESSAGE: "This is a Drill"

My name is Clinton Sanderstone and I live three miles due east of Wolf Creek. I have 65 dairy cows. What can I do to protect them from radiation. If they die or give off radioactive milk, it'll ruin me - we're just hanging on now. What can I do?

PC-30 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1320

MESSAGE: "This is a Drill"

I'm Cassandra Maxwell and I live in Emporia. Someone just came to our house and said he was a Wolf Creek employee. He said you are going to have to evacuate Emporia later today because of the radiation from Wolf Creek, and that unless we bought this filter he was selling we'd have to leave the house for at least a week. He said this filter would keep out the radioactive particles - he said it was treated with anti-radiation materials and would neutralize the particles. He said we should put the filter in our furnace and it would take care of everything. Then he tried to charge us $250 dollars for it. Why, it looks just like an ordinary furnace filter to me. I didn't buy it, but I started wondering if this guy was on the level. Are you going to evacuate Emporia? Are you really selling a radiation filter that would protect us from the radiation?

Note: Controller notify Lead Controller prior to placing call so rumor can be tracked.

PC-31 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1330

MESSAGE: "This is a Drill"

Good morning. This is Doyce Atwater at Wichita Iron & Metals Company. We have gotten news of the accident at the power plant and are wondering if we will lose electrical power as a result.

We are about to start running a batch of material and low power would ruin it and cost us a lot of money.

Can you help me or refer me to someone?

(If asked where your business is located, tell them at 922 Merton)

PC-32 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1340

MESSAGE: "This is a Drill"

Hi, this is Ron Fantin. My sister and her family live in New Strawn, across the road from Wolf Creek. No one answers at that number. Can you tell me have they moved people out of the area or something?

PC-33 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1350

MESSAGE: "This is a Drill"

Hello - KG&E? I hear on the news that your nuclear plant is going wild and giving off a lot of radiation.

If your plant blows what should I do to protect myself or am I in any danger at all this far away? My name is Peter Zeller, and I live in Ottawa.

PC-34 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1400

MESSAGE: "This is a Drill"

This is Dr. Jan Stephenson from the Psychology Department at Emporia State. I'm concerned about the psychological impact of the Wolf Creek accident on Coffey County residents. Do you have a company psychologist I can talk to?

PC-35 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1405

MESSAGE: "This is a Drill"

This is Dr. Bradley Bone from the Anthropology Department at Wichita State University. We have some artifacts in an exhibit on loan to Coffey County Museum in LeRoy. With the Wolf Creek accident and all, I'm very concerned about this exhibit - some of the artifacts are very rare and irreplaceable. Can you have someone move that exhibit out of there, just in case some radiation escapes and contaminates the town?

(If asked, the exhibit is in two show cases, each 6 feet long by 3 feet wide by 7 feet tall).

PC-36 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1410

MESSAGE: "This is a Drill"

This is Dr. Frank Geoffrey, and I'm head of radiological research at the University of California Medical Center. Part of our research program deals with the effects of radiation on humans. I understand you've had some people exposed out there at your disabled nuclear plant, and I'd like to go through whatever channels it takes to get in touch with those people, to arrange a long-term study of the effects on them. Who should I talk to to set this up?

PC-37 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1415

MESSAGE: "This is a Drill"

I'm Donna Reece and I live in Lebo. Is it safe for me to drive down to my sister's place in Yates Center? I usually go on 50 highway to highway 75, then south through Burlington.

PC-38 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1420

MESSAGE: "This is a Drill"

This is Lionell Jamison. I'm press secretary for Senator Bob Knoll. We are trying to get more information on the Wolf Creek accident - we've already been briefed by the NRC and FEMA, but wanted to see what you people could tell us. Can you give me a summary of what's happening?

PC-39 TO: Public Concern Phone Team

FROM: City of Wichita

TIME: 1425

MESSAGE: "This is a Drill"

This is Mike Everhard in the Environmental Services Department with the City of Wichita. We want to be ready to do some radiation monitoring here in case there is a major release from Wolf Creek. Can I speak to one of your environmentalists or health physicists to get some source term information?

PC-40 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1430

MESSAGE: "This is a Drill"

I'm calling from Fort Scott. My husband works for KG&E in the Line Department here, and he called earlier and said he was afraid they'd make him go work at Wolf Creek today. Do you know if anyone from Fort Scott has been sent up there. My name's Sally Jones.

PC-41 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1435

MESSAGE: "This is a Drill"

The news just said this accident at Wolf Creek may be the worst in history. This scares me - I live in Wichita, which isn't that far away. Is there a chance we may get radioactive fallout here?

PC-42 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1440

MESSAGE: "This is a Drill"

Hi, I'm Conrad Schraeder and I live at 702 W. 2nd in LeRoy. The sky looks kind of hazy over towards Wolf Creek. Is that radioactivity I see? Should I be doing something to protect myself?

PC-43 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1445

MESSAGE: "This is a Drill"

Hey, is this where I can get some answers about Wolf Creek? We were just having lunch, and started talking about this thing - who's going to pay for cleaning up? I mean, we've been paying these high electric bills because it cost so much to build that plant - now are our bills going to go up again because of this? I don't think that's right if they do - it's KG&E's problem, let them pay for it!

PC-44 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1450

MESSAGE: "This is a Drill"

I'm Carl Long and I work nights at IBP in Emporia. I just got out of bed, and I'm hearing all this about Wolf Creek on the radio. What's going on? I have a friend who works out there - have any employees been hurt? Do you think there will be a chance Emporia could be evacuated because of the radioactive fallout?

PC-45 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1455

MESSAGE: "This is a Drill"

Is this the number to call for information about the Wolf Creek disaster? I live in Yates Center which is about 20 miles from Burlington. How can I tell if any radioactive stuff is spreading down here? Can you see the radiation or taste it? Is it sticky? Is there some kind of antidote I can take if the fallout comes down here? I heard once that iodine is the worst thing to breathe in - does it burn when you breathe it, or how can you tell? Is Wolf Creek going to send someone down here to tell us what to do?

PC-46 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1500

MESSAGE: "This is a Drill"

My name's Joline Robins. I have a restaurant just off I-35 in Lebo. We're about 20 miles from Wolf Creek, and that's pretty close. My customers keep asking if we think there's any danger of radiation coming here. I'm not sure what to tell them. Is there any danger to us up here?

PC-47 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1505

MESSAGE: "This is a Drill"

This is Don Wacker, and I live six miles south of Waverly. About this Wolf Creek thing - I ain't about to leave my place just because some radioactivity. Hey, I was in the Army when they bombed them Bikiri Islands, and we weren't more than ten miles from the bomb. I figure I ain't dead yet and I ain't gonna be. They ain't gonna try to make me move out are they?

PC-48 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1510

MESSAGE: "This is a Drill"

I'm Alice St. John and I'm chairperson for Consumers Against Radioactive Plants. We at CARP have always said these nuke plants aren't safe - now maybe you'll listen. Anyway, I want to know who's going to pay to clean this mess up? If it's going to be the ratepayers, we're going to start building our case against it right now!

PC-49 TO: Public Concern Phone Team

FROM: Reporter

TIME: 1515

MESSAGE: "This is a Drill"

Hello, this is Marsh Roberts of CBS news. Is this the Media Release Center? Where is it located? If we fly into Topeka, can you give us directions how to get from the airport to the Media Release Center? What hours will it be open?

PC-50 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1520

MESSAGE: "This is a Drill"

I just heard the end of a news broadcast about Wolf Creek. They said something about not evacuating the Wichita area. Was there ever a chance Wichita would be evacuated? I don't know where we'd go or how to get there - no one's ever said anything about it to us.

PC-51 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1525

MESSAGE: "This is a Drill"

I'm Donna Marconi with the Sierra Club in Kansas City. We're very concerned about how this Wolf Creek catastrophe will affect the environment. Will there be miles of land around the plant that will be unusable for hundreds of years? Is there any way to decontaminate ground that has been contaminated? Is the Environmental Protection Agency in on this?

PC-52 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1530

MESSAGE: "This is a Drill"

This is Jim James, and I live about two miles from the nuclear plant. I think I heard a explosion a few minutes ago. Did something else happen at the plant? I called a couple radio stations, but they didn't know anything. Was there another explosion over there?

Note: Contact Lead Controller before placing call. Rumor to track.

PC-53 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1535

MESSAGE: "This is a Drill"

My name is Joe Jones. I farm and raise cattle in the area evacuated. Who will pay for the damages to my crops and animals? Wolf Creek or my insurance? Do I need to contact my insurance agent for him to assess the damage? Then, do I send my claim to Wolf Creek?

PC-54 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1540

MESSAGE: "This is a Drill"

This is Bert Douglas. I heard on the radio that Wolf Creek is buying the property of those people affected by the nuclear plant accident. I own some land out there, and would like Wolf Creek to make me an offer for my land.

Note: Controller coordinate with Lead Controller prior to placing call so misinformation can be tracked.

PC-55 TO: Public Concern Phone Team

FROM: Citizen

TIME: 1545

MESSAGE: "This is a Drill"

This is John Sampson. Do you have a list of the names of people who were evacuated and if they are all okay? My parents live in that area, and I haven't been able to get a hold of them. Can you tell me where they are, and how can I reach them?

MM-1 TO: Media Monitoring

FROM: Radio

TIME: 0900 (or 5 minutes after team is activated)

MESSAGE: "This is a Drill"

We interrupt regular programming for this special report from KFH news. A serious emergency seems to be developing at Wolf Creek nuclear power plant near Burlington, Kansas. Wolf Creek Nuclear Operating Corporation officials notified us of a fire at the plant earlier this morning. The situation has apparently worsened, and the plant is now on emergency alert. Plant officials say no deadly radiation is coming out of the plant yet, but area residents are being advised to be ready to take protective action. KFH news will interrupt programming again as soon as details are available on this potentially serious situation.

MM-2 TO: Media Monitoring

FROM: TV

TIME: 0910 (or 5 minutes after team is activated)

MESSAGE: "This is a Drill"

I'm Kelly Waldo with a special TV 10 news bulletin. The controversial Wolf Creek nuclear power plant suffered yet another blow this morning when a fire broke out inside the plant. Plant officials first reported the fire was under control, but have since said the plant is on Alert and emergency procedures are being followed. Apparently no radiation has been released, but people living near the crippled plant are being warned that they may need to take protective action. We'll have more on the nuclear accident as details become available.

MM-3 TO: Media Monitoring

FROM: TV

TIME: 0920 (or 5 minutes after team is activated)

MESSAGE: "This is a Drill"

This is a special report from the TV-12 newsroom. Wolf Creek nuclear power plant in eastern Kansas is under an Alert emergency, after a fire broke out at the plant earlier today. Plant officials say there has been no release of radiation, and that emergency procedures are being followed. KWCH-TV is attempting to obtain further details, and will interrupt programming again when more information is available. I'm Don Williams, from the KWCH newsroom.

MM-4 TO: Media Monitoring

FROM: Radio

TIME: 0930

MESSAGE: "This is a Drill"

First with news, this is KXLK radio. Wolf Creek Generating Station is on Alert this morning, after fire and other problems crippled the plant. Wolf Creek Nuclear Operating Corporation says no radiation has been released from the $3 billion dollar plant, but emergency procedures are being followed.

MM-5 TO: Media Monitoring

FROM: Radio

TIME: 0940

MESSAGE: "This is a Drill"

Good morning, this is Ray Means, KLZS radio news. A serious accident occurred early this morning at Wolf Creek nuclear power plant. Details are sketchy but Kansas Gas and Electric officials tell us the plant is on Alert status, which is the second level of four emergency classifications. A fire was reported to have started at the plant at 8:00 this morning. There are unconfirmed reports that three workers were seriously injured either by fire or by radiation - that is an unconfirmed report which we are attempting to verify. We'll have updates on the Wolf Creek disaster later, as more information becomes available.

Note: Controller notify Lead Controller prior to placing call so this rumor can be tracked.

MM-6 TO: Media Monitoring

FROM: Radio

TIME: 0950

MESSAGE: "This is a Drill"

This is Louise Simpson, KEYN news - people living near the Wolf Creek nuclear power plant in Coffey County are on the edge of their seats this morning waiting for news of how the emergency at the plant is being handled. Kansas Gas and Electric company is saying little about the situation, other than the plant is on Alert status and no radiation has been released. Plant officials are currently following emergency procedures. We'll provide more information as it becomes available. This is Louise Simpson, KEYN news.

MM-7 TO: Media Monitoring

FROM: TV

TIME: 1000

MESSAGE: "This is a Drill"

This is a CBS newsbreak. I'm Roger Perkins. A serious problem is developing on the plains of Kansas this morning. The Wolf Creek Nuclear Power Plant, located 90 miles southwest of Kansas City, Missouri is on Alert following a fire which disabled the plant earlier this morning. Utility officials report the situation is under control, and that no radiation has escaped the plant. However, area residents are being advised to keep their radios tuned to the Emergency Broadcast System in case the situation worsens and an evacuation is necessary.

Wolf Creek began operation in September, 1985 amid a storm of controversy from critics who charged the plant was not safe. As recently as last July, Congressman Dan Classman of Kansas charged the Nuclear Regulatory Commission with "sweeping some things under the rug" in their review of plant safety and construction quality. Glassman was unavailable this morning to comment on the potential disaster at the plant. We'll have more on the story in a special report, "Nuclear Plants: Power or Poison?", tonight at 11:30 Eastern Standard Time.

MM-8 TO: Media Monitoring

FROM: TV

TIME: 1010

MESSAGE: "This is a Drill"

From the KAKE-TV newsroom, this is an update on the accident at the Wolf Creek nuclear plant near Burlington, Kansas. Informed sources at Kansas Gas and Electric tell us that fire which erupted at the plant this morning has been extinguished. However, the plant is still in an emergency situation. Details are still sketchy, but we do know that people within 10 miles of the plant are being prepared in case evacuation is necessary. We'll interrupt regular programming as needed when more information is available. I'm Kelly Waldo, KAKE news.

MM-9 TO: Media Monitoring

FROM: Radio

TIME: 1020

MESSAGE: "This is a Drill"

We interrupt regular KFFX programming for this special bulletin from the newsroom. We have an unconfirmed report that a tank truck possibly carrying radioactive material from the crippled Wolf Creek nuclear power plant has overturned three miles north of I-35 on U.S. 75 highway. The highway patrol has confirmed that U.S. 75 north of I-35 has been closed. However, the highway patrol could not confirm the report of radioactive contamination from the wrecked truck. Our KFFX mobile unit is on the way, and we should have a further report within a few minutes. This is Brad Oliver, KFFX news.

Note: Controller notify Lead Controller prior to placing message so rumor can be tracked.

MM-10 TO: Media Monitoring

FROM: TV

TIME: 1030

MESSAGE: "This is a Drill"

Good morning, this is Greg Roberts, KSN news. The controversial Wolf Creek Generating Station is in the news again this morning. Earlier, plant officials reported that a fire had started inside the plant, and that other problems later came up which caused plant operators to take emergency action. There have been no reports of injury from the accident, nor do we know of any releases of radiation from the crippled plant. We'll have more on the Wolf Creek emergency on KSN's news at noon.

MM-11 TO: Media Monitoring

FROM: TV

TIME: 1040

MESSAGE: "This is a Drill"

He...o, I'm Kelly Waldo with a KAKE TV newsbreak. Disaster struck the Wolf Creek nuclear power plant near Burlington this morning, when a fire broke out inside the plant. Plant officials tell us that the fire was extinguished, but that a water line break in the plant's reactor has created an emergency situation. Officials tell us that no radiation has escaped, but that plant workers are following emergency procedures. The beleagured power plant began operation in September 1985, after nearly ten years of controversy. We'll have more about the accident on KAKE's news at noon.

MM-12 TO: Media Monitoring

FROM: Radio

TIME: 1050

MESSAGE: "This is a Drill"

This is Ray Means, KLZS radio news. Leading the news this morning is the accident at Wolf Creek nuclear power plant in Eastern Kansas. The plant is on Alert this morning, which is the second level of emergency classification according to plant officials. A fire erupted at the plant earlier this morning, followed by a loss of coolant around the reactor core. Earlier reports of injuries have proved incorrect. Plant officials assure us no one was hurt in the accident. At last report, plant workers were continuing to follow emergency procedures. We'll have more news as it becomes available.

MM-13 TO: Media Monitoring

FROM: Radio

TIME: 1100 **(WAIT UNTIL SAE STATEMENT IS ISSUED)**

MESSAGE: "This is a Drill"

This is Louise Simpson from the KEYN newsroom - we have just received a news release from Wolf Creek Nuclear Operating Corporation notifying us that a Site Area Emergency has been declared at Wolf Creek nuclear plant. This is the second most serious emergency classification, and there is a good change some radiation may be released from the plant. Details are sketchy as of the moment, and we're trying to get through to plant officials to determine what this really means. It appears to be a very serious situation. We'll follow up with more information as soon as it's available - stay tuned to your news station, KEYN.

MM-14 TO: Media Monitoring

FROM: Radio

TIME: 1110

MESSAGE: "This is a Drill"

This just in from the KLZS newsroom - a Site Area Emergency has been declared at Wolf Creek nuclear power plant near Burlington, Kansas. This is the second most serious of four emergency designations for atomic power plants. Federal, state and local officials have been called in to help with the situation. Wolf Creek is providing few details about the impending disaster, but they are urging local residents to listen to the Emergency Broadcast System for possible evacuation instructions.

Again, a Site Area Emergency has been declared at Wolf Creek. This would appear to have the makings of the worst nuclear power accident ever to occur in the United States. Stay tuned to KLZS for more information as it becomes available.

MM-15 TO: Media Monitoring

FROM: Radio

TIME: 1120

MESSAGE: "This is a Drill"

This is Wyley Reed in the KFDI newsroom. A bad situation is getting worse at the Wolf Creek nuclear power plant near Burlington. Wolf Creek has upgraded this morning's Alert emergency to a Site Area Emergency, the second most serious accident classification. During a Site Area Emergency, uncontrolled releases of radioactivity may occur requiring evacuation of area residents around the plant.

Officials say people living within ten miles of the plant have been advised to stay alert, monitor the Emergency Broadcast System and read their Emergency Information booklets for instructions.

The accident-plagued Wolf Creek plant started operation in September of 1985. Today's disaster will almost certainly mark the worst nuclear power disaster in United States history, and may approach the magnitude of the mishap at the Chernobyl plant in the Soviet Union. Stay tuned to KFDI for more late-breaking news on the Wolf Creek disaster.

MM-16 TO: Media Monitoring

FROM: TV

TIME: 1130

MESSAGE: "This is a Drill"

Good morning, this is Don Williams with an update on the nuclear accident at Wolf Creek power plant. The situation at the plant continues to deteriorate this morning. Wolf Creek officials have declared a Site Area Emergency, the second most serious of four emergency classifications. We have received unconfirmed reports that state troopers have blocked off roads leading to within 20 miles of the plant, and no one is being allowed closer. Officials tell us that there is a possibility radioactivity will be released from the plant, and that area residents should be ready to take emergency action. Our Channel 12 news team is on its way to Topeka, where state, county and utility officials have set up an emergency communications facility. We'll have a live report from that facility on our KWCH news at noon. This is Don Williams, Channel 12 news.

MM-17 TO: Media Monitoring

FROM: TV

TIME: 1140

MESSAGE: "This is a Drill"

This is an NBC newsbreak with Roger Sand. The nuclear utility industry is anxiously watching developments at the Wolf Creek nuclear plant in eastern Kansas. An emergency situation that started with a fire at the plant this morning has escalated into what appears to be a very serious nuclear accident. Plant officials are warning area residents to be prepared in case an evacuation is necessary. This is the first serious nuclear power accident since the Chernobyl plant in the Soviet Union exploded in 1986, killing more than 30 persons and spreading contamination across the globe.

MM-18 TO: Media Monitoring

FROM: TV

TIME: 1150

MESSAGE: "This is a Drill"

We interrupt regular programming for a special news update on the emergency at Wolf Creek Generating Station. WCNOC officials have upgraded the Alert emergency declared earlier this morning to the more serious Site Area Emergency.

A Site Area Emergency is declared when radiation escapes from two of the three radiation barriers at the plant. Federal, state and county emergency response officials have been notified and are monitoring the situation. State officials have assured KSN any actions necessary to protect the public health and safety will be taken.

Citizens of the area who might be affected by the release of radioactivity will be alerted by sirens and by the special tone alert radios issued by Coffey County Emergency personnel.

KSN will keep you informed as the crisis unfolds. Stay tuned. This is Greg Roberts, KSN news.

MM-19 TO: Media Monitoring

FROM: TV

TIME: 1200

MESSAGE: "This is a Drill"

From the KAKE-TV newsroom, this is a special report: CRISIS AT WOLF CREEK. KAKE-TV has received word from KG&E that the emergency at Wolf Creek has been upgraded to a more serious SITE AREA EMERGENCY. Apparently some radiation is now leaking from the plant's radiation barriers. Federal, State and County disaster response officials have been alerted and are monitoring the situation.

Reports from persons living near the crippled plant say that there has been a marked increase in activity by emergency vehicles in the past hour. KG&E is urging persons living within 10 miles of the plant to tune to a local radio station for emergency information.

Again, a SITE AREA EMERGENCY has been declared at Wolf Creek. Conditions at the plant have worsened since the fire at the plant happened this morning. Wolf Creek is located in Coffey County about four miles north of Burlington. The plant is 55 miles south of Topeka and 90 miles southwest of the Kansas City metropolitan area. We'll have more about the Wolf Creek disaster on KAKE news at noon. This is Kelly Waldo reporting.

Note: Controller notify Lead Controller prior to placing call so rumor can be tracked.

MM-20 TO: Media Monitoring

FROM: Radio

TIME: 1205

MESSAGE: "This is a Drill"

The emergency continues to deteriorate at Wolf Creek nuclear power plant in southeastern Kansas. KG&E officials have declared a SITE AREA EMERGENCY, the second most serious of four emergency classifications. We have received unconfirmed reports of emergency workers in the area around the plant testing for radioactive fallout. If these reports are true, it would seem the health and safety of anyone in the area could be in jeopardy.

State troopers have blocked off roads leading to the plant, and no one is being allowed any closer. We have no confirmed reports of radiation leaking from the plant, but that's certainly a possibility during this serious mishap. Area residents are being urged to listen to their radios for Emergency Broadcast System instructions. We'll try to have further details later in this WIBW radio newscast.

MM-21 TO: Media Monitoring

FROM: TV

TIME: 1210

MESSAGE: "This is a Drill"

This is Kelly Waldo at KAKE Monday News. Just how disasterous will it be? That's the question being asked in Kansas and throughout the world in the wake of a major nuclear power accident at Wolf Creek Generating Station. Events began to unfold at just before 6:00 this morning, when fire broke out inside the nuclear plant. Just when that problem appeared to be solved, plant officials report that coolant began to leak from the plant's reactor. Indications are the deadly uranium fuel inside the reactor may soon reach its melting point and cause a release of radiation to the environment. Wolf Creek is now under a Site Area Emergency, and local residents may soon be evacuated. We have a news team setting up in the Wolf Creek media center in Topeka. We hope to have an update from them by the end of this newscast.

MM-22 TO: Media Monitoring

FROM: TV

TIME: 1220

MESSAGE: "This is a Drill"

Hello, everyone. I'm Don Williams, and this is KWCH news at noon. The unthinkable is now happening at Wolf Creek nuclear plant, located 60 miles south of Topeka. The plant has declared a Site Area Emergency, wich is the second most serious of four emergency classifications. Multiple failures of the plant's safety systems has lead to a serious situation, and officials say a release of radiation is possible.

We were unable to arrange an on-camera interview with a Wolf Creek representative. However, we havge arrange to visit Mr. Clyde Widener, from the Union of Concerned Scientists. Mr. Widener, what do you think is going on at Wolf Creek now?

[Switch to 2nd Controller for "Widener"]

"Well, Don, it's hard to tell. I would expect a great deal of uncertainty, bordering on panic, among the plant operators. They realize the instruments they count on to run the plant may not function properly under emergency conditions. And they know if they make one false move, they could actually contaminate half the county or more. I'd say it's probably a real tense situation - I'm glad I'm not there!"

[Switch back to 1st Controller]

Thank you Mr. Widener. Again, there is a Site Area Emergency at Wolf Creek. Anyone living within ten miles of the plant is urged to listen to Emergency Broadcast System channels for evacuation instructions. We'll have more on the disaster later this afternoon.

MM-23 TO: Media Monitoring

FROM: TV

TIME: 1225

MESSAGE: "This is a Drill"

It could be the worst disaster in Kansas history. This is Greg Roberts, KSN news, and that's what experts are saying about the nuclear accident at Wolf Creek. County, state and federal officials have joined Wolf Creek employees to try and regain control of the nuclear power plant. At last report, Wolf Creek officials had declared a Site Area Emergency, the second most serious emergency classification. Residents of Coffey County are being prepared to evacuate their homes should radioactivity begin spewing from the plant.

NBC news will have a special report tonight at 10:30 Wolf Creek: A Sunflower Disaster will examine the controversial construction of the plant, and look at what may have gone wrong today.

MM-24 TO: Media Monitoring

FROM: Radio

TIME: 1235 **(Wait until General Emergency release is issued)**

MESSAGE: "This is a Drill"

This is a special report from KKRD. Federal, State and Coffey County officials have been mobilized to prepare for emergency actions near Wolf Creek nuclear power plant in Coffey County. KG&E has declared a General Emergency, which means clouds of radioactivity could be released from the nuclear reactor at any time. This is an accident very similar to what happened at the Chernobyl plant in the USSR last spring. Concerns during a massive release of radiation first center on children in the area who could suffer terribly if exposed to only minute doses of the deadly plutonium fallout.

While we have no confirmation of any life-threatening releases of radiation from the plant, listeners are urged to stay tuned to their radios for possible instructions from the Emergency Broadcast System. We'll be back with more details on this disaster in a few minutes.

MM-25 TO: Media Monitoring

FROM: TV

TIME: 1245 **(Wait until General Emergency release is issued)**

MESSAGE: "This is a Drill"

This is Kelly Waldo, KAKE TV news. A General Emergency has been declared at Wolf Creek Nuclear Operating Corporation's Wolf Creek nuclear power station. Officials report that a hydrogen explosion occurred at the plant about an hour ago, and significant amounts of radioactivity is or may soon be released. Persons living within ten miles of the crippled plant are urged to listen to an Emergency Broadcast System station for evacuation instructions. Again, a General Emergency has been declared at Wolf Creek. This is a very serious situation, and is especially dangerous for anyone in the Coffey County area. We expect more information within a few minutes from our reporter at the emergency media facility in Topeka. We'll interrupt programming again when that report is available.

MM-26 TO: Media Monitoring

FROM: Radio

TIME: 1255 **(Wait until General Emergency release is issued)**

MESSAGE: "This is a Drill"

From the KXLK radio news room, this is Sam Ware. A General Emergency has been declared at the Wolf Creek nuclear power plant in eastern Kansas. Officials from Wolf Creek made the announcement a few minutes ago in a news release from their emergency media facility. This is the most serious of four emergency classifications, and radiation is or will soon be leaking from damaged palnt. We have no indication of any deaths yet as a result of the accident. KXLK has learned that emergency officials from the State and Coffey County have joined in preparing to evacuate residents living within ten miles of the plant. No plans are yet being made to evacuate anyone outside this ten-mile emergency zone. Wolf Creek has been the center of controversy since construction began in 1977, but this is certainly the most serious problem ever encountered because of the plant. KXLK news is monitoring the situation closely, and will provide more details as they become available.

MM-27 TO: Media Monitoring

FROM: TV

TIME: 1305 **(Wait until General Emergency release is issued)**

MESSAGE: "This is a Drill"

This is Greg Roberts in the KSN newsroom, and this is a special report on the Crisis at Wolf Creek nuclear power plant. We've just received word that a General Emergency has been declared at Wolf Creek due to a hydrogen explosion in the reactor building. Unconfirmed reports from witnesses at the scene indicate that at least a third of the reactor building's doom roof has been blown off. Again these are unconfirmed reports. Roadblocks have been set up on all roads leading to the plant, and our reporters cannot get close enought to the plant for a first-hand look. If indeed these reports are true, this would indicate a condition far more serious than Wolf Creek officials are telling us - if there is a hole in the reactor building, we could be looking at a radioactive release on the scale of the Chernobyl accident in the Soviet Union. We'll have more in a few minutes from our mobile unit now in the Coffey County area.

Note: Controller notify Lead Controller before placing call so rumor can be tracked.

MM-28 TO: Media Monitoring

FROM: TV

TIME: 1315 (12:38 p.m.)

MESSAGE: "This is a Drill"

This is Don Williams, KWCH news with an update on the disaster at Wolf Creek. We now have made contact with our mobile satellite unit in the vicinity of Wolf Creek - let's switch to Wes Huntley at the scene.

[Change to different "reporter"]

This is Wes Huntley, and I'm standing in a field about a mile and a half to the northeast of the Wolf Creek plant. Roads leading to the plant have been blocked by the highway patrol, but we were able to get a closer look at the plant by taking our four-wheel-drive vehicle across country. At this time, things look quite normal at the plant. There is no evidence from this viewpoint of any damage to the reactor building. There's no smoke or any evidence of radiation escaping from the building. We have a geiger counter here at the truck and it doesn't read anything out of the ordinary, so apparently there has been no radiation release to this point. We have seen several groups of people at various locations in the area, presumably measuring for radioactivity. We'll try to get an interview with one of those groups if we can find one close by. This is Wes Huntley, reporting from near Wolf Creek nuclear plant.

Note: Controller notify Lead Controller before placing this message.

MM-29 TO: Media Monitoring

FROM: Radio

TIME: 1325

MESSAGE: "This is a Drill"

This is KEYN news update, I'm Louise Simpson. Shocking and disgusting - that's how Congressman Don Glassman described the accident at Wolf Creek nuclear plant. Glassman, a long-term critic of the project, said he plans to put pressure on the Nuclear Regulatory Commission to determine the exact cause of the accident. Meanwhile, Wolf Creek officials say the plant is still under a General Emergency.

MM-30 TO: Media Monitoring

FROM: TV

TIME: 1335

MESSAGE: "This is a Drill"

This is a CBS newsbreak, I'm Roger Perkins. What may be the worst nuclear power accident in U.S. history is now taking place at Wolf Creek Generating Station in eastern Kansas. A General Emergency, which is the most serious emergency classification, has been declared. Meanwhile in Washington, Congressman Don Glassman of Kansas is calling on the Nuclear Regulatory Commission to begin an investigation into the cause of the accident. We'll have more news later today.

MM-31 TO: Media Monitoring

FROM: Radio

TIME: 1345

MESSAGE: "This is a Drill"

This is Wyley Reed with a KFDI news update. Still no progress in solving the problems at Wolf Creek nuclear plant. Plant officials say there is still a General Emergency, meaning radioactive material may be escaping from the crippled reactor. Residents of Coffey County are being told to listen to Emergency Broadcast System stations for evacuation instructions. We'll have more on the crisis as it develops.

MM-32 TO: Media Monitoring

FROM: Radio

TIME: 1350

MESSAGE: "This is a Drill"

This is Norman Grant of KLZS Radio. The big story today has been the emergency at the Wolf Creek nuclear plant. We contacted a team of Wolf Creek employees at the Wichita, Kansas Rumor Control Hotline. Specially trained employees have assumed their emergency roles at Wolf Creek near Burlington, in Wichita, and at a media information center in Topeka. Assisting Wolf Creek in responding to the country's worst nuclear disaster are emergency teams from Coffey County, Kansas State, and the Nuclear Regulatory Commission.

MESSAGE NO: CEOC-6

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Coffey County Radiological Officer

FROM: Fire Leader

LOCATION: County EOC

TIME: 1230 (H+04:30)

MESSAGE: If we need to do decontamination at the access control points, we will need more water supplies then we have. Can you locate some water trucks?

NOTE: DO NOT TAKE ANY ACTION other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: CEOC-7

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Sheriff

FROM: Storekeeper

LOCATION: County EOC

TIME: 1300 (H+05:00)

MESSAGE: This is Brad Simmons - I own the drugstore here and I want to know just what kind of protection against looters I'll have if I leave. Are any of your men going to be guarding the stores?

NOTE: DO NOT TAKE ANY ACTION other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: CEOC-8

THIS IS A DRILL

<u>DO NOT</u> initiate actions affecting normal plant operations.

TO: County Commissioners

FROM: Senator's Aide

LOCATION: County EOC

TIME: 1330 (H+05:30)

MESSAGE: Senator Dan Glickman would like an update from County government on the status of local efforts to deal with the Wolf Creek situation. Do you require any assistance from Senator Glickman as far as calming down the public?

NOTE: **DO NOT TAKE ANY ACTION** other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: CEOC-9

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: County Agent

FROM: Farmer

LOCATION: County EOC

TIME: 1345 (H+05:45)

MESSAGE: I want help getting some stored feed for my dairy cows. My name is Herbert Rogers, my farm is just outside of Lebo, and I need that feed as soon as possible.

NOTE: DO NOT TAKE ANY ACTION other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: CEOC-10

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Shelter Systems Officer

FROM: Al' County EPC

LOCATION: Cou. EOC

TIME: 1400 (H+06:00)

MESSAGE: We are getting calls from evacuees that claim we are supposed to take care of th . What gives? I thought that Allen County was no longer a h nty. There must be 20 or 30 people here now that want lod nd meals because they were told to evacuate.

NOTE: DO NOT TAKE ANY ACTION other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: CEOC-11

THIS IS A DRILL

<u>DO NOT</u> initiate actions affecting normal plant operations.

TO: Health Medical Management Team

FROM: Waverly Nursing Home

LOCATION: County EOC

TIME: 1415 (H+06:15)

MESSAGE: We have had Burlington folks stop by to visit relatives. What is the chance of these evacuees contaminating patients?

NOTE: **DO NOT TAKE ANY ACTION** other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: CEOC-12

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Shelter Systems Officer

FROM: Host County

LOCATION: County EOC

TIME: 1500 (H+07:00)

MESSAGE: Your RO is to dispose of contaminated items. When will he get this stuff? We have several bags and people are getting concerned.

NOTE: **DO NOT TAKE ANY ACTION** other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: SEOC-1

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Director or Deputy Director KDEP

FROM: Director, FEMA, Region VII (Lead State EOC Controller)

LOCATION: State EOC

TIME: 1030 (H+02:30)

MESSAGE: What is the current status of the emergency at the power plant? Should we anticipate your requesting assistance from us and if so when? I would like to get my people prepared if you feel this situation will worsen. What do you think?

NOTE: **DO NOT TAKE ANY ACTION** other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: SEOC-2

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: National Guard

FROM: Kansas Department of Wildlife and Parks

LOCATION: State EOC

TIME: 1100 (H+03:00)

MESSAGE: How quickly can you get a helicopter out to Otter Creek? We've found a hunter who's apparently having a heart attack and it will take hours to get him out. What can you do to assist?

NOTE: DO NOT TAKE ANY ACTION other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: SEOC-3

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Director or Deputy Director KDEP

FROM: Director, FEMA Region VII

LOCATION: State EOC

TIME: 1230 (H+04:30)

MESSAGE: We have had several calls from the media about the emergency. It sounds like things are getting worse. I am going to make a news release in about 15 minutes. What is the current status of utility, state, and local efforts?

NOTE: DO NOT TAKE ANY ACTION other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: SEOC-4

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Kansas Department of Transportation

FROM: Goodwin Construction Company

LOCATION: State EOC

TIME: 1300 (H+05:00)

MESSAGE: We've got a real mess here on 75 just south of Lyndon. Our guys are doing some repair work and have one lane closed. There are about 80 cars backed-up that were trying to go south, and plenty more coming the other way. People are pretty nervous and going awfully fast because of the nuclear plant. I'm afraidwe're going to have a bad accident. What can you do to assist?

NOTE: **DO NOT TAKE ANY ACTION** other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: SEOC-5

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: National Guard

FROM: Coffey County Radiological Officer

LOCATION: State EOC

TIME: 1330 (H+05:30)

MESSAGE: Can you furnish water trucks for the Fire Department to use for decontamination at the access control points? How long would it take to get them here?

NOTE: **DO NOT TAKE ANY ACTION** other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: SEOC-6

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Kansas Department of Wildlife and Parks

FROM: Citizen

LOCATION: State EOC

TIME: 1400 (H+06:00)

MESSAGE: What is going to happen to the fishing in the Neosho River thanks to Wolf Creek? How far downstream are we going to have to go before we can be sure that the fish are safe to eat?

NOTE: DO NOT TAKE ANY ACTION other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: SEOC-7

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Kansas Highway Patrol

FROM: Governor's Office

LOCATION: State EOC

TIME: 1415 (H+06:15)

MESSAGE: We are getting calls from several communities about 50-60 miles southeast of Wolf Creek that are thinking about evacuating because of the accident at the plant. How many men can you get to that area to maintain traffic control in an hour? 2 hours?

NOTE: **DO NOT TAKE ANY ACTION** other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: SEOC-8

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Kansas Board of Agriculture

FROM: Governor's Press Secretary

LOCATION: State EOC

TIME: 1430 (H+06:30)

MESSAGE: We are getting calls from several communities northeast of Burlington that want to evacuate due to Wolf Creek. Should that occur, how long would it take to get some barricades set up to cordon off secondary roads and maintain traffic flow? How many people can you supply us with to assist in this effort?

NOTE: DO NOT TAKE ANY ACTION other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: SEOC-9

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Kansas Board of Agriculture

FROM: County Agent

LOCATION: State EOC

TIME: 1500 (H+07:00)

MESSAGE: Some farmers are asking me how long will it be before they can come back to their farms? Will anyone be responsible for milking their dairy cattle while the farmers are away?

NOTE: DO NOT TAKE ANY ACTION other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: SEOC-10

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Kansas Department of Health & Environment

FROM: Environmental Protection Agency

LOCATION: State EOC

TIME: 1515 (H+07:15)

MESSAGE: We're sending some of our people from Montgomery, Alabama and they'll be there in 24 hours. They'll be bringing their mobile van lab and will want to set it up in the closest possible location to the plume for sampling. Where can we set up?

NOTE: **DO NOT TAKE ANY ACTION** other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

MESSAGE NO: SEOC-11

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations.

TO: Kansas Department of Wildlife and Parks

FROM: U.S. Fish & Wildlife Service

LOCATION: State EOC

TIME: 1600 (H+08:00)

MESSAGE: This is Ed Harris from the Denver office of the U.S. Fish & Wildlife Service. We are very interested in how the accident at Wolf Creek will affect John Redmond Reservoir in the long run. Do you have any plans to do such a study? If not, is any other State agency planning to follow through?

NOTE: DO NOT TAKE ANY ACTION other than to coordinate a simulated response with the appropriate personnel.

THIS IS A DRILL

SECTION 5.0

PLANT DATA

Subsections | Page

PLANT PARAMETERS

Time-related plant parameters are provided in the following subsection. The data includes parameters for primary and secondary systems that may or may not have an impact on this scenario. The following subsection, "Plant Graphs" depicts the same data in graph form.

These parameters may be used as a source of data for control room (CR) operators by the CR Lead Controller, in case of simulator failure.

PLANT PARAMETERS

Time 0800 Relative Time (H+00:00)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	100	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	111	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	2297	-	-	-	-
Pressurizer Level (%)	60	-	-	-	-
Reactor Coolant System Temp. (^{O}F)					
T_C (wide-range)	-	558	558	558	558
T_H (wide-range)	-	618	618	618	618
T_{AVG} (narrow-range)	-	588	588	588	588
Core Exit Thermocouple Temp. (^{O}F)	599	-	-	-	-
Reactor Coolant Loop Flows (%)	-	100	100	100	100
Boron Concentration (ppm)	658	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	3805	3805	3805	3805
Steam Generator (WR) Levels (%)	-	67	67	67	67
Steam Generator Pressures (psig)	-	988	988	988	988
CONTAINMENT BUILDING					
Pressure (psig)	.4	-	-	-	-
Temperature (^{O}F)	80	-	-	-	-
Humidity (%)	33	-	-	-	-
Hydrogen Concentration (% vol)	0	-	-	-	-
Recirculation Sump Level (in)	0	-	-	-	-
TANKS					
Accumulator Levels (%)	-	59	59	59	59
Accumulator Pressures (psig)	-	645	645	645	645
Refueling Water Storage Tank Level(%)	99	-	-	-	-

PLANT PARAMETERS

Time 0815 Relative Time (H+00:15)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	100	-	-	-	-
Reactor Vessel L... (%) - Natural Circulation	111	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	2289	-	-	-	-
Pressurizer Level (%)	59	-	-	-	-
Reactor Coolant System Temp. (^{O}F)					
T_C (wide-range)	-	557	557	557	557
T_H (wide-range)	-	618	618	618	618
T_{AVG} (narrow-range)	-	587	587	587	587
Core Exit Thermocouple Temp. (^{O}F)	599	-	-	-	-
Reactor Coolant Loop Flows (%)	-	100	100	100	100
Boron Concentration (ppm)	657	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	3784	3761	3811	3784
Steam Generator (WR) Levels (%)	-	67	67	67	67
Steam Generator Pressures (psig)	-	979	979	979	979
CONTAINMENT BUILDING					
Pressure (psig)	.4	-	-	-	-
Temperature (^{O}F)	94	-	-	-	-
Humidity (%)	33	-	-	-	-
Hydrogen Concentration (% vol)	0	-	-	-	-
Recirculation Sump Level (in)	0	-	-	-	-
TANKS					
Accumulator Levels (%)	-	59	59	59	59
Accumulator Pressures (psig)	-	645	645	645	645
Refueling Water Storage Tank Level(%)	99	-	-	-	-

PLANT PARAMETERS

Time 0830 Relative Time (H+00:30)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	100	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	111	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	2301	-	-	-	-
Pressurizer Level (%)	60	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	557	557	557	557
T_H (wide-range)	-	618	618	618	618
T_{AVG} (narrow-range)	-	587	587	587	587
Core Exit Thermocouple Temp. (^{o}F)	599	-	-	-	-
Reactor Coolant Loop Flows (%)	-	100	100	100	100
Boron Concentration (ppm)	657	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	3783	3759	3810	3783
Steam Generator (WR) Levels (%)	-	67	67	67	67
Steam Generator Pressures (psig)	-	978	978	978	978
CONTAINMENT BUILDING					
Pressure (psig)	.8	-	-	-	-
Temperature (^{o}F)	106	-	-	-	-
Humidity (%)	33	-	-	-	-
Hydrogen Concentration (% vol)	0	-	-	-	-
Recirculation Sump Level (in)	0	-	-	-	-
TANKS					
Accumulator Levels (%)	-	59	59	59	59
Accumulator Pressures (psig)	-	645	645	645	645
Refueling Water Storage Tank Level(%)	99	-	-	-	-

PLANT PARAMETERS

Time 0845 Relative Time (H+00:45)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	90	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	109	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	2125	-	-	-	-
Pressurizer Level (%)	37	-	-	-	-
Reactor Coolant System Temp. (^{O}F)					
T_C (wide-range)	-	557	557	557	557
T_H (wide-range)	-	612	612	612	558
T_{AVG} (narrow-range)	-	585	585	585	558
Core Exit Thermocouple Temp. (^{O}F)	595	-	-	-	-
Reactor Coolant Loop Flows (%)	-	100	100	100	100
Boron Concentration (ppm)	691	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	3316	3292	3344	3318
Steam Generator (WR) Levels (%)	-	66	66	66	66
Steam Generator Pressures (psig)	-	989	989	989	989
CONTAINMENT BUILDING					
Pressure (psig)	1.6	-	-	-	-
Temperature (^{O}F)	115	-	-	-	-
Humidity (%)	57	-	-	-	-
Hydrogen Concentration (% vol)	0	-	-	-	-
Recirculation Sump Level (in)	0	-	-	-	-
TANKS					
Accumulator Levels (%)	-	59	59	59	59
Accumulator Pressures (psig)	-	645	645	645	645
Refueling Water Storage Tank Level(%)	99	-	-	-	-

PLANT PARAMETERS

Time 0847 Relative Time (H+00:47)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	.2	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	99	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	1677	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	557	557	557	557
T_H (wide-range)	-	559	559	559	559
T_{AVG} (narrow-range)	-	558	558	558	558
Core Exit Thermocouple Temp. (^{o}F)	559	-	-	-	-
Reactor Coolant Loop Flows (%)	-	100	100	100	100
Boron Concentration (ppm)	699	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	108	85	137	109
Steam Generator (WR) Levels (%)	-	55	55	55	55
Steam Generator Pressures (psig)	-	1089	1089	1089	1089
CONTAINMENT BUILDING					
Pressure (psig)	2.7	-	-	-	-
Temperature (^{o}F)	117	-	-	-	-
Humidity (%)	98	-	-	-	-
Hydrogen Concentration (% vol)	0	-	-	-	-
Recirculation Sump Level (in)	0	-	-	-	-
TANKS					
Accumulator Levels (%)	-	59	59	59	59
Accumulator Pressures (psig)	-	645	645	645	645
Refueling Water Storage Tank Level(%)	99	-	-	-	-

PLANT PARAMETERS

Time 0900 Relative Time (H+01:00)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	97	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	1141	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	524	524	524	524
T_H (wide-range)	-	565	565	565	565
T_{AVG} (narrow-range)	-	544	544	544	544
Core Exit Thermocouple Temp. (^{o}F)	525	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	816	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate (x10^6 lbm/hr)	-	48	62	0	64
Steam Generator (WR) Levels (%)	-	58	58	57	58
Steam Generator Pressures (psig)	-	1105	1105	1105	1105
CONTAINMENT BUILDING					
Pressure (psig)	5	-	-	-	-
Temperature (^{o}F)	128	-	-	-	-
Humidity (%)	98	-	-	-	-
Hydrogen Concentration (% vol)	0	-	-	-	-
Recirculation Sump Level (in)	0	-	-	-	-
TANKS					
Accumulator Levels (%)	-	59	59	59	59
Accumulator Pressures (psig)	-	645	645	645	645
Refueling Water Storage Tank Level(%)	99	-	-	-	-

PLANT PARAMETERS

Time 0915 Relative Time (H+01:15)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	97	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	1039	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	523	523	523	523
T_H (wide-range)	-	554	554	554	554
T_{AVG} (narrow-range)	-	539	539	539	539
Core Exit Thermocouple Temp. (^{o}F)	500	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	989	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	98	79	119	148
Steam Generator (WR) Levels (%)	-	61	61	61	62
Steam Generator Pressures (psig)	-	992	992	992	992
CONTAINMENT BUILDING					
Pressure (psig)	6	-	-	-	-
Temperature (^{o}F)	136	-	-	-	-
Humidity (%)	98	-	-	-	-
Hydrogen Concentration (% vol)	0	-	-	-	-
Recirculation Sump Level (in)	0	-	-	-	-
TANKS					
Accumulator Levels (%)	-	59	59	59	59
Accumulator Pressures (psig)	-	645	645	645	645
Refueling Water Storage Tank Level(%)	99	-	-	-	-

PLANT PARAMETERS

Time 0930 Relative Time (H+01:30)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	96	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	442	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^oF)					
T_C (wide-range)	-	400	400	400	400
T_H (wide-range)	-	448	448	448	448
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^oF)	493	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1361	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	53	39	88	70
Steam Generator (WR) Levels (%)	-	66	65	65	66
Steam Generator Pressures (psig)	-	445	445	445	445
CONTAINMENT BUILDING					
Pressure (psig)	6	-	-	-	-
Temperature (^oF)	138	-	-	-	-
Humidity (%)	98	-	-	-	-
Hydrogen Concentration (% vol)	0	-	-	-	-
Recirculation Sump Level (in)	0	-	-	-	-
TANKS					
Accumulator Levels (%)	-	39	39	39	39
Accumulator Pressures (psig)	-	432	432	432	432
Refueling Water Storage Tank Level(%)	96	-	-	-	-

PLANT PARAMETERS

Time 0945 Relative Time (H+01:45)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	76	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	345	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	360	360	360	360
T_H (wide-range)	-	418	418	418	418
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	440	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1594	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate (x10^6 lbm/hr)	-	62	78	48	63
Steam Generator (WR) Levels (%)	-	73	70	71	71
Steam Generator Pressures (psig)	-	285	285	285	285
CONTAINMENT BUILDING					
Pressure (psig)	6.9	-	-	-	-
Temperature (^{o}F)	141	-	-	-	-
Humidity (%)	98	-	-	-	-
Hydrogen Concentration (% vol)	0	-	-	-	-
Recirculation Sump Level (in)	0	-	-	-	-
TANKS					
Accumulator Levels (%)	-	23	23	23	23
Accumulator Pressures (psig)	-	336	336	336	336
Refueling Water Storage Tank Level(%)	94	-	-	-	-

PLANT PARAMETERS

Time 1000 Relative Time (H+02:00)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	45	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	271	-	-	-	-
Pressurizer Level (%)	12	-	-	-	-
Reactor Coolant System Temp. (^oF)					
T_C (wide-range)	-	364	364	364	364
T_H (wide-range)	-	410	410	410	410
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^oF)	1250	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1789	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	24	47	31
Steam Generator (WR) Levels (%)	-	75	73	73	74
Steam Generator Pressures (psig)	-	198	198	198	198
CONTAINMENT BUILDING					
Pressure (psig)	15	-	-	-	-
Temperature (^oF)	149	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	0	-	-	-	-
Recirculation Sump Level (in)	0	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	201	201	201	201
Refueling Water Storage Tank Level(%)	93	-	-	-	-

PLANT PARAMETERS

Time 1015 Relative Time (H+02:15)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	45	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	152	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^oF)					
T_C (wide-range)	-	316	314	314	315
T_H (wide-range)	-	363	363	363	363
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^oF)	1250	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1889	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	171	168	184	170
Steam Generator (WR) Levels (%)	-	75	74	75	76
Steam Generator Pressures (psig)	-	106	104	104	105
CONTAINMENT BUILDING					
Pressure (psig)	25	-	-	-	-
Temperature (^oF)	383	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	0	-	-	-	-
Recirculation Sump Level (in)	0	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	125	125	125	125
Refueling Water Storage Tank Level(%)	90	-	-	-	-

PLANT PARAMETERS

Time 1030 Relative Time (H+02:30)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	41	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	50	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^oF)					
T_C (wide-range)	-	257	258	258	256
T_H (wide-range)	-	286	286	286	286
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^oF)	1300	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1968	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	74	54	62	74
Steam Generator (WR) Levels (%)	-	74	71	71	77
Steam Generator Pressures (psig)	-	28	29	29	28
CONTAINMENT BUILDING					
Pressure (psig)	25	-	-	-	-
Temperature (^oF)	383	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	0.5	-	-	-	-
Recirculation Sump Level (in)	0	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	86	-	-	-	-

PLANT PARAMETERS

Time 1045 Relative Time (H+02:45)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	43	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	44	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^oF)					
T_C (wide-range)	-	250	255	255	249
T_H (wide-range)	-	278	278	278	278
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^oF)	1500	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1974	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	69	41	36	69
Steam Generator (WR) Levels (%)	-	74	71	71	77
Steam Generator Pressures (psig)	-	25	26	26	25
CONTAINMENT BUILDING					
Pressure (psig)	25	-	-	-	-
Temperature (^oF)	383	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	0.5	-	-	-	-
Recirculation Sump Level (in)	100	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	85	-	-	-	-

PLANT PARAMETERS

Time 1100 Relative Time (H+03:00)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	43	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	44	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	250	255	255	249
T_H (wide-range)	-	278	278	278	278
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	1600	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1974	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	69	41	36	69
Steam Generator (WR) Levels (%)	-	74	71	71	77
Steam Generator Pressures (psig)	-	25	26	26	25
CONTAINMENT BUILDING					
Pressure (psig)	25	-	-	-	-
Temperature (^{o}F)	383	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	1.5	-	-	-	-
Recirculation Sump Level (in)	100	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	85	-	-	-	-

PLANT PARAMETERS

Time 1115 / Relative Time (H+03:15)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	43	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	44	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	250	255	255	249
T_H (wide-range)	-	278	278	278	278
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	2000	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1974	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	69	41	36	69
Steam Generator (WR) Levels (%)	-	74	71	71	77
Steam Generator Pressures (psig)	-	25	26	26	25
CONTAINMENT BUILDING					
Pressure (psig)	25	-	-	-	-
Temperature (^{o}F)	383	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	2.5	-	-	-	-
Recirculation Sump Level (in)	100	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	85	-	-	-	-

PLANT PARAMETERS

Time 1130 Relative Time (H+03:30)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	43	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	44	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	250	255	255	249
T_H (wide-range)	-	278	278	278	278
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	2500	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1974	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	69	41	36	69
Steam Generator (WR) Levels (%)	-	74	71	71	77
Steam Generator Pressures (psig)	-	25	26	26	25
CONTAINMENT BUILDING					
Pressure (psig)	25	-	-	-	-
Temperature (^{o}F)	383	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	3.75	-	-	-	-
Recirculation Sump Level (in)	100	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	85	-	-	-	-

PLANT PARAMETERS

Time 1145 Relative Time . (H+03:45)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	43	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	44	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	250	255	255	249
T_H (wide-range)	-	278	278	278	278
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	2700	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1964	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate (x10^6 lbm/hr)	-	69	41	36	69
Steam Generator (WR) Levels (%)	-	74	71	71	77
Steam Generator Pressures (psig)	-	25	26	26	25
CONTAINMENT BUILDING					
Pressure (psig)	25	-	-	-	-
Temperature (^{o}F)	383	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	4.8	-	-	-	-
Recirculation Sump Level (in)	100	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	85	-	-	-	-

PLANT PARAMETERS

Time 1200 Relative Time (H+04:00)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	45	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	36	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	234	242	242	232
T_H (wide-range)	-	266	266	266	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	2900	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	59	28	23	59
Steam Generator (WR) Levels (%)	-	74	70	70	77
Steam Generator Pressures (psig)	-	20	25	25	20
CONTAINMENT BUILDING					
Pressure (psig) Real time 1200	47	-	-	-	-
Real time 1201	8				
Temperature (^{o}F)	383	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	6.5	-	-	-	-
Recirculation Sump Level (in)	110	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	85	-	-	-	-

PLANT PARAMETERS

Time 1215 Relative Time (H+04:15)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	47	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^oF)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^oF)	2900	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	7	-	-	-	-
Temperature (^oF)	350	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	112	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	85	-	-	-	-

PLANT PARAMETERS

Time 1230 Relative Time (H+04:30)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	49	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	0	-	-	-	-
Reactor Coolant System Temp. (^oF)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^oF)	2700	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	7	-	-	-	-
Temperature (^oF)	310	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	112	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	85	-	-	-	-

PLANT PARAMETERS

Time 1245 Relative Time (H+04:45)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	51	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	6	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	2300	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	5	-	-	-	-
Temperature (^{o}F)	270	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	112	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	85	-	-	-	-

PLANT PARAMETERS

Time 1300 Relative Time (H+05:00)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	53	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	20	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	2000	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	5	-	-	-	-
Temperature (^{o}F)	230	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	114	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	85	-	-	-	-

PLANT PARAMETERS

Time 1315 Relative Time (H+05:15)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	55	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	35	-	-	-	-
Reactor Coolant System Temp. (^oF)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^oF)	1600	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate (x10^6 lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	4	-	-	-	-
Temperature (^oF)	190	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	116	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	85	-	-	-	-

PLANT PARAMETERS

Time 1330 Relative Time (H+05:30)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	85	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	45	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	1400	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	2	-	-	-	-
Temperature (^{o}F)	150	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	116	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	85	-	-	-	-

PLANT PARAMETERS

Time 1345 Relative Time (H+05:45)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	100	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	100	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	1000	-	-	-	-
Reactor Coolant Loop Flows (%)	-	0	0	0	0
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	0	-	-	-	-
Temperature (^{o}F)	110	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	116	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	25	-	-	-	-

PLANT PARAMETERS

Time 1400 Relative Time (H+06:00)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	100	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	100	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	900	-	-	-	-
Reactor Coolant Loop Flows (%)	-	10	10	10	10
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	0	-	-	-	-
Temperature (^{o}F)	100	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	116	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	25	-	-	-	-

PLANT PARAMETERS

Time 1415 Relative Time (H+06:15)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	100	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	100	-	-	-	-
Reactor Coolant System Temp. (^{O}F)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{O}F)	900	-	-	-	-
Reactor Coolant Loop Flows (%)	-	10	10	10	10
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	0	-	-	-	-
Temperature (^{O}F)	100	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	116	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	25	-	-	-	-

PLANT PARAMETERS

Time 1430 Relative Time (H+06:30)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	100	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	100	-	-	-	-
Reactor Coolant System Temp. (^{O}F)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{O}F)	900	-	-	-	-
Reactor Coolant Loop Flows (%)	-	10	10	10	10
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	0	-	-	-	-
Temperature (^{O}F)	100	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	116	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	25	-	-	-	-

PLANT PARAMETERS

Time 1445 Relative Time (H+06:45)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	100	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	100	-	-	-	-
Reactor Coolant System Temp. (^{O}F)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{O}F)	900	-	-	-	-
Reactor Coolant Loop Flows (%)	-	10	10	10	10
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	0	-	-	-	-
Temperature (^{O}F)	100	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	116	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	25	-	-	-	-

PLANT PARAMETERS

Time 1500 Relative Time (H+07:00)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	100	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	100	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	900	-	-	-	-
Reactor Coolant Loop Flows (%)	-	10	10	10	10
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	0	-	-	-	-
Temperature (^{o}F)	100	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	116	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	25	-	-	-	-

PLANT PARAMETERS

Time 1515 Relative Time (H+07:15)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	100	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	100	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	900	-	-	-	-
Reactor Coolant Loop Flows (%)	-	10	10	10	10
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	0	-	-	-	-
Temperature (^{o}F)	100	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	116	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	25	-	-	-	-

PLANT PARAMETERS

Time 1530 Relative Time (H+07:30)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	100	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	100	-	-	-	-
Reactor Coolant System Temp. (^oF)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^oF)	600	-	-	-	-
Reactor Coolant Loop Flows (%)	-	10	10	10	10
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	0	-	-	-	-
Temperature (^oF)	100	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	116	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	25	-	-	-	-

PLANT PARAMETERS

Time 1545 Relative Time (H+07:45)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	100	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	100	-	-	-	-
Reactor Coolant System Temp. (^{o}F)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{o}F)	600	-	-	-	-
Reactor Coolant Loop Flows (%)	-	10	10	10	10
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	0	-	-	-	-
Temperature (^{o}F)	100	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	116	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	25	-	-	-	-

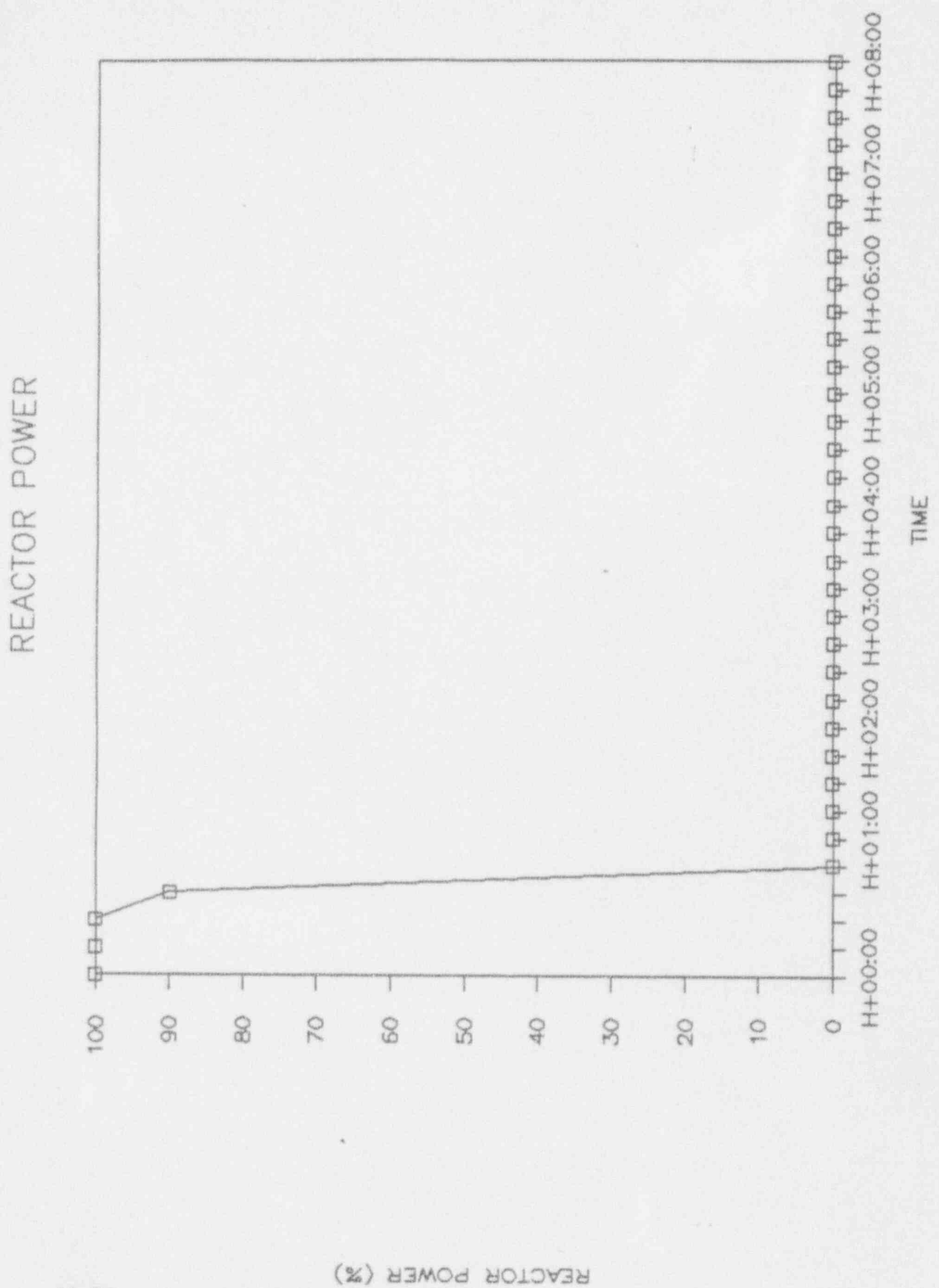
PLANT PARAMETERS

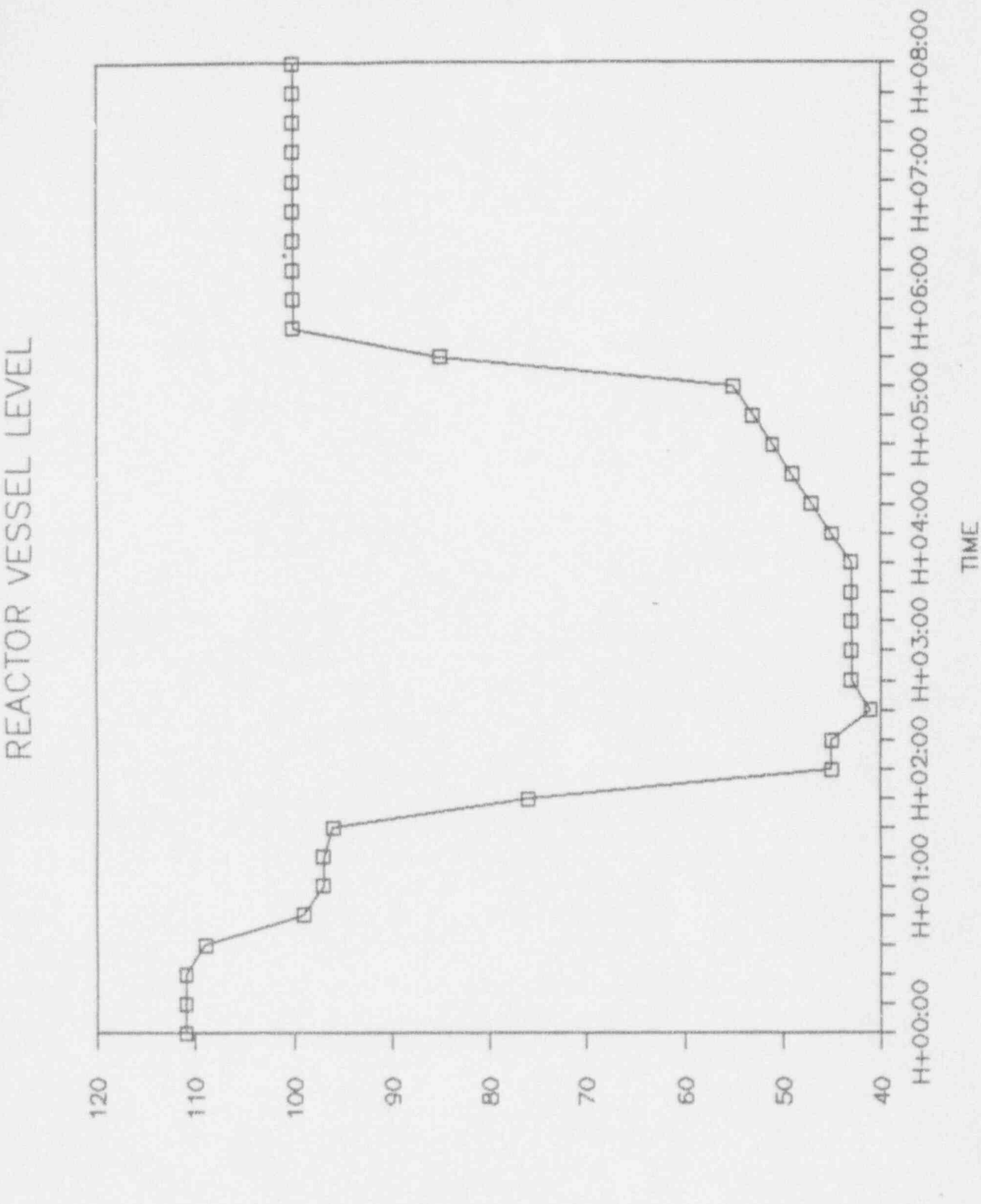
Time 1600 Relative Time (H+08:00)	Plant Values	Loop 1 or A Values	Loop 2 or B Values	Loop 3 or C Values	Loop 4 or D Values
PRIMARY SYSTEM					
Reactor Power (%)	0	-	-	-	-
Reactor Vessel Level (%) - Natural Circulation	100	-	-	-	-
Reactor Coolant System (RCS) Pressure (psig)	30	-	-	-	-
Pressurizer Level (%)	100	-	-	-	-
Reactor Coolant System Temp. (^{O}F)					
T_C (wide-range)	-	230	230	230	232
T_H (wide-range)	-	260	260	260	266
T_{AVG} (narrow-range)	-	530	530	530	530
Core Exit Thermocouple Temp. (^{O}F)	600	-	-	-	-
Reactor Coolant Loop Flows (%)	-	10	10	10	10
Boron Concentration (ppm)	1982	-	-	-	-
SECONDARY SYSTEMS					
Main Steam Flow Rate ($x10^6$ lbm/hr)	-	55	30	20	60
Steam Generator (WR) Levels (%)	-	70	70	70	70
Steam Generator Pressures (psig)	-	20	20	20	20
CONTAINMENT BUILDING					
Pressure (psig)	0	-	-	-	-
Temperature (^{O}F)	100	-	-	-	-
Humidity (%)	100	-	-	-	-
Hydrogen Concentration (% vol)	.5	-	-	-	-
Recirculation Sump Level (in)	116	-	-	-	-
TANKS					
Accumulator Levels (%)	-	0	0	0	0
Accumulator Pressures (psig)	-	26	26	26	26
Refueling Water Storage Tank Level(%)	25	-	-	-	-

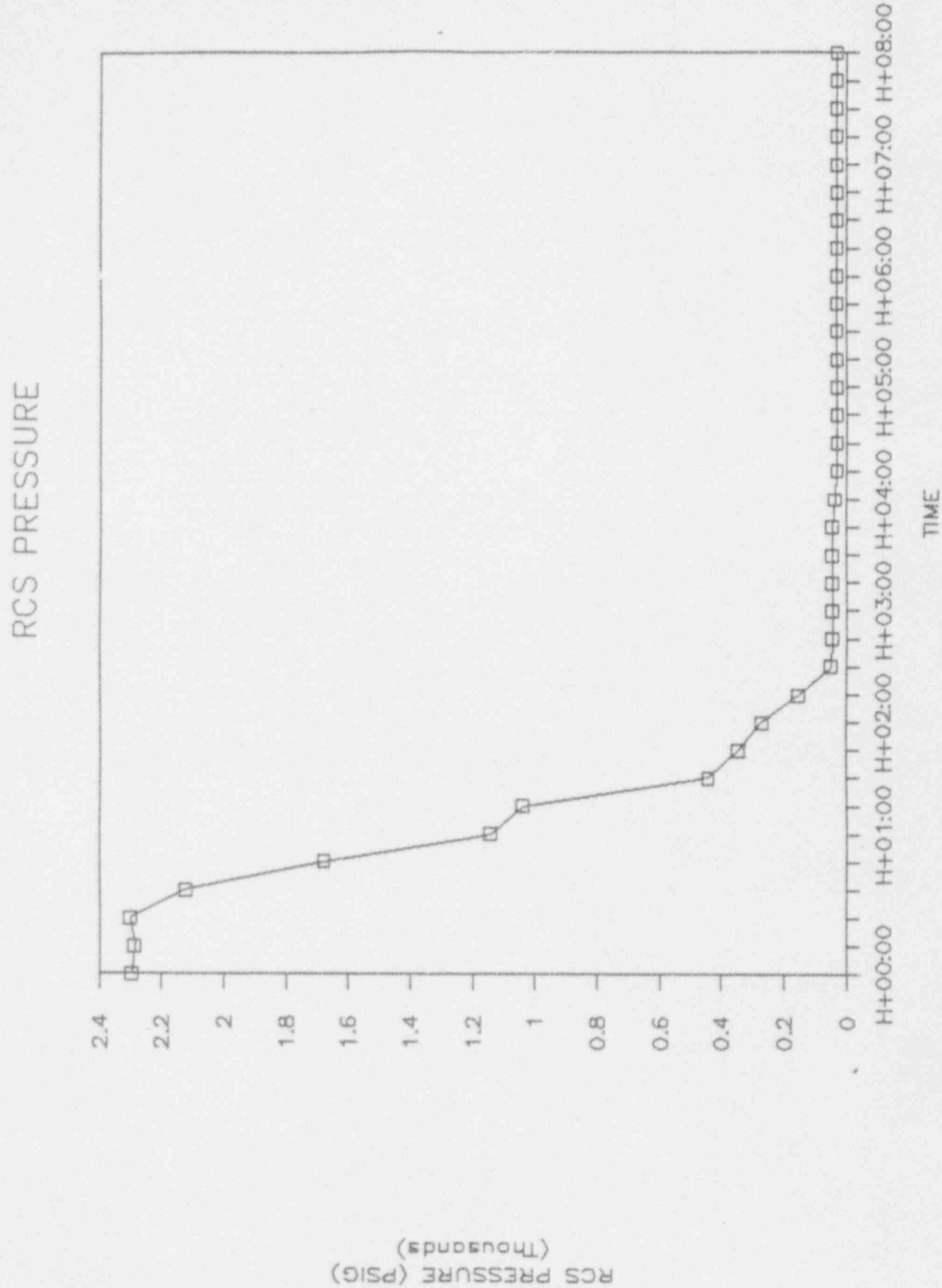
PLANT GRAPHS

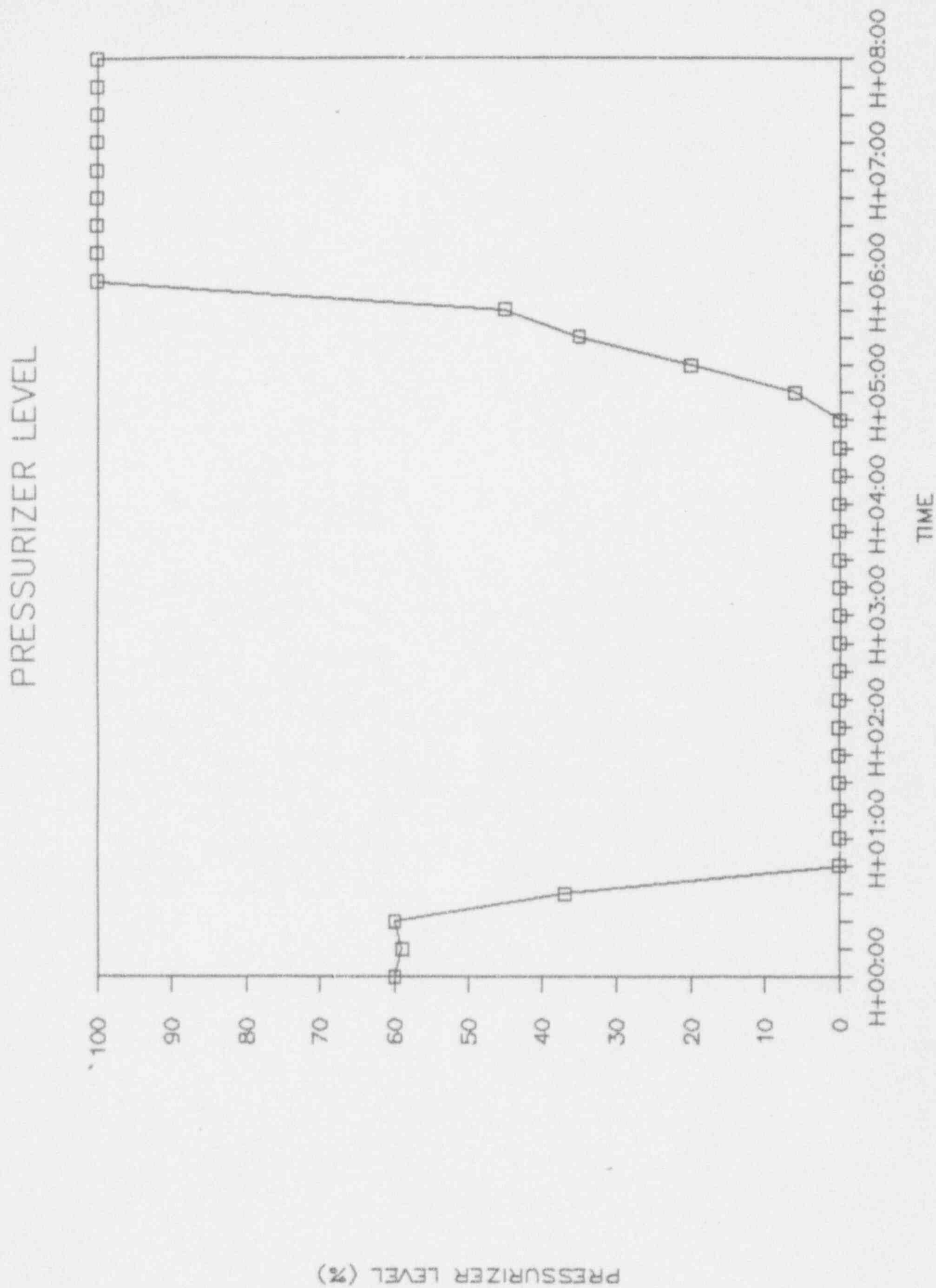
Time-related plant parameters are provided in the following subsection. The data is depicted in graphic form versus time. The previous subsection, "Plant Parameters", depicts the same data in tabular form.

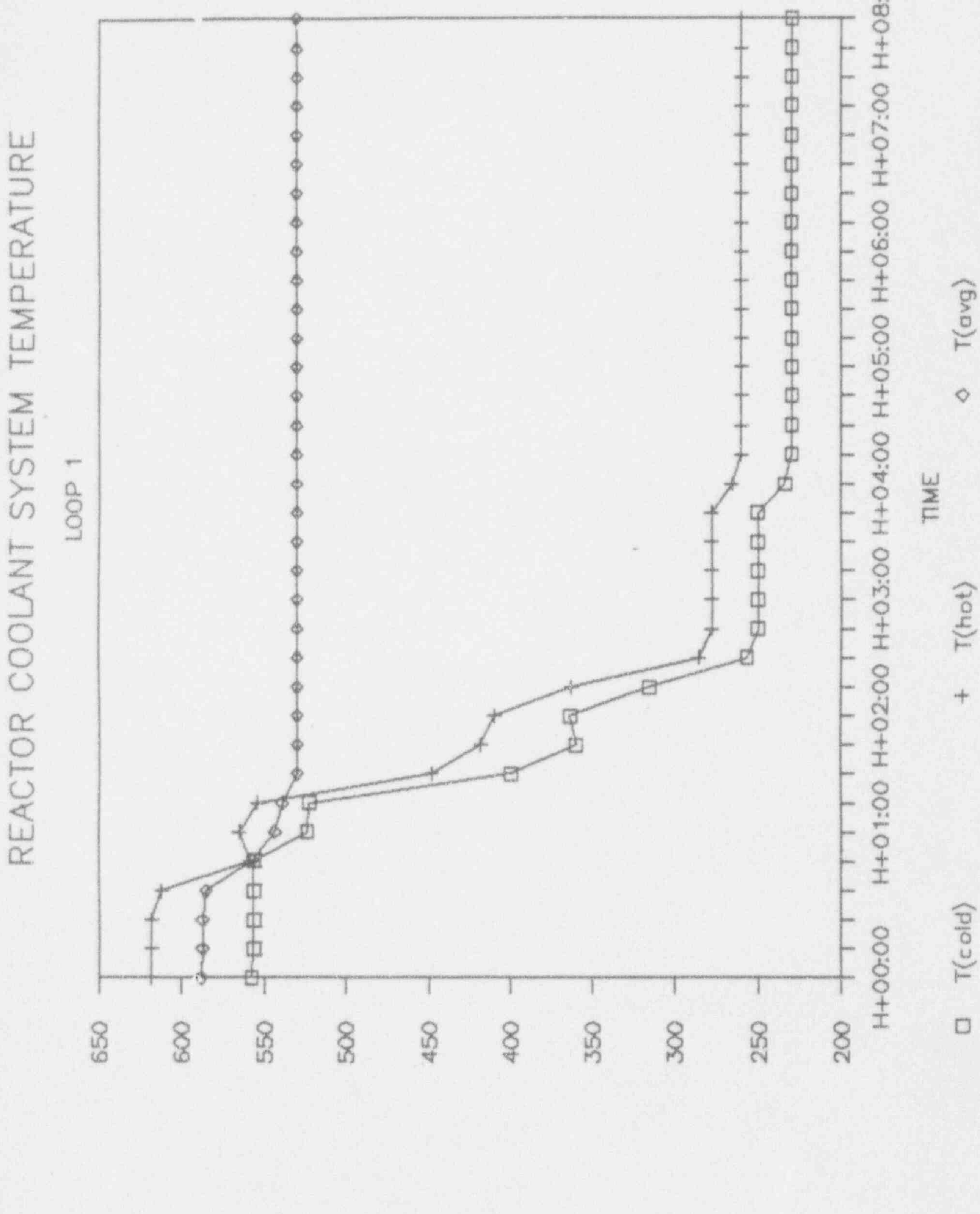
These graphs may be used as a source of data for Control Room (CR) operators, by the CR Lead Controller, in case of simulator failure.

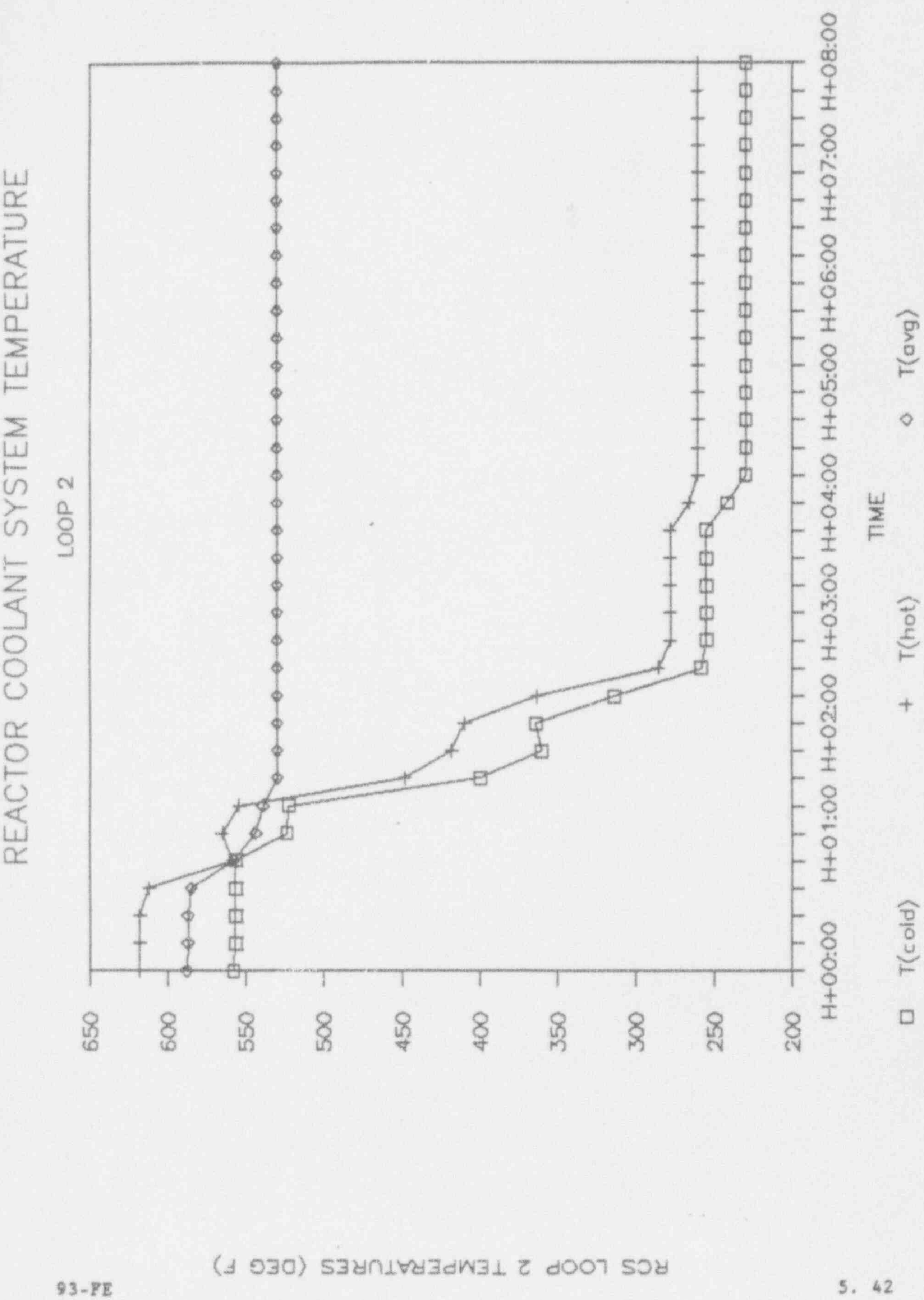

REACTOR POWER
REACTOR POWER (%)
100
90
80
70
60
50
40
30
20
10
0
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME

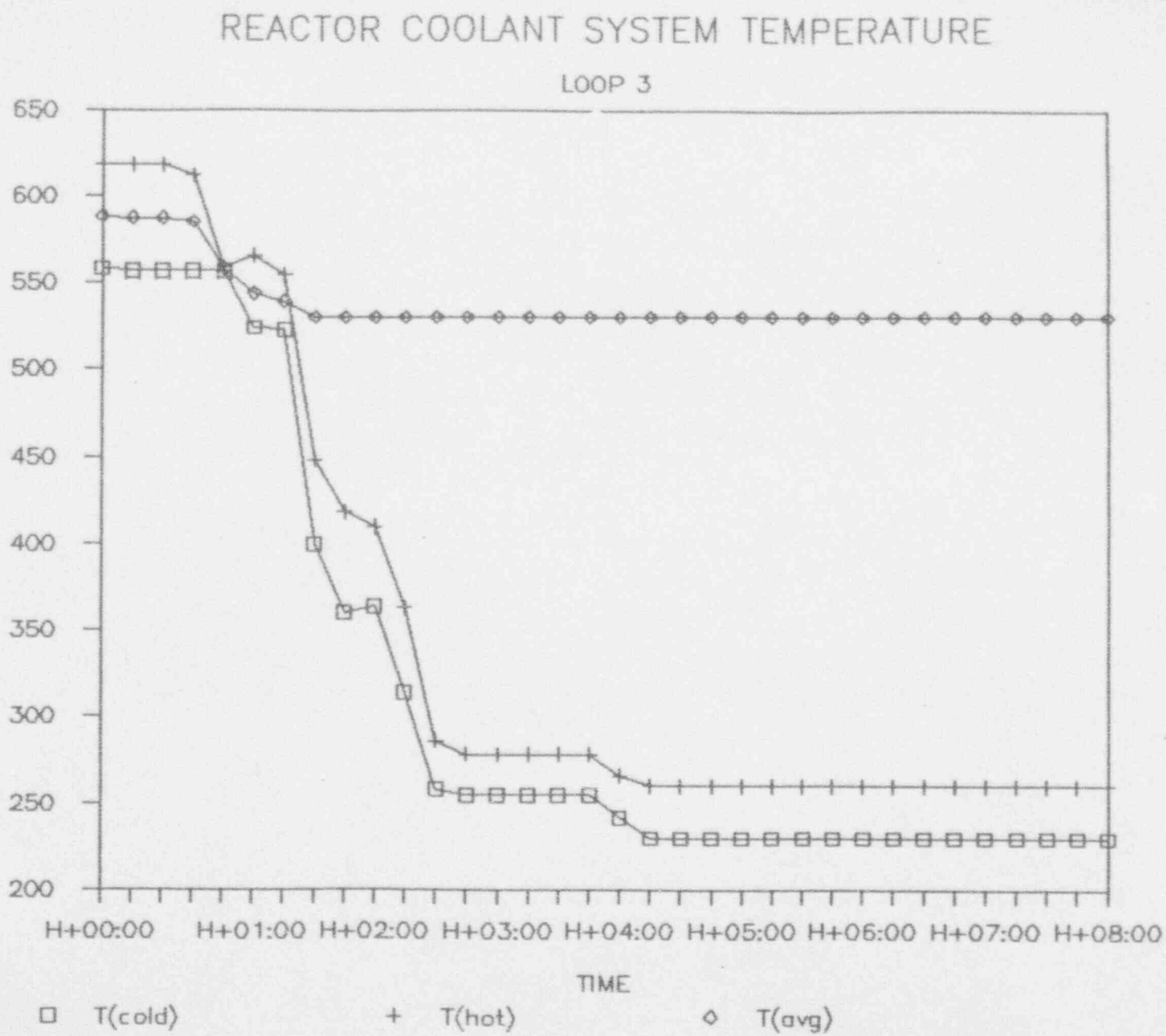

REACTOR VESSEL LEVEL
REACTOR VESSEL LEVEL (%)
120
110
100
90
80
70
60
50
40
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME

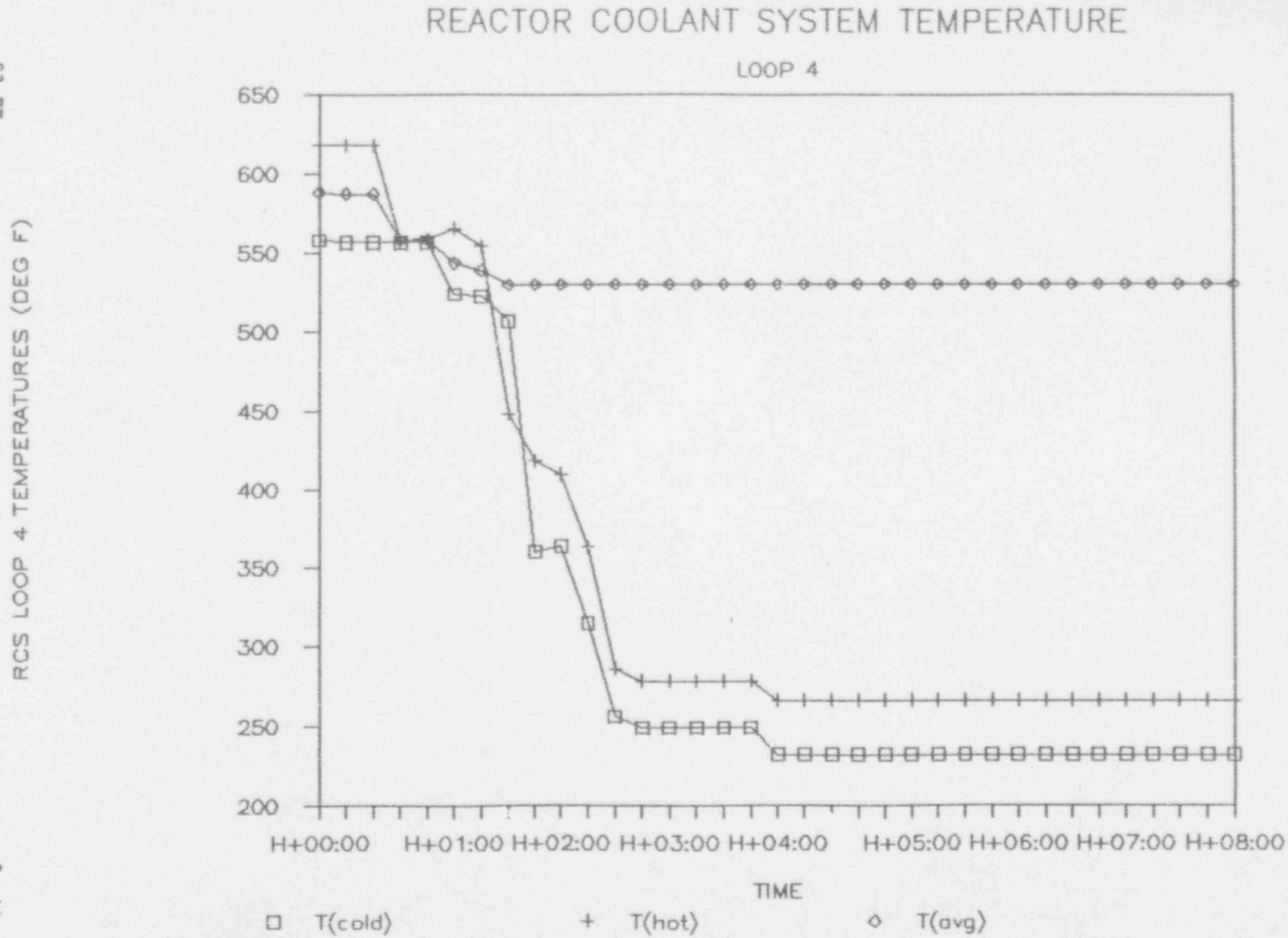

RCS PRESSURE
RCS PRESSURE (PSIG)
(Thousands)
2.4
2.2
2
1.8
1.6
1.4
1.2
1
0.8
0.6
0.4
0.2
0
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME

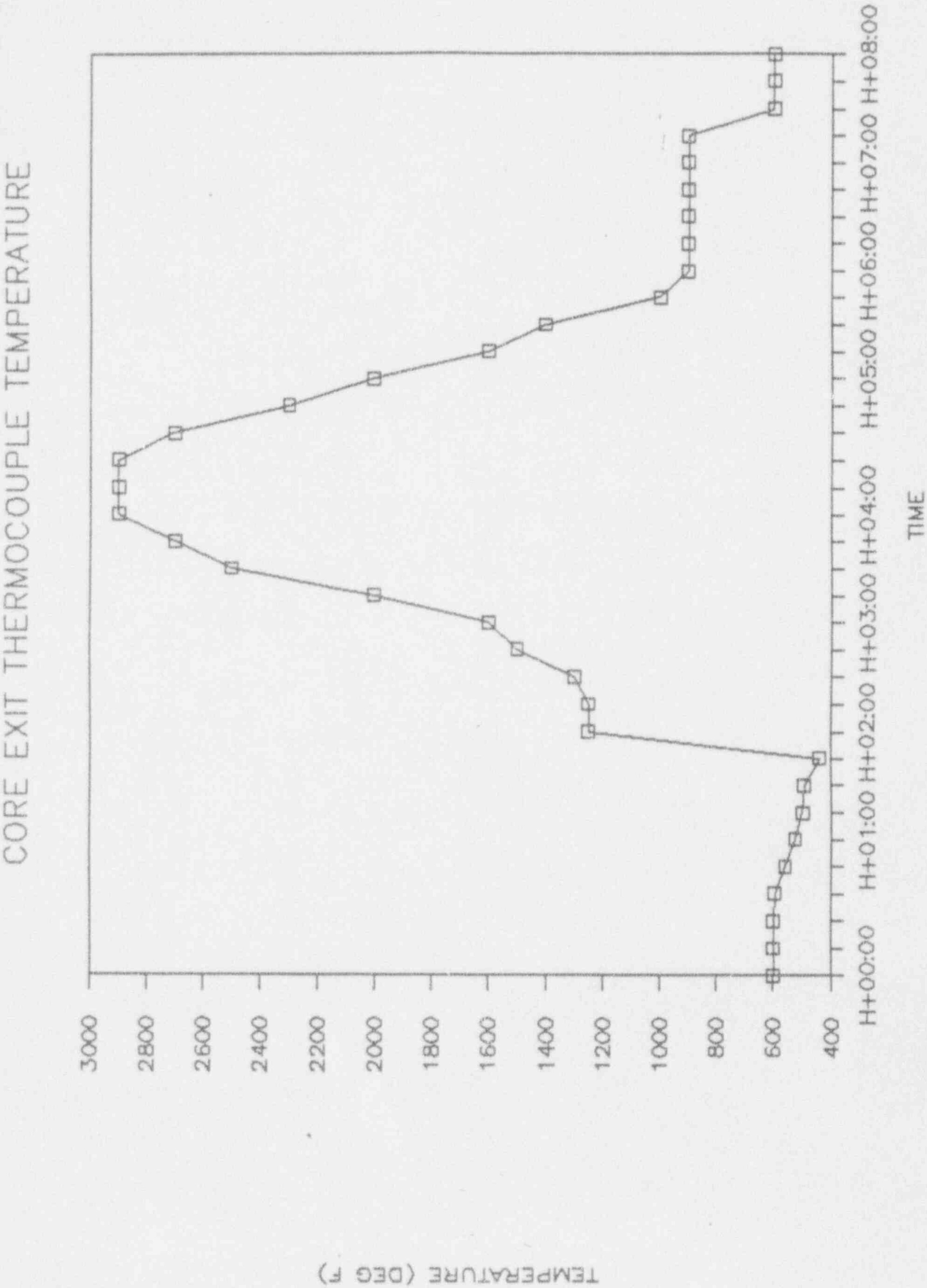

PRESSURIZER LEVEL
PRESSURIZER LEVEL (%)
100
90
80
70
60
50
40
30
20
10
0
H+00:00 H+01:00 H+02:00 H+03:00 H+04:00 H+05:00 H+06:00 H+07:00 H+08:00
TIME

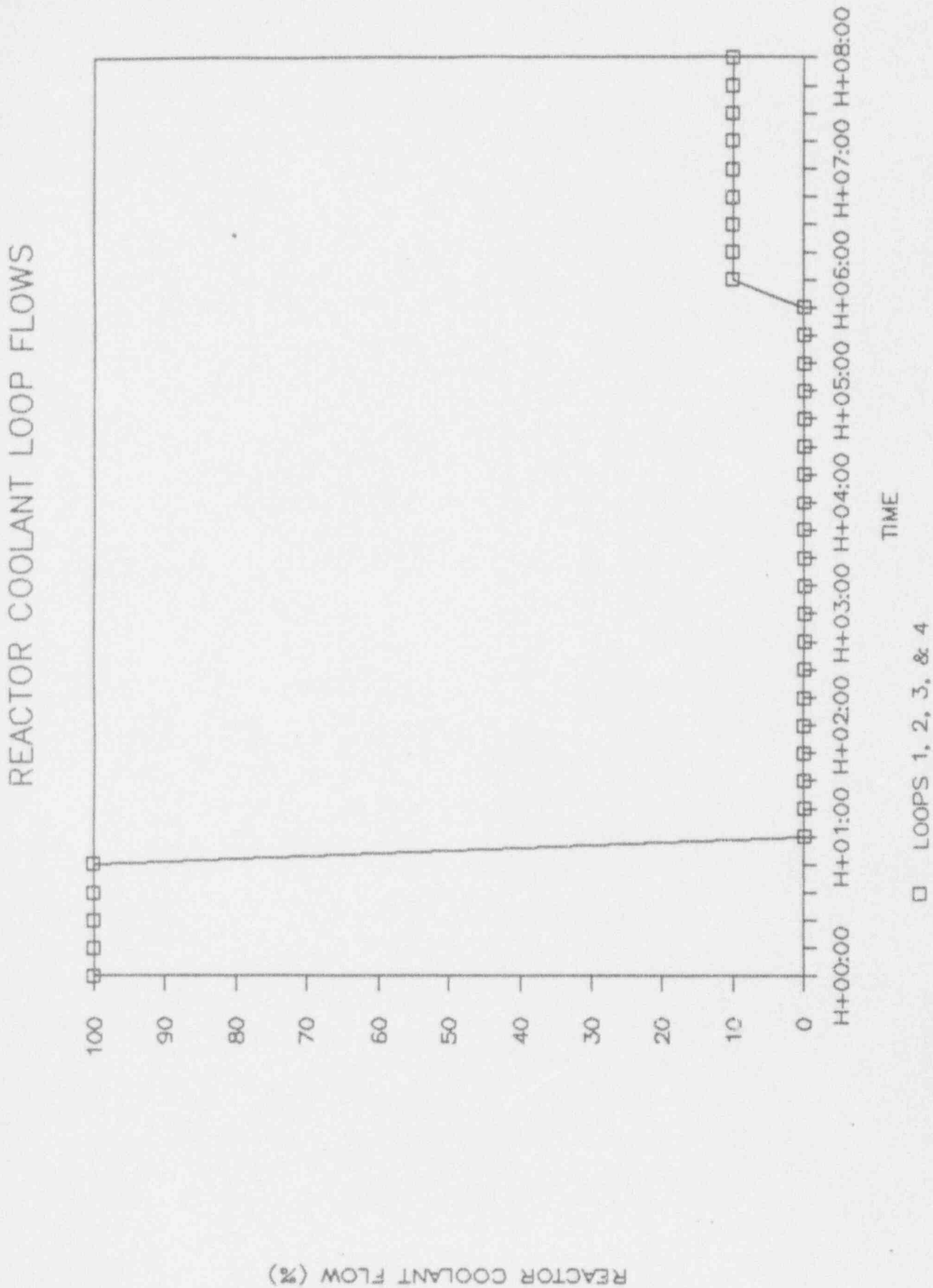

REACTOR COOLANT SYSTEM TEMPERATURE
LOOP 1
RCS LOOP 1 TEMPERATURES (DEG F)
650
600
550
500
450
400
350
300
250
200
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME
□ T(cold)
+ T(hot)
◇ T(avg)

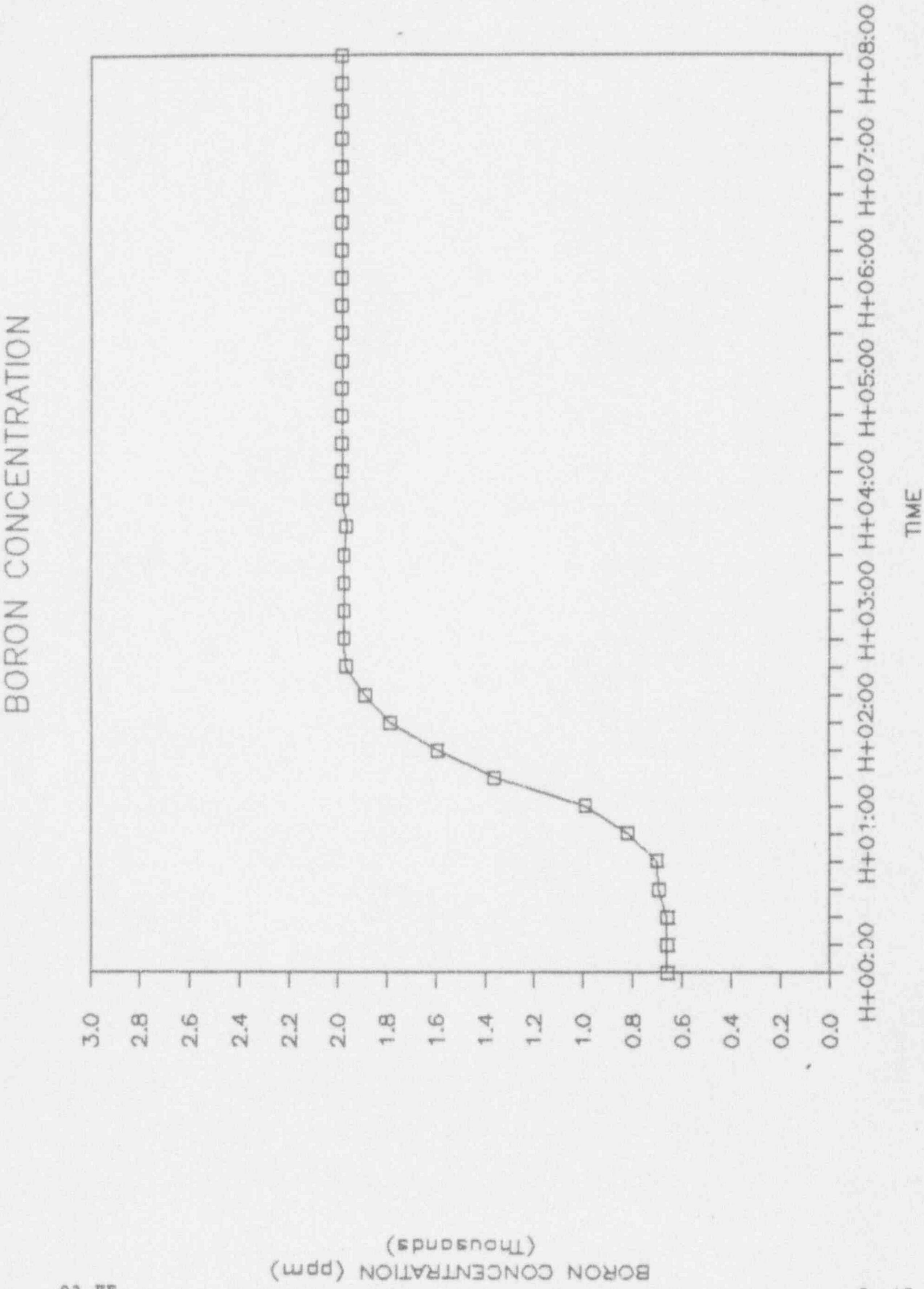

REACTOR COOLANT SYSTEM TEMPERATURE
LOOP 2
650
600
550
500
450
400
350
300
250
200
RCS LOOP 2 TEMPERATURES (DEG F)
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME
□ T(cold)
+ T(hot)
◇ T(avg)


REACTOR COOLANT SYSTEM TEMPERATURE
LOOP 3
RCS LOOP 3 TEMPERATURES (DEG F)
650
600
550
500
450
400
350
300
250
200
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME
T(cold)
T(hot)
T(avg)


REACTOR COOLANT SYSTEM TEMPERATURE
LOOP 4
RCS LOOP 4 TEMPERATURES (DEG F)
650
600
550
500
450
400
350
300
250
200
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME
T(cold)
T(hot)
T(avg)

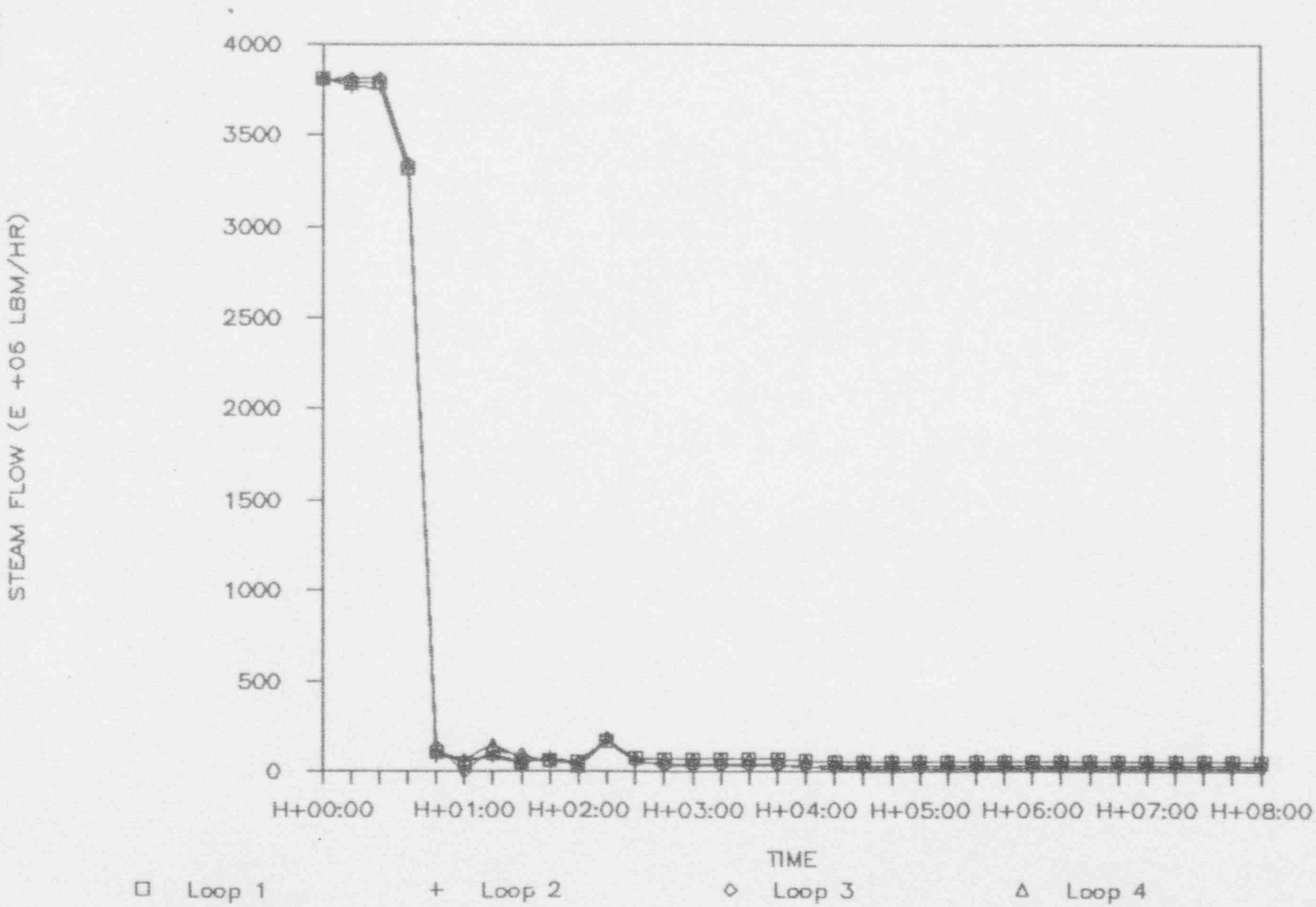

CORE EXIT THERMOCOUPLE TEMPERATURE
TEMPERATURE (DEG F)
3000
2800
2600
2400
2200
2000
1800
1600
1400
1200
1000
800
600
400
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME

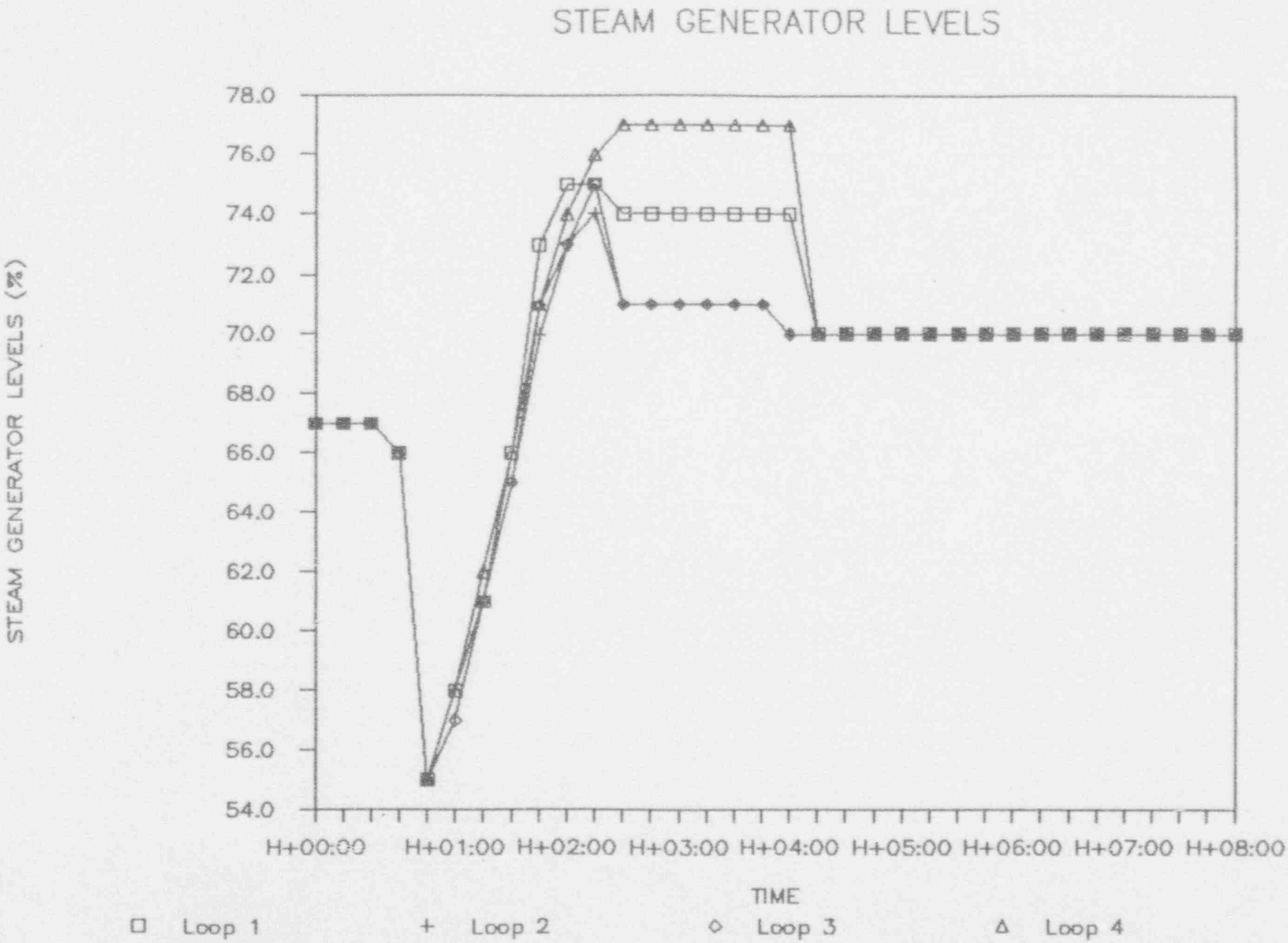

REACTOR COOLANT LOOP FLOWS
REACTOR COOLANT FLOW (%)
100
90
80
70
60
50
40
30
20
10
0
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME
LOOPS 1, 2, 3, & 4

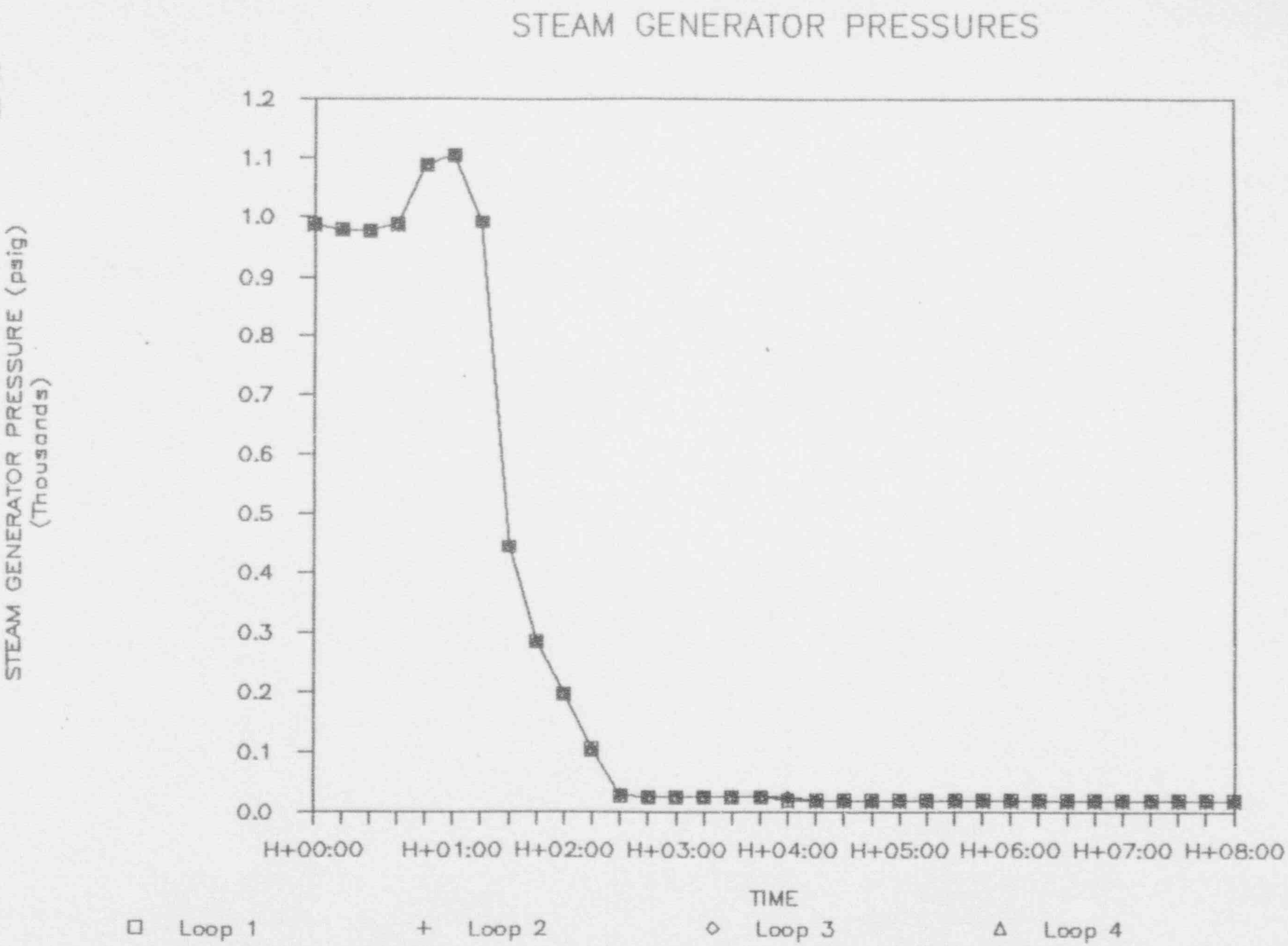


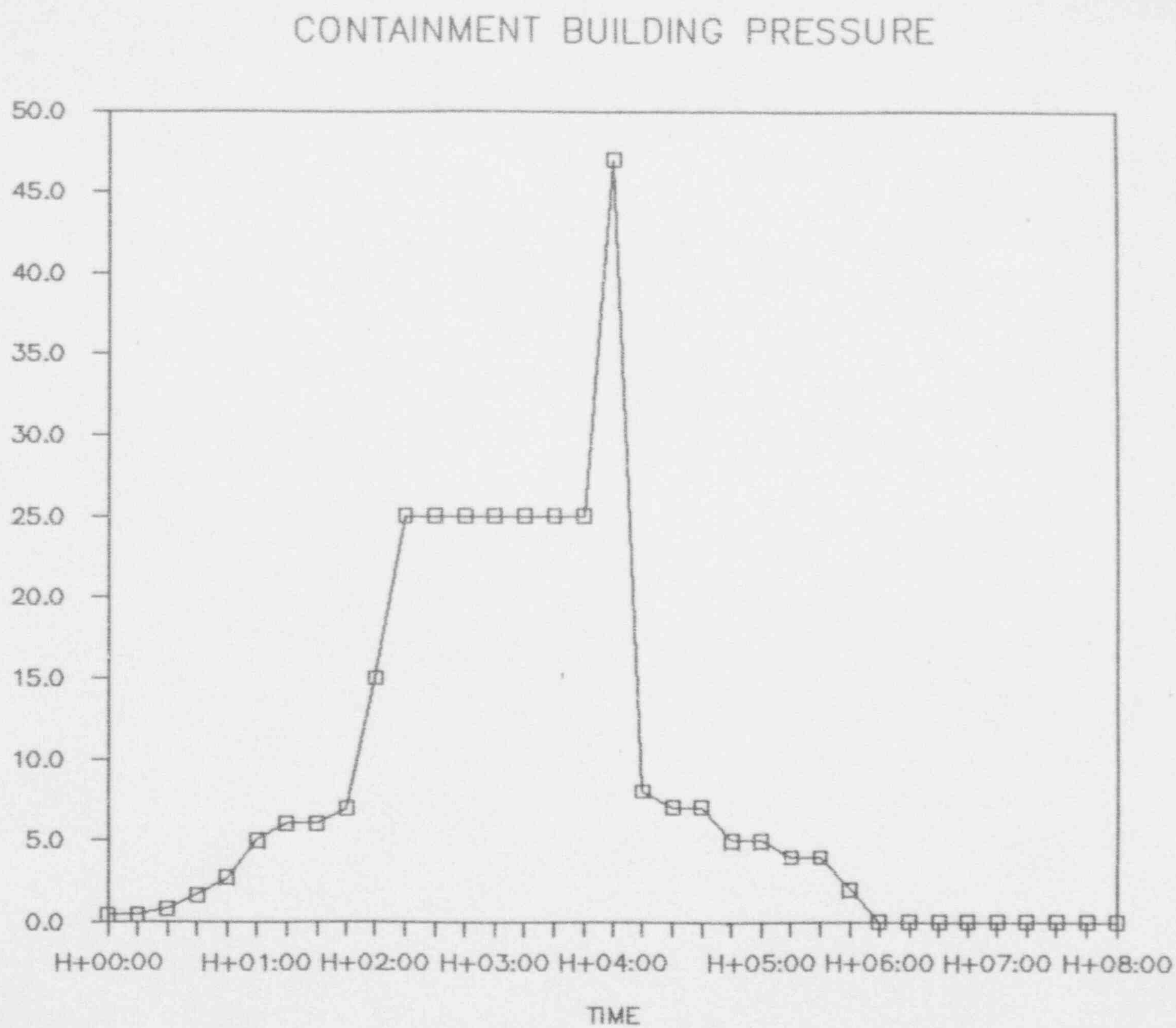
BORON CONCENTRATION
BORON CONCENTRATION (ppm)
(Thousands)
3.0
2.8
2.6
2.4
2.2
2.0
1.8
1.6
1.4
1.2
1.0
0.8
0.6
0.4
0.2
0.0
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME

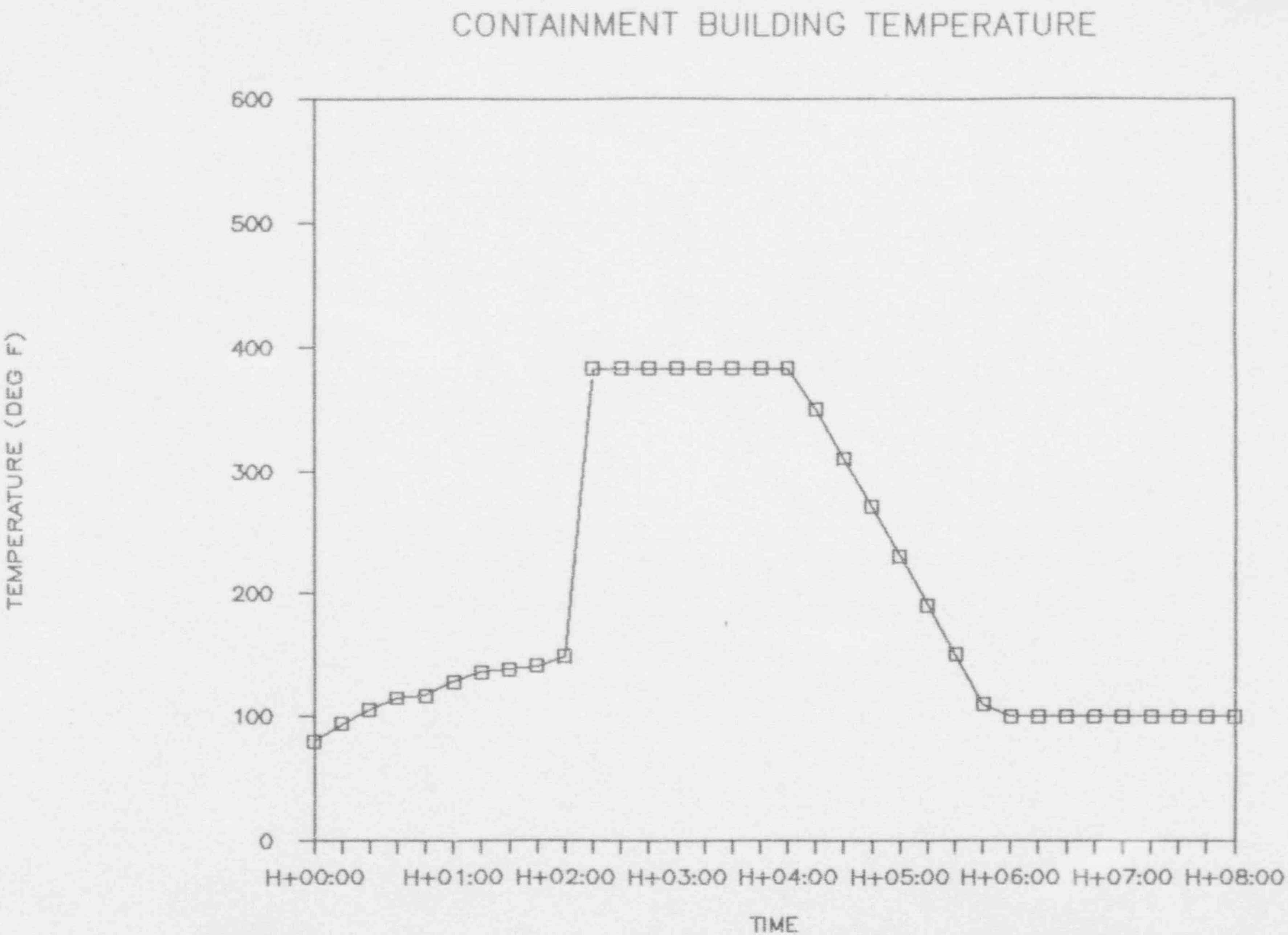
MAIN STEAM FLOW RATE

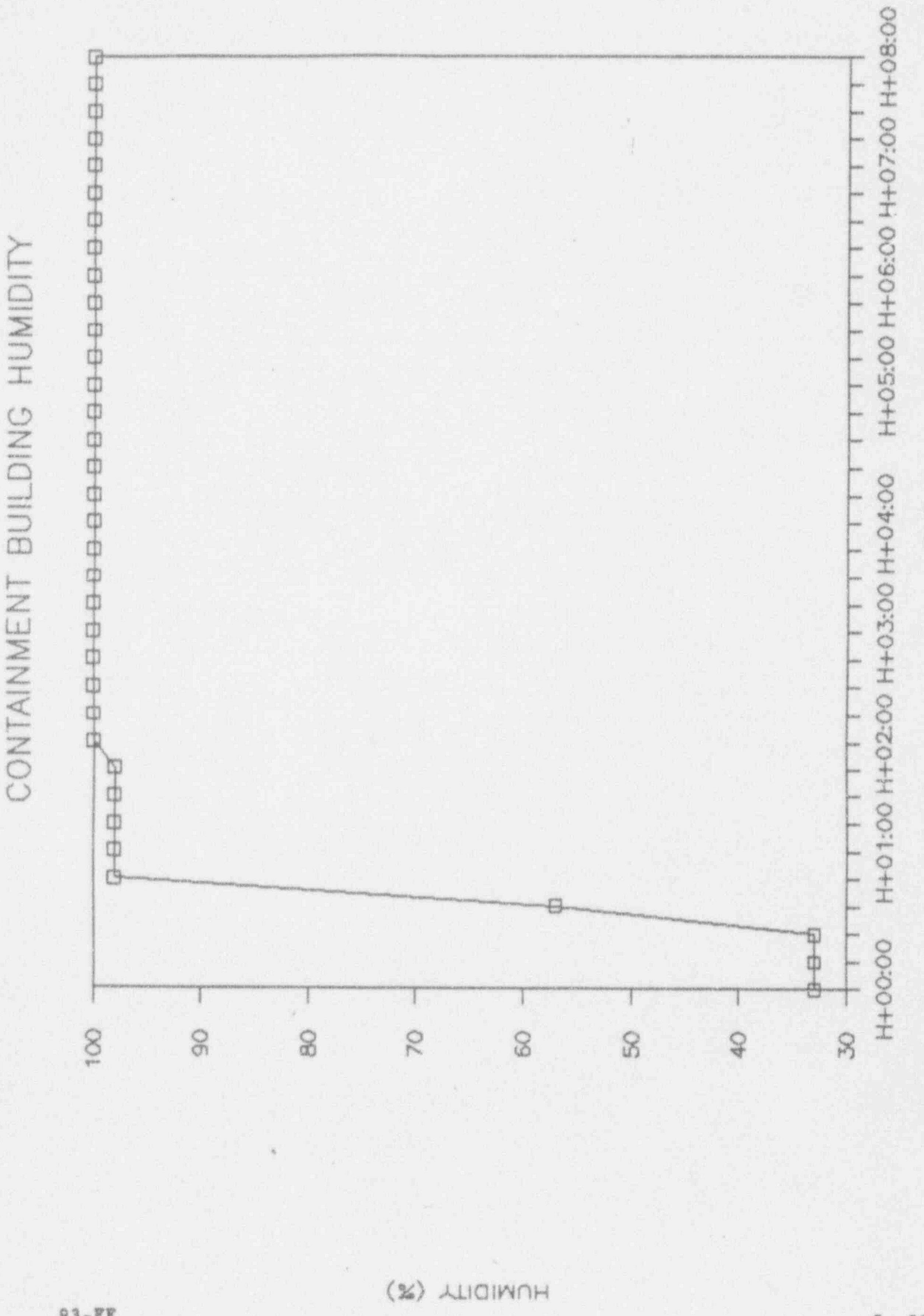



STEAM GENERATOR LEVELS
STEAM GENERATOR LEVELS (%)
78.0
76.0
74.0
72.0
70.0
68.0
66.0
64.0
62.0
60.0
58.0
56.0
54.0
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME
Loop 1
Loop 2
Loop 3
Loop 4


STEAM GENERATOR PRESSURES
STEAM GENERATOR PRESSURE (psig)
(Thousands)
1.2
1.1
1.0
0.9
0.8
0.7
0.6
0.5
0.4
0.3
0.2
0.1
0.0
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME
□ Loop 1
+ Loop 2
◇ Loop 3
Δ Loop 4

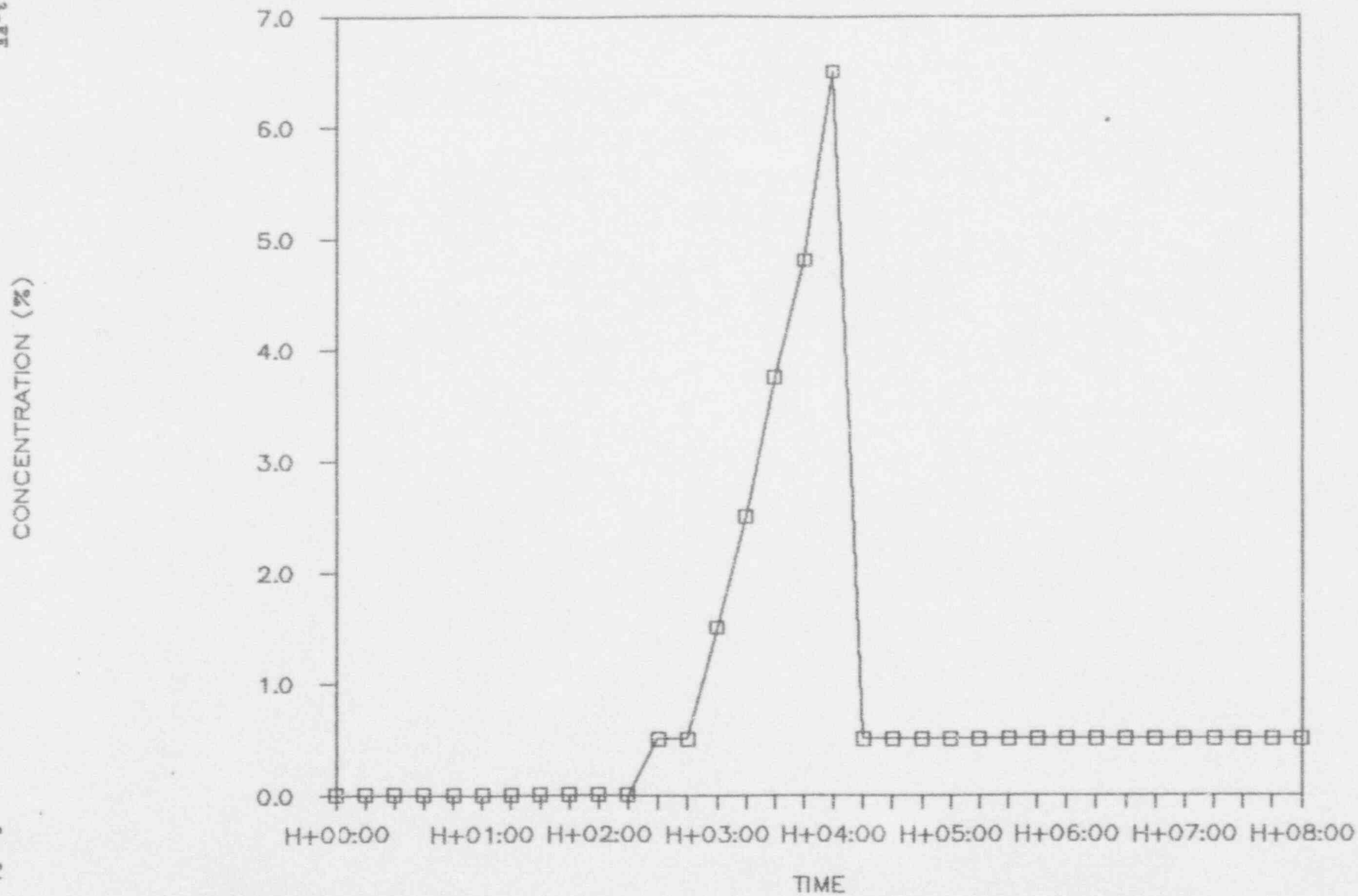

CONTAINMENT BUILDING PRESSURE
PRESSURE (psig)
50.0
45.0
40.0
35.0
30.0
25.0
20.0
15.0
10.0
5.0
0.0
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME

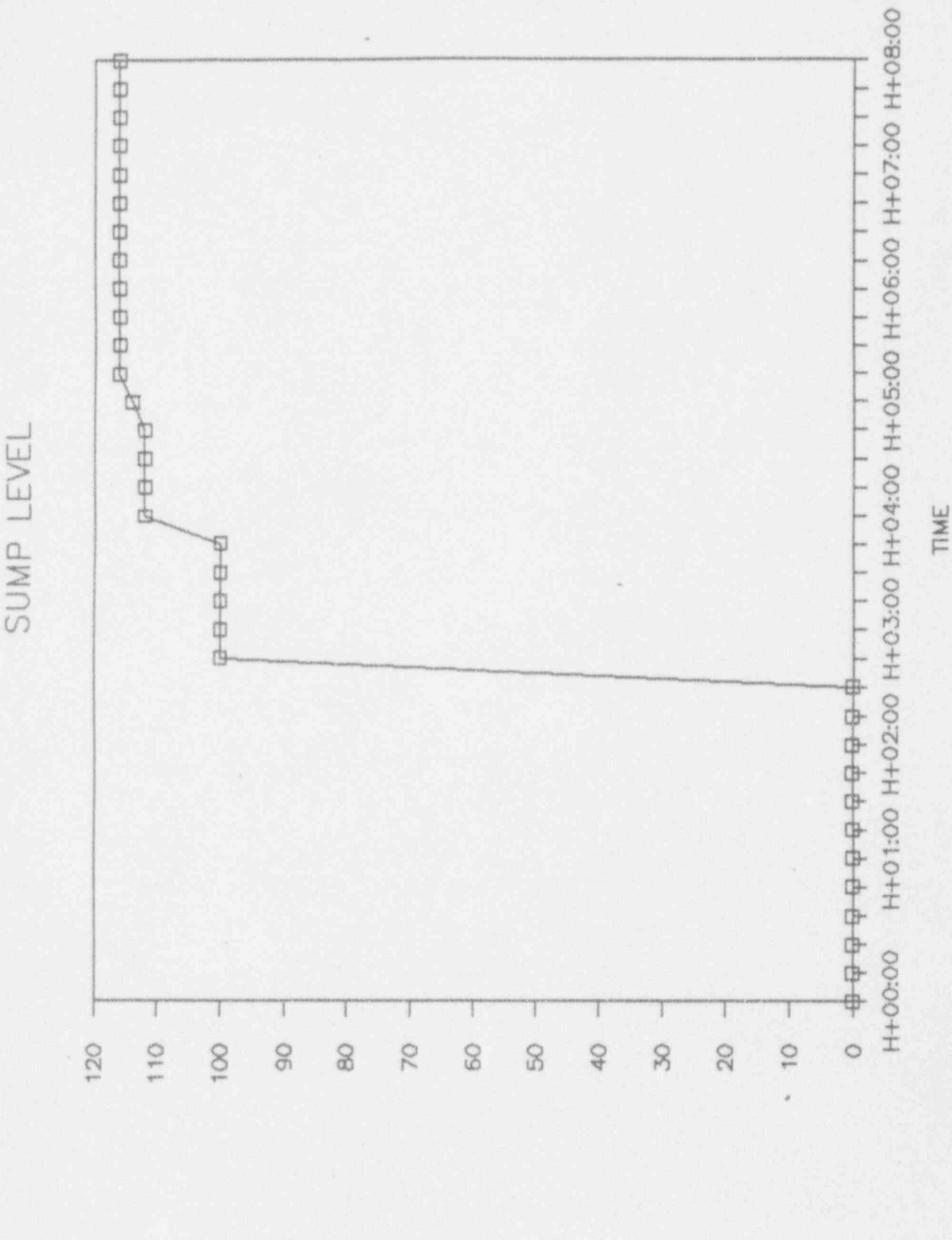

CONTAINMENT BUILDING TEMPERATURE
TEMPERATURE (DEG F)
600
500
400
300
200
100
0
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME

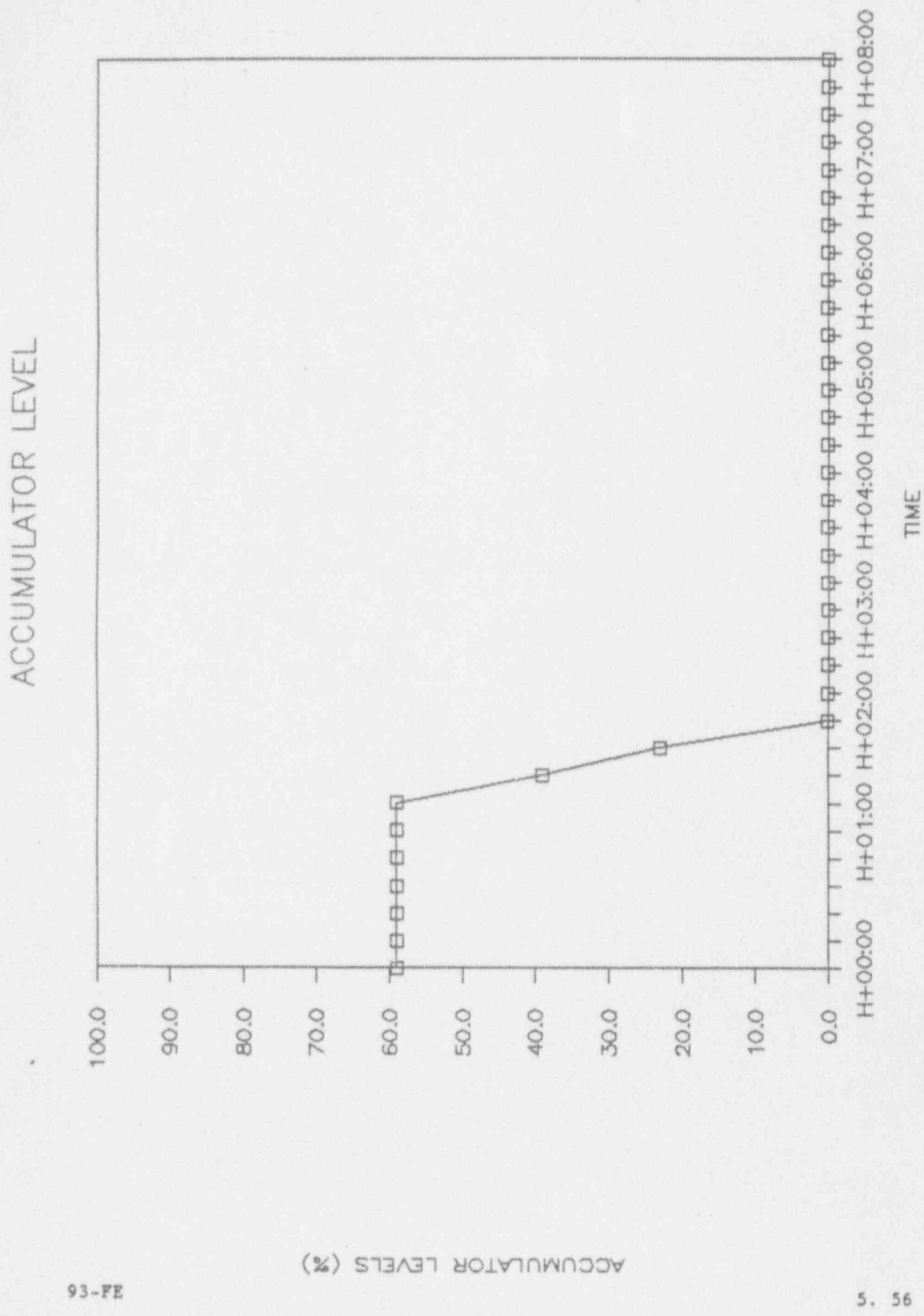

CONTAINMENT BUILDING HUMIDITY
HUMIDITY (%)
100
90
80
70
60
50
40
30
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME

93-FE

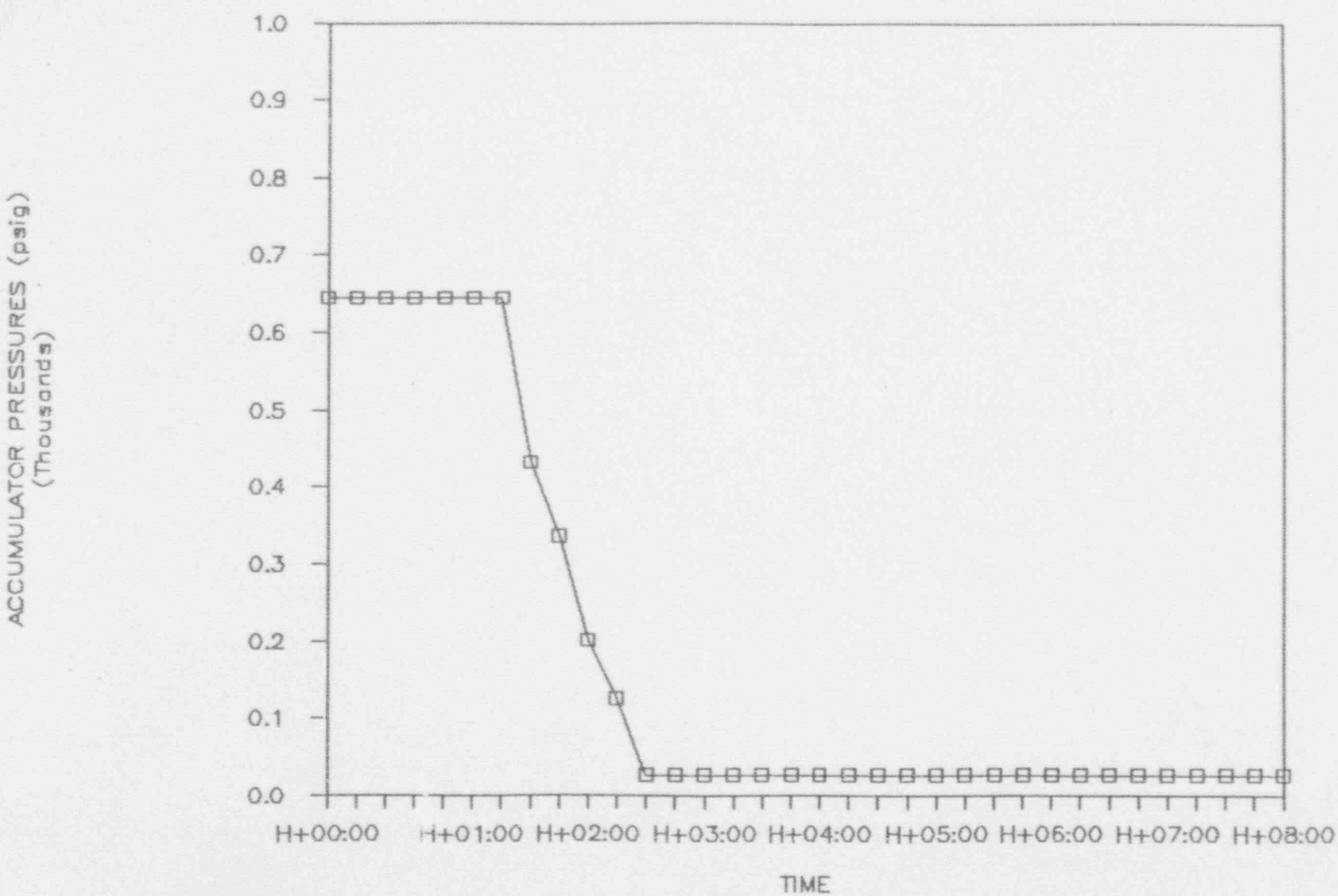
CONTAINMENT HYDROGEN CONCENTRATION

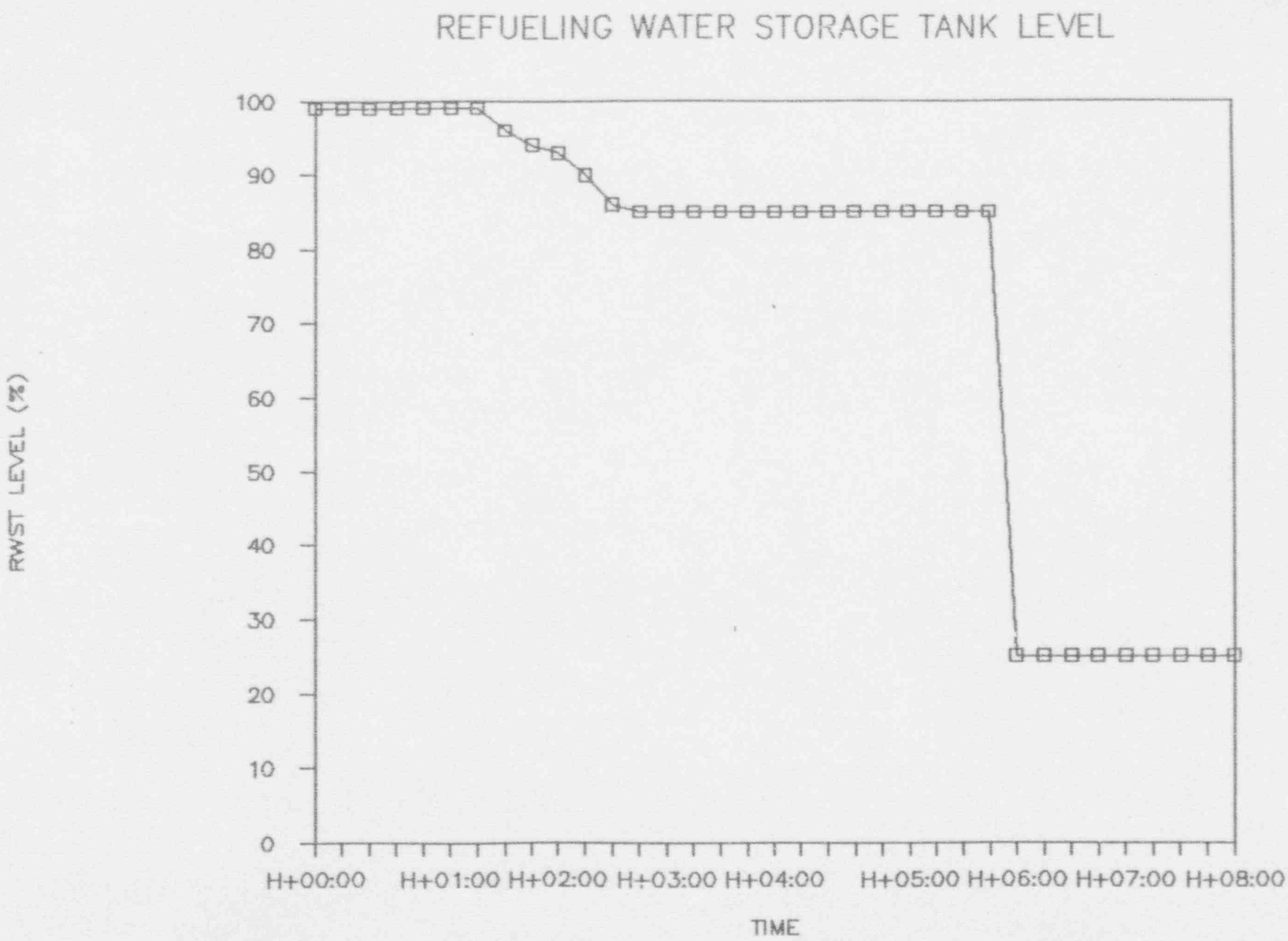



SUMP LEVEL
SUMP LEVEL (INCHES)
120
110
100
90
80
70
60
50
40
30
20
10
0
TIME
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00


ACCUMULATOR LEVEL
ACCUMULATOR LEVELS (%)
100.0
90.0
80.0
70.0
60.0
50.0
40.0
30.0
20.0
10.0
0.0
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME

ACCUMULATOR PRESSURE

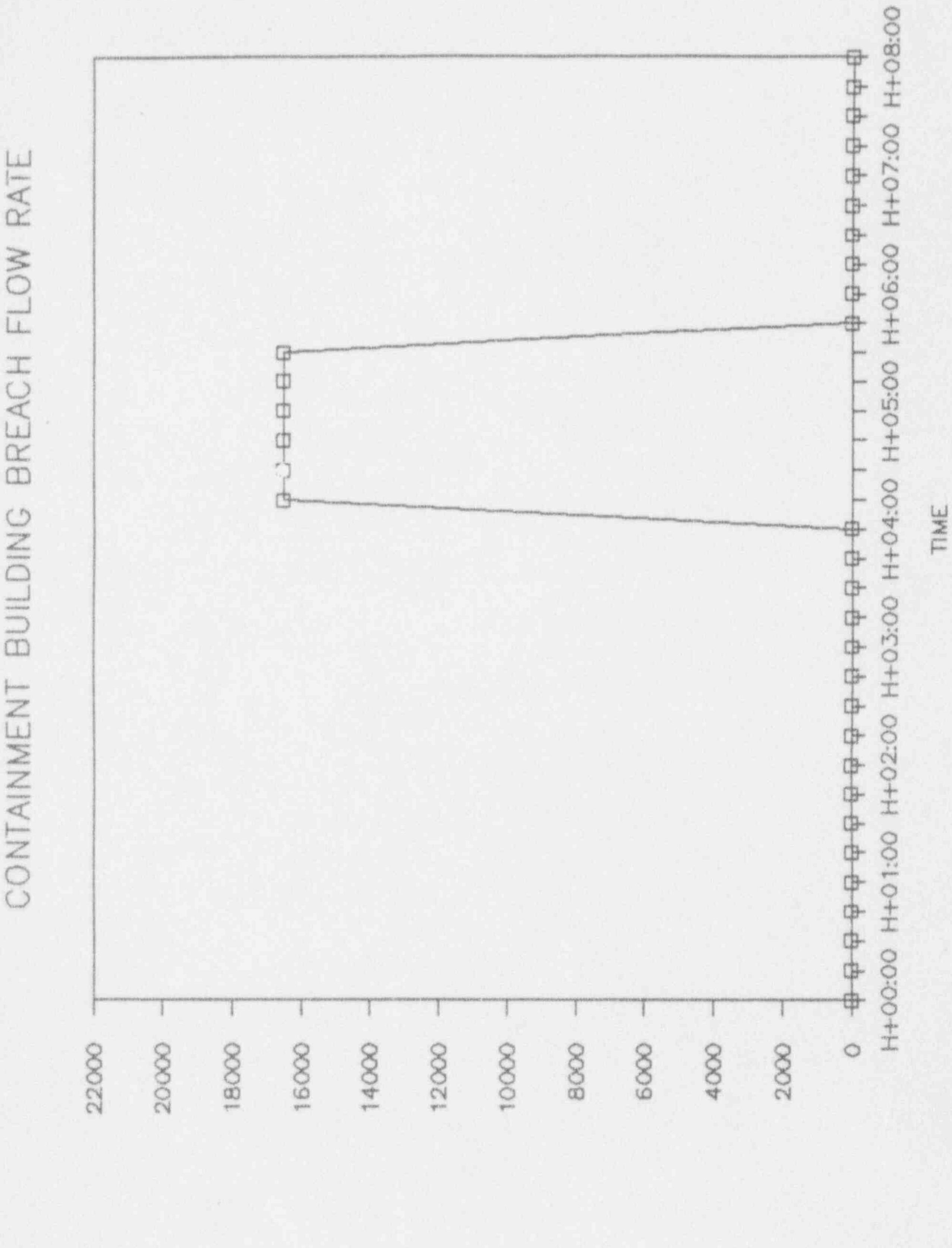



REFUELING WATER STORAGE TANK LEVEL
RWST LEVEL (%)
100
90
80
70
60
50
40
30
20
10
0
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME

CONTAINMENT BUILDING BREACH FLOW RATES

Time-related containment building (ctmt bldg) breach flow rates are provided in the following subsection. The data is depicted in graphic form versus time in units of cubic feet per minute (cfm).

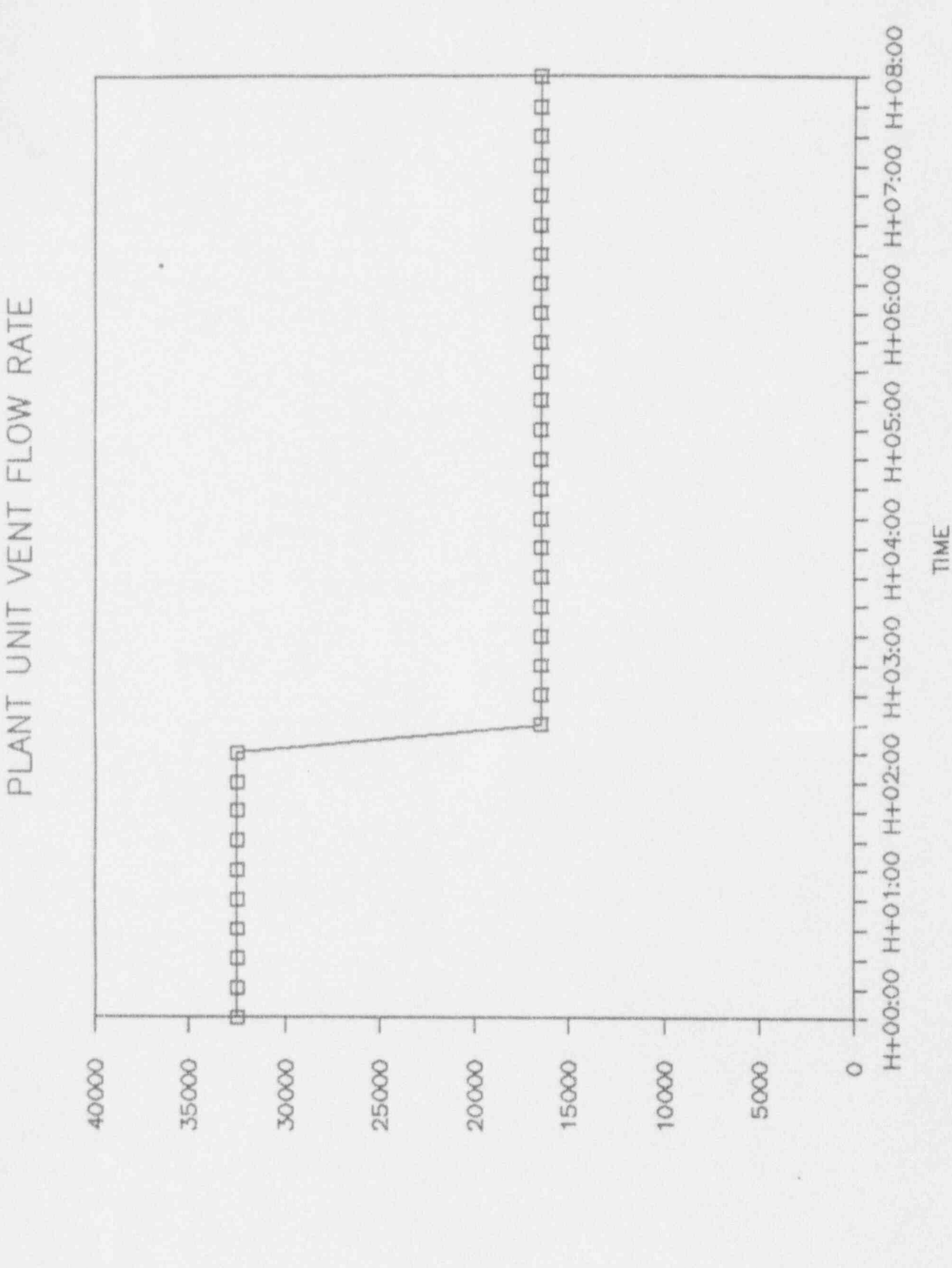
Flow rates graphed are calculated at 14.7 psia and 100 F and not at the conditions of inside the ctmt bldg.


CONTAINMENT BUILDING BREACH FLOW RATE
FLOW (CFM)
22000
20000
18000
16000
14000
12000
10000
8000
6000
4000
2000
0
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00
TIME

PLANT UNIT VENT FLOW RATE

Time-related plant unit vent flow rates are provided in the following subsection. The data is depicted in graphic forms versus time in units of cubic feet per minute (cfm).

Flow rates graphed are calculated from the conditions existing at the unit vent, 14.7 psia and 100 F.


PLANT UNIT VENT FLOW RATE
FLOW (CFM)
40000
35000
30000
25000
20000
15000
10000
5000
0
TIME
H+00:00
H+01:00
H+02:00
H+03:00
H+04:00
H+05:00
H+06:00
H+07:00
H+08:00

CORE DAMAGE ASSESSMENT

A precalculated core inventory necessary in assessing core damage is included in the following subsection.

The core inventory is a summation of curies present in the total mass of fuel pellets at the time of damage to the core. The inventory can be considered to be a corrected fuel pellet inventory for accident assessment purposes. Various assumptions utilized are listed on the following pages.

CORE DAMAGE ASSESSMENT

CORE INVENTORY

Nuclide	Corrected Fuel Pellet Inventory (Ci)
KR-83M	1.48E+07
KR-85M	4.62E+07
KR-85	1.46E+06
KR-87	8.32E+07
KR-88	1.14E+08
XE-133M	4.93E+06
XE-133	2.00E+08
XE-135M	5.55E+07
XE-135	1.91E+08
XE-138	1.70E+08
I-131	8.94E+07
I-132	1.36E+08
I-133	2.00E+08
I-134	2.34E+08
I-135	1.82E+08
RB-88	1.07E+08
CE-144	6.80E+07
TE-132	2.30E+08
CS-134	3.70E+07
CS-137	1.80E+07
CS-138	7.01E+07
LA-140	2.93E+08
LA-142	3.00E+07
BA-140	2.70E+08

CORE DAMAGE ASSESSMENT

Assumptions:

1. Clad gap activities can be assumed to be 10.0% of fuel pellet activity for all isotopes except KR-85 which is 30% of core activity.
2. Radioactive decay, time after shutdown, is a factor for accident assessment.
3. Reactor coolant system volume is static at 10600 cu ft, 3.00E+08 cc.

Results: Core damage should be assessed to be failure of 100%-25% fuel cladding and overtemperature conditions in approximately 20% of the core. A zircolay-water reaction occurred adequate enough to produce greater than 4.0% hydrogen in the containment building.

SECTION 6.0

<u>METEOROLOGICAL DATA</u>

METEOROLOGICAL DATA

(NPIS Met. Data Read Out)

TIME (ACTUAL)	800	815	830	845	900	915	930	945	1000	1015	1030	1045	1100	1115	1130	1145
(RELATIVE)	12:00 AM	12:15 AM	12:30 AM	12:45 AM	1:00 AM	1:15 AM	1:30 AM	1:45 AM	2:00 AM	2:15 AM	2:30 AM	2:45 AM	3:00 AM	3:15 AM	3:30 AM	3:45 AM
WIND SPEED (KPH)																
10M	12.6	12.7	12.6	12.8	12.6	12.8	12.6	12.7	12.6	12.8	12.6	12.7	12.6	12.8	12.6	12.7
35M	13.8	13.8	13.9	14	13.9	14	13.8	13.8	13.9	14	13.8	13.8	13.9	14	13.8	13.8
60M	14.8	14.7	14.6	14.7	14.6	14.7	14.8	14.7	14.6	14.7	14.8	14.7	14.6	14.7	14.8	14.7
WIND SPEED (MPH)																
10M	7.9	7.9	7.9	8.0	7.9	8.0	7.9	7.9	7.9	8.0	7.9	7.9	7.9	8.0	7.9	7.9
35M	8.6	8.6	8.7	8.8	8.7	8.8	8.6	8.6	8.7	8.8	8.6	8.6	8.7	8.8	8.6	8.6
60M	9.3	9.2	9.1	9.2	9.1	9.2	9.3	9.2	9.1	9.2	9.3	9.2	9.1	9.2	9.3	9.2
WIND DIR (DEG FROM)																
10M	225	224	227	224	225	224	227	224	225	224	227	224	225	225	224	224
35M	234	233	235	235	234	233	235	235	234	233	235	235	234	234	233	235
60M	241	242	241	240	241	242	241	240	241	242	241	240	241	241	242	240
WIND DIR VAR. (DEG)																
10M	12.2	12.4	12.4	12.2	12.4	12.2	12.2	12.4	12.4	12.2	12.2	12.4	12.4	12.2	12.2	12.4
60M	11.9	11.9	12	12	12	12	11.9	11.9	12	12	11.9	11.9	12	12	11.9	11.9
VERT TEMP DIFF (DEG F)																
10-35M	-0.63	-0.63	-0.65	-0.65	-0.63	-0.63	-0.65	-0.65	-0.63	-0.63	-0.65	-0.65	-0.63	-0.63	-0.63	-0.65
10-60M	-1.28	-1.28	-1.29	-1.30	-1.28	-1.28	-1.28	-1.30	-1.28	-1.28	-1.28	-1.30	-1.28	-1.28	-1.28	-1.30
10-85M	-1.96	-1.98	-1.98	-1.98	-1.96	-1.98	-1.98	-1.98	-1.96	-1.98	-1.98	-1.98	-1.96	-1.96	-1.98	-1.98
DEW POINT (DEG F)																
10M	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4
REF TEMP (DEG F)																
10M	80.0	81.8	81.8	81.8	80.0	81.8	81.8	81.8	80.0	81.8	81.8	81.8	80.0	80.0	81.8	81.8
PRECIP (IN)	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
STAB CLASS	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0

METEOROLOGICAL DATA

(NPIS Met. Data Read Out)

TIME (ACTUAL)	1200	1215	1230	1245	1300	1315	1330	1345	1400	1415	1430	1445	1500	1515	1530	1545
(RELATIVE)	3:30 AM	3:45 AM	4:00 AM	4:15 AM	4:30 AM	4:45 AM	5:00 AM	5:15 AM	5:30 AM	5:45 AM	6:00 AM	6:15 AM	6:30 AM	6:45 AM	7:00 AM	7:15 AM
WIND SPEED (KPH)																
10M	12.6	12.8	12.6	15.6	15.9	16	16.1	16.1	15.9	15.6	15.9	16	16.1	16.1	15.9	16.3
35M	13.8	14	13.8	13.8	13.9	14	13.9	15.1	16.9	17.6	17.8	17.9	18.2	17.9	18	18.1
60M	14.8	14.7	14.8	14.7	14.6	14.7	14.8	16.9	17.5	18.4	18.7	18.8	18.7	18.9	18.6	18.7
WIND SPEED (MPH)																
10M	7.9	8.0	7.9	9.8	9.9	10.0	10.1	10.1	9.9	9.8	9.9	10.0	10.1	10.1	9.9	10.2
35M	8.6	8.8	8.6	8.6	8.7	8.8	8.7	9.4	10.6	11.0	11.1	11.2	11.4	11.2	11.3	11.3
60M	9.3	9.2	9.3	9.2	9.1	9.2	9.3	10.6	10.9	11.5	11.7	11.8	11.7	11.8	11.6	11.7
WIND DIR (DEG FROM)																
10M	227	224	224	224	227	224	226	232	235	223	225	222	225	226	228	229
35M	235	233	233	233	235	235	235	237	241	228	231	229	232	233	234	235
60M	241	242	242	242	241	240	239	242	245	233	235	230	237	237	238	236
WIND DIR VAR. (DEG)																
10M	12.2	12.2	12.2	12.7	12.4	12.7	13	13	13.4	13.4	13.4	13.6	13.6	13.6	14.1	14.3
60M	11.9	12	11.9	12.5	12	12.5	12.8	12.8	13.1	13.1	13.1	13.4	13.4	13.4	13.6	14.1
VERT TEMP DIFF (DEG F)																
10-35M	-0.65	-0.63	-0.63	-0.67	-0.67	-0.67	-0.68	-0.68	-0.68	-0.68	-0.68	-0.68	-0.68	-0.70	-0.70	-0.72
10-60M	-1.28	-1.28	-1.28	-1.35	-1.37	-1.37	-1.37	-1.37	-1.37	-1.37	-1.37	-1.37	-1.37	-1.40	-1.40	-1.42
10-85M	-1.98	-1.98	-1.98	-2.03	-2.03	-2.03	-2.05	-2.05	-2.05	-2.05	-2.05	-2.05	-2.07	-2.09	-2.09	-2.12
DEW POINT (DEG F)																
10M	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4
REF TEMP (DEG F)																
10M	81.8	81.8	81.8	83.6	83.6	83.6	85.4	85.4	85.4	85.4	85.4	85.4	85.4	87.2	87.2	89.0
PRECIP (IN)	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
STAB CLASS	D	D	D	C	C	C	C	C	C	C	C	C	C	C	C	C

METEOROLOGICAL DATA

(NPIS Met. Data Read Out)

TIME (ACTUAL)	1600	1615	1630	1645	1700	1715	1730	1745	1800	1815	1830	1845
(RELATIVE)	7:30 AM	7:45 AM	8:00 AM	8:15 AM	8:30 AM	8:45 AM	9:00 AM	9:15 AM	9:30 AM	9:45 AM	10:00 AM	10:15 AM
WIND SPEED (KPH)												
10M	16.2	16.3	16.1	16	15.9	16.1	16.1	16	15.9	16.1	15.9	16.1
35M	18	17.8	17.7	18	18.1	17.9	17.7	18	18.1	17.9	18.1	17.9
60M	18.9	19	18.7	18.9	18.9	18.8	18.7	18.9	18.9	18.8	18.9	18.8
WIND SPEED (MPH)												
10M	10.1	10.2	10.1	10.0	9.9	10.1	10.1	10.0	9.9	10.1	9.9	10.1
35M	11.3	11.1	11.1	11.3	11.3	11.2	11.1	11.3	11.3	11.2	11.3	11.2
60M	11.8	11.9	11.7	11.8	11.8	11.8	11.7	11.8	11.8	11.8	11.8	11.8
WIND DIR (DEG FROM)												
10M	231	229	228	230	227	230	226	227	227	230	226	227
35M	234	232	231	234	233	235	235	234	233	235	235	234
60M	236	237	238	238	237	237	238	236	237	237	238	236
WIND DIR VAR. (DEG)												
10M	14.8	15.1	15.7	15.9	16	16.4	16.8	17	16	16.4	16.8	17
60M	14.3	14.8	15.1	15.7	15.9	16	16.4	16.8	15.9	16	16.4	16.8
VERT TEMP DIFF (DEG F)												
10-35M	-0.72	-0.72	-0.74	-0.74	-0.74	-0.74	-0.74	-0.74	-0.74	-0.74	-0.74	-0.74
10-60M	-1.42	-1.44	-1.46	-1.48	-1.48	-1.49	-1.46	-1.48	-1.48	-1.49	-1.48	-1.49
10-85M	-2.16	-2.18	-2.20	-2.21	-2.23	-2.23	-2.20	-2.21	-2.23	-2.23	-2.23	-2.23
DEW POINT (DEG F)												
10M	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4	76.4
REF TEMP (DEG F)												
10M	89.0	90.8	90.8	90.8	90.8	92.6	90.8	90.8	90.8	92.6	90.8	92.6
PRECIP (IN)	0	0	0	0	0	0	0	0	0	0	0	0
STAB CLASS	C	C	C	C	C	C	C	C	C	C	C	C

SECTION 7.0

ONSITE RADIOLOGICAL PARAMETERS

PROCESS RADIOCHEMISTRY

Time-related process radiochemistry is provided in the following subsection. Concentration data is provided by isotope in units of uCi/cc for the reactor coolant system (RCS) including the containment building (ctmt bldg) emergency sump, the ctmt atmosphere, and the plant unit vent.

Isotopic concentrations are determined using the radiation monitoring system as a reference. All samples are determined at 14.7 psia and 100 F. Concentrations will change considerably if referenced from another pressure or temperature.

Time frames at the top of the page are based on the time at which the sample is collected, not when it was analyzed.

A time delay of 30 minutes should be allowed from the time a post-accident sampling system (PASS) sample is collected to when the sample is analyzed and results are obtained.

1997 Field Exercise Data

CONTAINMENT ATM.								
TIME	H+0:00 TO H+0:30	H+0:45	H+1:00	H+1:15	H+1:30	H+1:45	H+2:00	H+ 2:15
NUCLIDE (uCi/cc)								
Kr-85m	6.87E-07	3.08E-05	4.45E-05	5.51E-05	5.69E-05	5.29E-05	6.19E-03	1.93E+01
Kr-87	1.24E-06	5.05E-05	6.61E-05	7.42E-05	6.96E-05	5.86E-05	6.22E-03	1.76E+01
Kr-88	1.70E-05	7.49E-05	1.06E-04	1.28E-04	1.29E-04	1.17E-04	1.34E-02	4.08E+01
Xe-133	3.00E-06	1.40E-04	2.10E-04	2.70E-04	2.90E-04	2.80E-04	3.40E-02	1.10E+02
Xe-133m	7.93E-08	3.71E-06	5.57E-06	7.16E-06	7.69E-06	7.42E-06	9.01E-04	2.92E+00
Xe-135	2.87E-06	1.32E-04	1.94E-04	2.45E-04	2.58E-04	2.45E-04	2.92E-02	9.27E+01
Xe-13[illegible]	8.33E-07	1.98E-05	1.51E-05	9.82E-06	5.35E-06	2.62E-06	1.62E-04	2.65E-01
Xe-138	2.55E-06	5.73E-05	4.14E-05	2.56E-05	1.32E-05	6.16E-06	3.60E-04	5.61E-01
TOTAL NOBLE GAS	1.30E-05	5.16E-04	6.83E-04	8.15E-04	8.30E-03	7.85E-04	9.04E-02	2.84E+02
I-131	3.00E-12	3.10E-12	3.60E-12	3.70E-12	3.80E-12	1.14E-11	3.90E-10	2.00E-06
I-132	4.58E-12	4.74E-12	5.50E-12	5.24E-12	4.99E-12	1.39E-11	4.40E-10	1.80E-06
I-133	6.74E-12	6.96E-12	8.08E-12	8.24E-12	8.41E-12	2.50E-11	8.50E-10	4.27E-06
I-134	7.87E-12	8.13E-12	9.44E-12	7.95E-12	6.70E-12	1.65E-11	4.62E-10	1.31E-06
I-135	6.11E-12	6.32E-12	7.34E-12	7.35E-12	7.36E-12	2.15E-11	7.18E-10	3.41E-06
TOTAL IODINE	2.83E-11	2.93E-11	3.40E-11	3.25E-11	3.13E-11	8.83E-11	2.86E-09	1.28E-05
DOSE EQUIVLENT IODINE	5.63E-12	5.82E-12	6.76E-12	6.87E-12	6.98E-12	2.07E-11	7.04E-10	3.53E-06
Cs-134	7.44E-12	8.63E-12	8.51E-12	1.82E-11	2.77E-11	3.09E-11	9.24E-10	4.79E-06
Cs-137	5.58E-12	6.47E-12	6.38E-12	1.37E-11	2.08E-11	2.30E-11	6.93E-10	3.59E-06
Cs-138	5.39E-10	6.25E-10	4.47E-10	6.92E-10	7.62E-10	8.10E-10	1.84E-08	3.62E-05
La-140	4.65E-14	4.60E-14	4.60E-14	4.74E-14	5.00E-14	5.50E-14	4.54E-11	2.72E-06
Ba-140	7.10E-14	7.00E-14	7.85E-14	7.45E-14	7.50E-14	9.69E-14	7.06E-11	4.24E-06
Total Particulates	5.52E-10	6.40E-10	4.62E-10	7.24E-10	8.11E-10	8.64E-10	2.00E-08	5.15E-05
GROSS TOTAL ACTIVITY	1.30E-05	5.16E-04	6.83E-04	8.15E-04	8.30E-03	7.85E-04	9.04E-02	2.84E+02

1993 Field Exercise Data

CONTAINMENT ATM.								
TIME	H+2:30	H+2:45	H+3:00	H+3:15	H+3:30	H+3:45	H+4:00	H+4:15
NUCLIDE (uCi/cc)								
Kr-85m	1.89E+02	1.82E+02	1.91E+02	1.69E+02	1.48E+02	1.57E+02	1.24E+02	1.44E+02
Kr-87	1.56E+02	1.36E+02	1.29E+02	1.04E+02	8.25E+01	7.90E+01	5.67E+01	5.97E+01
Kr-88	3.91E+02	3.68E+02	3.77E+02	3.26E+02	2.79E+02	2.88E+02	2.23E+02	2.53E+02
Xe-133	1.12E+03	1.12E+03	1.22E+03	1.12E+03	1.02E+03	1.12E+03	9.20E+02	1.11E+03
Xe-133m	2.97E+01	2.97E+01	3.23E+01	2.97E+01	2.70E+01	2.97E+01	2.42E+01	2.94E+01
Xe-135	9.26E+02	9.10E+02	9.73E+02	8.77E+02	7.85E+02	8.46E+02	6.82E+02	8.08E+02
Xe-135m	1.37E+00	6.96E-01	3.85E-01	1.79E-01	8.29E-02	4.62E-02	1.93E-02	1.18E-02
Xe-138	2.75E+00	1.32E+00	6.94E-01	3.07E-01	1.35E-01	7.11E-02	2.81E-02	1.63E-02
TOTAL NOBLE GAS	2.82E+03	2.75E+03	2.92E+03	2.63E+03	2.34E+03	2.52E+03	2.03E+03	2.40E+03
I-131	2.00E-01	3.00E+00	7.10E+00	7.00E+00	4.90E+01	5.00E+01	4.80E+01	5.00E+01
I-132	1.67E-01	2.32E+00	5.08E+00	4.64E+00	3.10E+01	2.85E+01	2.54E+01	2.45E+01
I-133	4.24E-01	6.32E+00	1.48E+01	1.45E+01	1.01E+02	1.02E+02	9.76E+01	1.01E+02
I-134	1.07E-01	1.32E+00	2.55E+00	2.06E+00	1.18E+01	9.91E+00	7.80E+00	6.66E+00
I-135	3.33E-01	4.87E+00	1.12E+01	1.08E+01	7.37E+01	7.33E+01	6.86E+01	6.97E+01
TOTAL IODINE	1.23E+00	1.78E+01	4.07E+01	3.90E+01	2.66E+02	2.64E+02	2.47E+02	2.52E+02
DOSE EQUIVLENT IODINE	3.50E-01	5.22E+00	1.23E+01	1.20E+01	8.38E+01	8.50E+01	8.12E+01	8.41E+01
Cs-134	4.65E-01	4.79E-01	9.71E+00	9.57E+00	6.40E+01	6.53E+01	6.27E+01	6.53E+01
Cs-137	3.49E-01	3.59E-01	7.28E+00	7.18E+00	4.80E+01	4.90E+01	4.70E+01	4.90E+01
Cs-138	2.55E+00	1.90E+00	2.78E+01	1.99E+01	9.62E+01	7.11E+01	4.94E+01	3.73E+01
La-140	8.05E-01	8.15E-01	2.82E+01	5.43E+01	3.61E+02	3.71E+02	3.51E+02	3.71E+02
Ba-140	1.26E+00	1.28E+00	4.44E+01	8.57E+01	5.72E+02	5.89E+02	5.59E+02	5.93E+02
Total Particulates	5.43E+00	4.83E+00	1.17E+02	1.77E+02	1.14E+03	1.15E+03	1.07E+03	1.12E+03
GROSS TOTAL ACTIVITY	2.82E+03	2.77E+03	3.08E+03	2.84E+03	3.75E+03	3.93E+03	3.35E+03	3.77E+03

1993 Field Exercise Data

CONTAINMENT ATM.								
TIME	H+4:30	H+4:45	H+5:00	H+5:15	H+5:30	H+5:45	H+6:00	H+6:15
NUCLIDE (uCi/cc)								
Kr-85m	1.50E+02	1.30E+02	1.46E+02	1.17E+02	1.02E+02	1.09E+02	1.09E+02	1.10E+02
Kr-87	5.63E+01	4.42E+01	4.51E+01	3.28E+01	2.59E+01	2.50E+01	2.28E+01	2.08E+01
Kr-88	2.58E+02	2.18E+02	2.39E+02	1.88E+02	1.60E+02	1.66E+02	1.64E+02	1.61E+02
Xe-133	1.20E+03	1.08E+03	1.26E+03	1.05E+03	9.50E+02	1.05E+03	1.10E+03	1.15E+03
Xe-133m	3.18E+01	2.86E+01	3.34E+01	2.78E+01	2.52E+01	2.78E+01	2.91E+01	3.05E+01
Xe-135	8.58E+02	7.58E+02	8.69E+02	7.11E+02	6.32E+02	6.86E+02	7.05E+02	7.24E+02
Xe-135m	6.47E-03	2.95E-03	1.75E-03	7.40E-04	3.40E-04	1.91E-04	1.01E-04	5.38E-05
Xe-138	8.51E-03	3.69E-03	2.07E-03	8.31E-04	3.62E-04	1.93E-04	9.72E-05	4.89E-05
TOTAL NOBLE GAS	2.55E+03	2.26E+03	2.59E+03	2.13E+03	1.90E+03	2.06E+03	2.13E+03	2.20E+03
I-131	5.00E+01	4.80E+01	4.90E+01	4.70E+01	4.60E+01	4.70E+01	4.75E+01	4.80E+01
I-132	2.27E+01	2.02E+01	1.91E+01	1.70E+01	1.54E+01	1.46E+01	1.37E+01	1.28E+01
I-133	1.00E+02	9.56E+01	9.69E+01	9.23E+01	8.97E+01	9.10E+01	9.13E+01	9.16E+01
I-134	5.46E+00	4.30E+00	3.60E+00	2.83E+00	2.27E+00	1.90E+00	1.57E+00	1.30E+00
I-135	6.79E+01	6.36E+01	6.33E+01	5.92E+01	5.65E+01	5.62E+01	5.54E+01	5.46E+01
TOTAL IODINE	2.46E+02	2.32E+02	2.32E+02	2.18E+02	2.10E+02	2.11E+02	2.09E+02	2.08E+02
DOSE EQUIVLENT IODINE	8.37E+01	8.00E+01	8.12E+01	7.76E+01	7.56E+01	7.69E+01	7.74E+01	7.78E+01
Cs-134	6.55E+01	6.31E+01	6.45E+01	6.07E+01	5.93E+01	6.07E+01	6.13E+01	6.20E+01
Cs-137	4.91E+01	4.73E+01	4.84E+01	4.55E+01	4.45E+01	4.55E+01	4.60E+01	4.65E+01
Cs-138	2.71E+01	1.89E+01	1.40E+01	9.51E+00	6.74E+00	4.99E+00	3.65E+00	2.67E+00
La-140	3.74E+02	3.58E+02	3.71E+02	3.45E+02	3.35E+02	3.45E+02	3.50E+02	3.55E+02
Ba-140	6.00E+02	5.76E+02	5.98E+02	5.58E+02	5.44E+02	5.61E+02	5.71E+02	5.81E+02
Total Particulates	1.12E+03	1.06E+03	1.10E+03.	1.02E+03	9.90E+02	1.02E+03	1.03E+03	1.05E+03
GROSS TOTAL ACTIVITY	3.92E+03	3.55E+03	3.92E+03	3.36E+03	3.04E+03	3.29E+03	3.36E+03	3.45E+03

93-FE

93 Field Exercise Data

CONTAINMENT ATM.							
TIME	H+6:30	H+6:45	H+7:00	H+7:15	H+7:30	H+7:45	H+8:00
NUCLIDES in uCi/cc							
Kr-85m	9.69E+01	9.51E+01	9.41E+01	8.65E+01	8.65E+01	8.55E+01	8.11E+01
Kr-87	1.66E+01	1.48E+01	1.33E+01	1.10E+01	1.00E+01	9.85E+00	8.71E+00
Kr-88	1.38E+02	1.33E+02	1.28E+02	1.15E+02	1.13E+02	1.05E+02	9.45E+01
Xe-133	1.05E+03	1.07E+03	1.10E+03	1.05E+03	1.09E+03	1.05E+03	1.05E+03
Xe-133m	2.78E+01	2.84E+01	2.91E+01	2.78E+01	2.89E+01	2.80E+01	2.82E+01
Xe-135	6.49E+02	6.50E+02	6.56E+02	6.15E+02	6.27E+02	6.00E+02	6.03E+02
Xe-135m	2.49E-05	1.29E-05	6.72E-06	3.26E-06	1.72E-06	8.55E-07	5.10E-07
Xe-138	2.15E-05	1.06E-05	5.22E-06	2.40E-06	1.20E-06	7.82E-07	3.52E-07
TOTAL NOBLE GAS	2.00E+03	2.01E+03	2.04E+03	1.92E+03	1.97E+03	1.88E+03	1.87E+03
I-131	4.70E+01	4.72E+01	4.75E+01	4.70E+01	4.74E+01	4.71E+01	4.70E+01
I-132	1.16E+01	1.08E+01	1.01E+01	9.26E+00	8.65E+00	7.92E+00	7.75E+00
I-133	8.91E+01	8.88E+01	8.88E+01	8.72E+01	8.73E+01	8.81E+01	7.91E+01
I-134	1.05E+00	8.62E-01	7.11E-01	5.77E-01	4.77E-01	3.69E-01	2.75E-01
I-135	5.21E+01	5.10E+01	5.01E+01	4.83E+01	4.75E+01	4.56E+01	4.50E+01
TOTAL IODINE	2.01E+02	1.99E+02	1.97E+02	1.92E+02	1.91E+02	1.89E+02	1.79E+02
DOSE EQUIVLENT IODINE	7.59E+01	7.59E+01	7.61E+01	7.50E+01	7.53E+01	7.50E+01	7.24E+01
Cs-134	6.53E+01	6.56E+01	6.60E+01	6.53E+01	6.12E+01	6.25E+01	6.20E+01
Cs-137	4.90E+01	4.92E+01	4.96E+01	4.90E+01	4.59E+01	4.62E+01	4.59E+01
Cs-138	2.04E+00	1.48E+00	1.08E+00	7.73E-01	5.25E-01	3.42E-01	2.00E-01
La-140	3.71E+02	3.73E+02	3.76E+02	3.71E+02	3.48E+02	3.39E+02	3.40E+02
Ba-140	6.09E+02	6.14E+02	6.21E+02	6.15E+02	5.78E+02	5.50E+02	5.55E+02
Total Particulates	1.17E+03	1.18E+03	1.19E+03	1.17E+03	1.10E+03	9.89E+02	1.00E+03
GROSS TOTAL ACTIVITY	3.37E+03	3.38E+03	3.43E+03	3.29E+03	3.26E+03	3.07E+03	3.05E+03

7. 5

1993 Field Exercise Data

RCS / CONT. SUMP								
TIME	H+0:00 TO H+0:30	H+0:45	H+1:00	H+1:15	H+1:30	H+1:45	H+2:00	H+ 2:15
NUCLIDE (uCi/cc)								
Kr-85m	1.17E-02	1.10E-03	5.29E-04	4.89E-05	4.90E-06	4.53E-06	1.27E-03	2.63E+00
Kr-87	2.12E-02	1.80E-03	7.87E-04	6.59E-05	5.99E-06	5.02E-06	1.28E-03	2.39E+00
Kr-88	2.90E-02	2.67E-03	1.26E-03	1.14E-04	1.11E-05	1.01E-05	2.76E-03	5.56E+00
Xe-133	5.11E-02	5.00E-03	2.50E-03	2.40E-04	2.50E-05	2.40E-05	6.99E-03	1.50E+01
Xe-133m	1.35E-03	1.32E-04	6.60E-05	6.35E-06	6.62E-07	6.35E-07	1.85E-04	3.97E-01
Xe-135	4.88E-02	4.69E-03	2.30E-03	2.17E-04	2.58E-05	2.09E-05	6.00E-03	1.26E+01
Xe-135m	1.42E-02	7.06E-04	1.79E-04	8.72E-06	4.61E-07	2.25E-07	3.33E-05	3.62E-02
Xe-138	4.34E-02	2.04E-03	4.92E-04	2.27E-05	1.14E-06	5.27E-07	7.41E-05	7.64E-02
TOTAL NOBLE GAS	2.21E-01	1.81E-02	8.29E-03	7.24E-04	7.51E-05	6.59E-05	1.86E-02	3.87E+01
I-131	2.59E-03	5.29E-04	5.29E-04	5.29E-04	5.39E-04	5.29E-04	1.30E-02	9.99E+01
I-132	3.95E-03	7.50E-04	6.95E-04	6.44E-04	6.09E-04	5.54E-04	1.26E-02	8.98E+01
I-133	5.81E-03	1.18E-03	1.17E-03	1.16E-03	1.18E-03	1.15E-03	2.79E-02	2.13E+02
I-134	6.78E-03	1.14E-03	9.33E-04	7.65E-04	6.39E-04	5.14E-04	1.03E-02	6.52E+01
I-135	5.27E-03	1.05E-03	1.03E-03	1.00E-03	9.93E-04	9.50E-04	2.27E-02	1.70E+02
TOTAL IODINE	2.44E-02	4.65E-03	4.36E-03	4.10E-03	3.96E-03	3.70E-03	8.65E-02	6.38E+02
DOSE EQUIVLENT IODINE	4.86E-03	9.82E-04	9.72E-04	9.62E-04	9.74E-04	9.48E-04	2.31E-02	1.76E+02
Cs-134	2.66E-04	4.67E-05	4.67E-05	4.80E-05	4.80E-05	4.53E-05	6.45E-03	9.33E-01
Cs-137	1.69E-04	3.50E-05	3.50E-05	3.60E-05	3.60E-05	3.40E-05	4.84E-03	7.00E-01
Cs-138	1.64E-02	2.45E-03	1.77E-03	1.32E-03	9.56E-04	6.54E-04	6.74E-02	7.06E+00
La-140	1.35E-06	2.99E-07	2.99E-07	3.09E-07	3.09E-07	2.89E-07	3.85E-05	5.28E-01
Ba-140	2.07E-06	4.58E-07	4.59E-07	4.76E-07	4.77E-07	4.48E-07	6.00E-05	8.24E-01
Total Particulates	1.68E-02	2.53E-03	1.85E-03	1.40E-03	1.04E-03	8.05E-04	7.88E-02	1.00E+01
GROSS TOTAL ACTIVITY	2.62E-01	2.53E-02	1.45E-02	6.23E-03	5.08E-03	4.56E-03	1.84E-01	6.87E+02

1993 Field Exercise Data

RCS / CONT. SUMP								
TIME	H+2:30	H+2:45	H+3:00	H+3:15	H+3:30	H+3:45	H+4:00	H+4:15
NUCLIDE (uCi/cc)								
Kr-85m	8.10E+00	7.63E+00	7.51E+00	7.08E+00	6.67E+00	6.56E+00	6.05E+00	6.09E+00
Kr-87	6.68E+00	5.71E+00	5.09E+00	4.35E+00	3.72E+00	3.31E+00	2.77E+00	2.52E+00
Kr-88	1.67E+01	1.54E+01	1.48E+01	1.37E+01	1.26E+01	1.21E+01	1.09E+01	1.07E+01
Xe-133	4.80E+01	4.70E+01	4.80E+01	4.70E+01	4.60E+01	4.70E+01	4.50E+01	4.70E+01
Xe-133m	1.27E+00	1.24E+00	1.27E+00	1.24E+00	1.22E+00	1.24E+00	1.19E+00	1.24E+00
Xe-135	3.97E+01	3.81E+01	3.82E+01	3.68E+01	3.53E+01	3.55E+01	3.33E+01	3.42E+01
Xe-135m	5.87E-02	2.92E-02	1.51E-02	7.52E-03	3.73E-03	1.94E-03	9.41E-04	4.99E-04
Xe-138	1.18E-01	5.55E-02	2.73E-02	1.29E-02	6.06E-03	2.98E-03	1.38E-03	6.91E-04
TOTAL NOBLE GAS	1.21E+02	1.15E+02	1.15E+02	1.10E+02	1.06E+02	1.06E+02	9.92E+01	1.02E+02
I-131	5.99E+03	8.99E+03	9.99E+03	2.00E+04	2.00E+04	2.10E+04	1.90E+04	2.10E+04
I-132	4.99E+03	6.94E+03	7.15E+03	1.33E+04	1.23E+04	1.20E+04	1.00E+04	1.03E+04
I-133	1.27E+04	1.89E+04	2.09E+04	4.15E+04	4.12E+04	4.30E+04	3.86E+04	4.24E+04
I-134	3.21E+03	3.94E+03	3.59E+03	5.89E+03	4.83E+03	4.16E+03	3.08E+03	2.79E+03
I-135	9.97E+03	1.46E+04	1.58E+04	3.08E+04	3.00E+04	3.07E+04	2.71E+04	2.92E+04
TOTAL IODINE	3.69E+04	5.34E+04	5.74E+04	1.11E+05	1.08E+05	1.11E+05	9.78E+04	1.06E+05
DOSE EQUIVLENT IODINE	1.05E+04	1.56E+04	1.73E+04	3.44E+04	3.42E+04	3.57E+04	3.21E+04	3.53E+04
Cs-134	1.73E+00	1.87E+00	2.00E+00	1.87E+00	1.73E+00	1.87E+00	1.60E+00	1.87E+00
Cs-137	1.30E+00	1.40E+00	1.50E+00	1.40E+00	1.30E+00	1.40E+00	1.20E+00	1.40E+00
Cs-138	9.49E+00	7.40E+00	5.74E+00	3.88E+00	2.61E+00	2.03E+00	1.26E+00	1.07E+00
La-140	3.17E+00	4.75E+00	5.38E+00	1.06E+01	9.36E+00	1.06E+01	8.57E+00	1.06E+01
Ba-140	4.98E+00	7.46E+00	8.47E+00	1.67E+01	1.48E+01	1.68E+01	1.36E+01	1.69E+01
Total Particulates	2.07E+01	2.29E+01	2.31E+01	3.45E+01	2.98E+01	3.27E+01	2.62E+01	3.18E+01
GROSS TOTAL ACTIVITY	3.70E+04	5.35E+04	5.76E+04	1.12E+05	1.08E+05	1.11E+05	9.79E+04	1.06E+05

1993 Field Exercise Data

RCS / CONT. SUMP								
TIME	H+4:30	H+4:45	H+5:00	H+5:15	H+5:30	H+5:45	H+6:00	H+6:15
NUCLIDE (uCi/cc)								
Kr-85m	5.99E+00	5.64E+00	5.67E+00	5.23E+00	4.93E+00	4.85E+00	4.72E+00	4.60E+00
Kr-87	2.25E+00	1.93E+00	1.75E+00	1.47E+00	1.25E+00	1.12E+00	9.85E-01	8.69E-01
Kr-88	1.03E+01	9.48E+00	9.30E+00	8.39E+00	7.73E+00	7.43E+00	7.07E+00	6.72E+00
Xe-133	4.80E+01	4.70E+01	4.90E+01	4.70E+01	4.60E+01	4.70E+01	4.75E+01	4.80E+01
Xe-133m	1.27E+00	1.24E+00	1.30E+00	1.24E+00	1.22E+00	1.24E+00	1.26E+00	1.27E+00
Xe-135	3.43E+01	3.30E+01	3.38E+01	3.18E+01	3.06E+01	3.07E+01	3.04E+01	3.02E+01
Xe-135m	2.58E-04	1.28E-04	6.80E-05	3.31E-05	1.64E-05	8.52E-06	4.37E-06	2.24E-06
Xe-138	3.40E-04	1.60E-04	8.05E-05	3.72E-05	1.75E-05	8.61E-06	4.19E-06	2.04E-06
TOTAL NOBLE GAS	1.02E+02	9.83E+01	1.01E+02	9.51E+01	9.17E+01	9.23E+01	9.19E+01	9.17E+01
I-131	2.20E+04	2.10E+04	2.30E+04	2.10E+04	2.00E+04	2.10E+04	2.15E+04	2.20E+04
I-132	9.97E+03	8.83E+03	8.96E+03	7.58E+03	6.69E+03	6.51E+03	6.18E+03	5.86E+03
I-133	4.41E+04	4.18E+04	4.54E+04	4.12E+04	3.89E+04	4.06E+04	4.13E+04	4.19E+04
I-134	2.40E+03	1.88E+03	1.69E+03	1.26E+03	9.85E+02	8.48E+02	7.12E+02	5.97E+02
I-135	2.99E+04	2.78E+04	2.97E+04	2.64E+04	2.45E+04	2.51E+04	2.15E+04	2.50E+04
TOTAL IODINE	1.08E+05	1.01E+05	1.09E+05	9.74E+04	9.11E+04	9.41E+04	9.47E+04	9.54E+04
DOSE EQUIVLENT IODINE	3.68E+04	3.50E+04	3.81E+04	3.46E+04	3.28E+04	3.43E+04	3.50E+04	3.56E+04
Cs-134	2.00E+00	1.87E+00	2.13E+00	1.87E+00	1.73E+00	1.87E+00	1.93E+00	2.00E+00
Cs-137	1.50E+00	1.40E+00	1.60E+00	1.40E+00	1.30E+00	1.40E+00	1.45E+00	1.50E+00
Cs-138	8.26E-01	5.58E-01	4.62E-01	2.93E-01	1.97E-01	1.53E-01	1.15E-01	8.62E-02
La-140	1.16E+01	1.06E+01	1.25E+01	1.06E+01	9.56E+00	1.06E+01	1.11E+01	1.16E+01
Ba-140	1.85E+01	1.70E+01	2.02E+01	1.71E+01	1.56E+01	1.72E+01	1.80E+01	1.89E+01
Total Particulates	3.44E+01	3.14E+01	3.96E+01	3.13E+01	2.83E+01	3.12E+01	3.26E+01	3.41E+01
GROSS TOTAL ACTIVITY	1.09E+05	1.01E+05	1.09E+05	9.76E+04	9.12E+04	9.42E+04	9.48E+04	9.55E+04

1993 Field Exercise Data

RCS / CONT. SUMP							
TIME	H+6:30	H+6:45	H+7:00	H+7:15	H+7:30	H+7:45	H+8:00
NUCLIDE (uCi/cc)							
Kr-85m	4.33E+00	4.19E+00	4.03E+00	3.89E+00	3.76E+00	3.69E+00	3.46E+00
Kr-87	7.43E-01	6.51E-01	5.67E-01	4.94E-01	4.35E-01	3.88E-01	3.21E-01
Kr-88	6.19E+00	5.85E+00	5.49E+00	5.16E+00	4.89E+00	4.64E+00	4.28E+00
Xe-133	4.70E+01	4.72E+01	4.71E+01	4.70E+01	4.74E+01	4.72E+01	4.70E+01
Xe-133m	1.24E+00	1.25E+00	1.25E+00	1.24E+00	1.25E+00	1.24E+00	1.25E+00
Xe-135	2.90E+01	2.86E+01	2.81E+01	2.75E+01	2.72E+01	2.65E+01	2.60E+01
Xe-135m	1.11E-06	5.68E-07	2.88E-07	1.46E-07	7.45E-08	2.23E-08	8.83E-09
Xe-138	9.62E-07	4.65E-07	2.23E-07	1.07E-07	5.21E-08	2.29E-08	6.72E-08
TOTAL NOBLE GAS	8.93E+01	8.85E+01	8.72E+01	8.56E+01	8.56E+01	8.37E+01	8.23E+01
I-131	2.10E+04	2.12E+04	2.15E+04	2.10E+04	2.14E+04	2.12E+04	2.13E+04
I-132	5.19E+03	4.86E+03	4.56E+03	4.13E+03	3.90E+03	3.82E+03	3.73E+03
I-133	3.98E+04	3.99E+04	4.01E+04	3.89E+04	3.94E+04	3.99E+04	3.97E+04
I-134	4.67E+02	3.87E+02	3.22E+02	2.58E+02	2.15E+02	1.75E+02	1.62E+02
I-135	2.33E+04	2.29E+04	2.26E+04	2.16E+04	2.14E+04	2.13E+04	2.15E+04
TOTAL IODINE	8.97E+04	8.92E+04	8.91E+04	8.59E+04	8.63E+04	8.64E+04	8.64E+04
DOSE EQUIVLENT IODINE	3.39E+04	3.41E+04	3.44E+04	3.35E+04	3.40E+04	3.43E+03	3.40E+04
Cs-134	1.87E-01	6.21E-01	5.79E-01	5.25E-01	4.96E-01	4.72E-01	4.48E-01
Cs-137	1.40E-01	4.66E-01	4.34E-01	3.94E-01	3.72E-01	3.45E-01	3.15E-01
Cs-138	5.82E-03	1.40E-02	9.46E-03	6.22E-03	4.25E-03	2.99E-03	1.09E-03
La-140	1.06E+00	3.52E+00	3.29E+00	2.97E+00	2.81E+00	2.52E+00	2.28E+00
Ba-140	1.73E+00	5.79E+00	5.43E+00	4.92E+00	4.67E+00	4.31E+00	3.92E+00
Total Particulates	3.33E+00	1.11E+01	1.04E+01	9.39E+00	8.89E+00	7.65E+00	6.96E+00
GROSS TOTAL ACTIVITY	8.98E+04	8.93E+04	8.92E+04	8.60E+04	8.64E+04	8.65E+04	8.65E+04

1993 Field Exercise Data

UNIT VENT								
TIME	H+0:00 TO H+0:30	H+0:45	H+1:00	H+1:15	H+1:30	H+1:45	H+1:00	H+1:15
NUCLIDE (uCi/cc)								
Kr-85m	4.50E-08	4.50E-08	4.50E-08	4.50E-08	4.50E-08	4.50E-08	4.50E-08	4.50E-08
Kr-87	8.30E-08	8.30E-08	8.30E-08	8.30E-08	8.30E-08	8.30E-08	8.30E-08	8.30E-08
Kr-88	1.14E-07	1.14E-07	1.14E-07	1.14E-07	1.14E-07	1.14E-07	1.14E-07	1.14E-07
Xe-133	2.00E-07	2.00E-07	2.00E-07	2.00E-07	2.00E-07	2.00E-07	2.00E-07	2.00E-07
Xe-133m	5.25E-09	5.25E-09	5.25E-09	5.25E-09	5.25E-09	5.25E-09	5.25E-09	5.25E-09
Xe-135	1.85E-07	1.85E-07	1.85E-07	1.85E-07	1.85E-07	1.85E-07	1.85E-07	1.85E-07
Xe-135m	5.35E-08	5.35E-08	5.35E-08	5.35E-08	5.35E-08	5.35E-08	5.35E-08	5.35E-08
Xe-138	1.55E-07	1.55E-07	1.55E-07	1.55E-07	1.55E-07	1.55E-07	1.55E-07	1.55E-07
TOTAL NOBLE GAS	8.8E-07	8.8E-07	8.8E-07	8.8E-07	8.8E-07	8.8E-07	8.8E-07	8.8E-07
I-131	1.00E-12	1.00E-12	1.00E-12	1.00E-12	1.00E-12	1.00E-12	1.00E-12	1.00E-12
I-132	1.51E-11	1.51E-11	1.51E-11	1.51E-11	1.51E-11	1.51E-11	1.51E-11	1.51E-11
I-133	2.22E-11	2.22E-11	2.22E-11	2.22E-11	2.22E-11	2.22E-11	2.22E-11	2.22E-11
I-134	2.62E-11	2.62E-11	2.62E-11	2.62E-11	2.62E-11	2.62E-11	2.62E-11	2.62E-11
I-135	2.03E-11	2.03E-11	2.03E-11	2.03E-11	2.03E-11	2.03E-11	2.03E-11	2.03E-11
TOTAL IODINE	9.4E-11	9.4E-11	9.4E-11	9.4E-11	9.4E-11	9.4E-11	9.4E-11	9.4E-11
DOSE EQUIVLENT IODINE	1.86E-11	1.86E-11	1.86E-11	1.86E-11	1.86E-11	1.86E-11	1.86E-11	1.86E-11
Cs-134	2.75E-11	2.75E-11	2.75E-11	2.75E-11	2.75E-11	2.75E-11	2.75E-11	2.75E-11
Cs-137	2.06E-11	2.06E-11	2.06E-11	2.06E-11	2.05E-11	2.06E-11	2.06E-11	2.06E-11
Cs-138	2.00E-09	2.00E-09	2.00E-09	2.00E-09	2.00E-09	2.00E-09	2.00E-09	2.00E-09
La-140	1.75E-13	1.75E-13	1.75E-13	1.75E-13	1.75E-13	1.75E-13	1.75E-13	1.75E-13
Ba-140	2.60E-13	2.60E-13	2.60E-13	2.60E-13	2.60E-13	2.60E-13	2.60E-13	2.60E-13
Total Particulates	2.31E-09	2.31E-09	2.31E-09	2.31E-09	2.31E-09	2.31E-08	2.31E-09	2.31E-09
TOTAL GROSS ACTIVITY	8.4E-07	8.4E-07	8.4E-07	8.4E-07	8.4E-07	8.4E-07	8.4E-07	8.4E-07

1993 Field Exercise Data

UNIT VENT								
TIME	H+1:30	H+1:45	H+2:00	H+2:15	H+2:30	H+2:45	H+3:00	H+3:15
NUCLIDE (uCi/cc)								
Kr-85m	4.50E-08	4.50E-08	4.50E-08	4.50E-08	4.50E-08	4.50E-08	4.50E-08	4.50E-08
Kr-87	8.30E-08	8.30E-08	8.30E-08	8.30E-08	8.30E-08	8.30E-08	8.30E-08	8.30E-08
Kr-88	1.14E-07	1.14E-07	1.14E-07	1.14E-07	1.14E-07	1.14E-07	1.14E-07	1.14E-07
Xe-133	2.00E-07	2.00E-07	2.00E-07	2.00E-07	2.00E-07	2.00E-07	2.00E-07	2.00E-07
Xe-133m	5.25E-09	5.25E-09	5.25E-09	5.25E-09	5.25E-09	5.25E-09	5.25E-09	5.25E-09
Xe-135	1.85E-07	1.85E-07	1.85E-07	1.85E-07	1.85E-07	1.85E-07	1.85E-07	1.85E-07
Xe-135m	5.35E-08	5.35E-08	5.35E-08	5.35E-08	5.35E-08	5.35E-08	5.35E-08	5.35E-08
Xe-138	1.55E-07	1.55E-07	1.55E-07	1.55E-07	1.55E-07	1.55E-07	1.55E-07	1.55E-07
TOTAL NOBLE GAS	8.8E-07	8.8E-07	8.8E-07	8.8E-07	8.8E-07	8.8E-07	8.8E-07	8.8E-07
I-131	1.00E-12	1.00E-12	1.00E-12	1.00E-12	1.00E-12	1.00E-12	1.00E-12	1.00E-12
I-132	1.51E-11	1.51E-11	1.51E-11	1.51E-11	1.51E-11	1.51E-11	1.51E-11	1.51E-11
I-133	2.22E-11	2.22E-11	2.22E-11	2.22E-11	2.22E-11	2.22E-11	2.22E-11	2.22E-11
I-134	2.62E-11	2.62E-11	2.62E-11	2.62E-11	2.62E-11	2.62E-11	2.62E-11	2.62E-11
I-135	2.03E-11	2.03E-11	2.03E-11	2.03E-11	2.03E-11	2.03E-11	2.03E-11	2.03E-11
TOTAL IODINE	9.4E-11	9.4E-11	9.4E-11	9.4E-11	9.4E-11	9.4E-11	9.4E-11	9.4E-11
DOSE EQUIVLENT IODINE	1.86E-11	1.86E-11	1.86E-11	1.86E-11	1.86E-11	1.86E-11	1.86E-11	1.86E-11
Cs-134	2.75E-11	2.75E-11	2.75E-11	2.75E-11	2.75E-11	2.75E-11	2.75E-11	2.75E-11
Cs-137	2.06E-11	2.06E-11	2.06E-11	2.06E-11	2.06E-11	2.06E-11	2.06E-11	2.06E-11
Cs-138	2.00E-09	2.00E-09	2.00E-09	2.00E-09	2.00E-09	2.00E-09	2.00E-09	2.00E-09
La-140	1.75E-13	1.75E-13	1.75E-13	1.75E-13	1.75E-13	1.75E-13	1.75E-13	1.75E-13
Ba-140	2.60E-13	2.60E-13	2.60E-13	2.60E-13	2.60E-13	2.60E-13	2.60E-13	2.60E-13
Total Particulates	2.31E-09	2.31E-09	2.31E-09	2.31E-09	2.31E-09	2.31E-09	2.31E-09	2.31E-09
TOTAL GROSS ACTIVITY	8.4E-07	8.4E-07	8.4E-07	8.4E-07	8.4E-07	8.4E-07	8.4E-07	8.4E-07

1993 Field Exercise Data

UNIT VENT								
TIME	H+3:30	H+3:45	H+4:00	H+4:15	H+4:30	H+4:45	H+5:00	H+5:15
NUCLIDE (uCi/cc)								
Kr-85m	4.50E-08	4.50E-08	4.50E-08	9.30E+01	1.22E+02	1.49E+02	1.30E+02	1.46E+02
Kr-87	8.30E-08	8.30E-08	8.30E-08	4.26E+01	5.06E+01	5.61E+01	4.44E+01	3.26E+01
Kr-88	1.14E-07	1.14E-07	1.14E-07	1.67E+02	2.15E+02	2.57E+02	2.19E+02	1.87E+02
Xe-133	2.00E-07	2.00E-07	2.00E-07	6.91E+02	9.43E+02	1.20E+03	1.08E+03	1.04E+03
Xe-133m	5.25E-09	5.25E-09	5.25E-09	1.83E+01	2.50E+01	3.17E+01	2.86E+01	2.77E+01
Xe-135	1.85E-07	1.85E-07	1.85E-07	5.13E+02	6.85E+02	8.54E+02	7.61E+02	7.07E+02
Xe-135m	5.35E-08	5.35E-08	5.35E-08	1.45E-02	5.15E-02	6.44E-03	2.96E-03	7.36E-04
Xe-138	1.55E-07	1.55E-07	1.55E-07	1.24E-02	7.11E-02	8.50E-03	3.69E-03	8.25E-04
TOTAL NOBLE GAS	8.9E-07	8.8E-07	8.8E-07	1.52E+03	2.04E+03	2.55E+03	2.26E+03	2.11E+03
I-131	1.00E-12	1.00E-12	1.00E-12	8.00E-05	1.00E-04	1.10E-04	1.00E-04	1.30E-05
I-132	1.51E-11	1.51E-11	1.51E-11	4.23E-05	4.90E-05	4.99E-05	4.21E-05	4.69E-06
I-133	2.22E-11	2.22E-11	2.22E-11	1.63E-04	2.02E-04	2.21E-04	1.99E-04	2.55E-05
I-134	2.62E-11	2.62E-11	2.62E-11	1.30E-05	1.33E-05	1.20E-05	8.95E-06	7.83E-07
I-135	2.03E-11	2.03E-11	2.03E-11	1.14E-04	1.39E-04	1.49E-04	1.32E-04	1.64E-05
TOTAL IODINE	9.4E-11	9.4E-11	9.4E-11	4.12E-04	5.04E-04	5.42E-04	4.83E-04	6.03E-05
DOSE EQUIVLENT IODINE	1.86E-11	1.86E-11	1.86E-11	1.35E-04	1.68E-04	1.84E-04	1.67E-04	2.14E-05
Cs-134	2.75E-11	2.75E-11	2.75E-11	1.14E-05	1.17E-05	1.18E-05	1.17E-05	2.34E-06
Cs-137	2.06E-11	2.06E-11	2.06E-11	8.55E-06	8.75E-06	8.85E-06	8.75E-06	1.75E-06
Cs-138	2.00E-09	2.00E-09	2.00E-09	8.99E-06	6.66E-06	4.88E-06	3.49E-06	3.66E-07
La-140	1.75E-13	1.75E-13	1.75E-13	6.43E-05	6.63E-05	6.72E-05	6.63E-05	1.33E-05
Ba-140	2.60E-13	2.60E-13	2.60E-13	1.02E-04	1.09E-04	1.09E-04	1.07E-04	2.14E-05
Total Particulates	2.31E-09	2.31E-09	2.31E-09	2.11E-04	2.15E-04	2.15E-04	2.11E-04	3.91E-05
TOTAL GROSS ACTIVITY	8.4E-07	8.4E-07	8.4E-07	1.52E+03	2.04E+03	2.55E+03	2.66E+03	2.11E+03

1993 Field Exercise Data

UNIT VENT DATA										
TIME	H+5:30	H+5:45	H+6:00	H+6:15	H+6:30	H+6:45	H+7:00	H+7:15	H+7:30	H+7:45
NUCLIDE (uCi/cc)										
Kr-85m	1.05E+02	1.07E+02	3.98E-07	3.83E-07	3.69E-07	3.56E-07	3.46E-07	3.30E-07	3.17E-07	3.02E-07
Kr-87	2.67E+01	2.46E+01	8.30E-08	7.25E-08	6.33E-08	5.52E-08	4.82E-08	4.21E-08	3.67E-08	3.35E-08
Kr-88	1.65E+02	1.64E+02	5.96E-07	5.61E-07	5.27E-07	4.96E-07	4.67E-07	4.39E-07	4.13E-07	4.00E-07
Xe-133	9.79E+02	1.03E+03	4.00E-06	4.00E-06	4.00E-06	4.00E-06	4.00E-06	4.00E-06	4.00E-06	4.00E-06
Xe-133m	2.59E+01	2.74E+01	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07
Xe-135	6.52E+02	6.75E+02	2.57E-06	2.52E-06	2.47E-06	2.43E-06	2.39E-06	2.34E-06	2.30E-06	2.25E-06
Xe-135m	3.49E-04	1.87E-04	3.68E-13	1.87E-13	9.49E-14	4.82E-14	2.44E-14	1.24E-14	6.30E-15	3.02E-15
Xe-138	3.73E-04	1.90E-04	3.53E-13	1.70E-13	8.19E-14	3.94E-14	1.90E-14	9.14E-15	4.40E-15	1.86E-15
TOTAL NOBLE GAS	1.95E+03	2.03E+03	7.75E-06	7.64E-06	7.54E-06	7.44E-06	7.36E-06	7.26E-06	7.17E-06	7.09E-06
I-131	5.11E-06	2.20E-06	1.05E-11	1.10E-11	1.00E-11	1.02E-11	1.05E-11	1.00E-11	9.89E-12	8.85E-12
I-132	1.72E-06	6.84E-07	3.02E-12	2.93E-12	2.47E-12	2.34E-12	2.23E-12	1.97E-12	1.75E-12	1.52E-12
I-133	9.94E-06	4.27E-06	2.02E-11	2.10E-11	1.90E-11	1.92E-11	1.96E-11	1.86E-11	1.82E-11	1.66E-11
I-134	2.52E-07	8.89E-08	3.48E-13	2.99E-13	2.23E-13	1.86E-13	1.57E-13	1.23E-13	1.10E-13	1.09E-13
I-135	6.82E-06	2.64E-06	1.23E-11	1.25E-11	1.11E-11	1.10E-11	1.11E-11	1.03E-11	9.69E-12	7.96E-12
TOTAL IODINE	2.38E-05	9.88E-06	4.64E-11	4.77E-11	4.28E-11	4.29E-11	4.36E-11	4.10E-11	3.96E-11	3.50E-11
DOSE EQUIVLENT IODINE	8.43E-06	3.60E-06	1.71E-11	1.78E-11	1.62E-11	1.64E-11	1.68E-11	1.60E-11	1.57E-11	1.41E-11
Cs-134	8.43E-07	4.84E-07	2.87E-11	2.93E-11	2.80E-11	2.83E-11	2.80E-11	2.81E-11	2.79E-11	2.65E-11
Cs-137	6.34E-07	3.62E-07	2.15E-11	2.20E-11	2.10E-11	2.12E-11	2.50E-11	2.10E-11	2.20E-11	1.15E-11
Cs-138	9.60E-08	3.97E-08	1.71E-12	1.26E-12	8.73E-13	6.38E-13	4.69E-13	3.30E-13	2.90E-13	2.85E-13
La-140	4.79E-06	2.74E-06	1.64E-10	1.69E-10	1.59E-10	1.61E-10	1.64E-10	1.59E-10	1.60E-10	1.55E-10
Ba-140	7.77E-06	4.47E-06	2.68E-10	2.77E-10	2.61E-10	2.65E-10	2.70E-10	2.63E-10	2.55E-10	2.45E-10
Total Particulates	1.41E-05	8.10E-06	4.84E-10	4.99E-10	4.70E-10	4.76E-10	4.87E-10	4.71E-10	4.65E-10	4.38E-10
GROSS TOTAL ACTIVITY	1.95E+03	2.03E+03	7.75E-06	7.64E-06	7.54E-06	7.44E-06	7.36E-06	7.26E-06	7.17E-06	7.09E-06

PROCESS MONITORS

AIRBORNE

Time-related implant airborne process monitoring data is provided in the following subsection. Concentration data is provided in the units as indicated. The process monitors are identified by identification numbers as well as its common name.

Some of the listed process monitors have three channels to monitor particulates, iodines or noble gases. For the process monitor data we have assumed that the particulate and noble gas channels determine gross activities whereas the iodine channel determines the activity of Iodine 131 and not gross iodine.

PROCESS MONITORS

AIRBORNE

Time Real Relative			0800 (H+00:00)	0815 (H+00:15)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	2.40E-09	2.30E-09
		I	1.00E-11	9.90E-12
		G	8.80E-07	8.40E-07
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	3.20E-10	3.20E-10
		I	1.40E-11	1.40E-11
		G	2.10E-07	2.10E-07
GGRE 28		P (uCi/cc)	3.10E-10	3.10E-10
		I	1.30E-11	1.30E-11
		G	2.00E-07	2.00E-07
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	8.00E-11	8.00E-11
		I	4.00E-12	4.00E-12
		G	6.60E-06	6.60E-06
GTRE 32		P (uCi/cc)	8.10E-11	8.10E-11
		I	4.00E-12	4.00E-12
		G	6.90E-06	6.90E-06
GTRE 59	Ctmt. High Range Rad.	(R/hr)	<1.00E+00	<1.00E+00
GTRE 60		(R/hr)	<1.00E+00	<1.00E+00

PROCESS MONITORS

AIRBORNE

Time	Real / Relative		0830 (H+00:30)	0845 (H+00:45)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	2.30E-09	1.60E-09
		I	1.00E-11	1.00E-11
		G	8.80E-07	7.40E-07
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	3.20E-10	3.20E-10
		I	1.40E-11	1.40E-11
		G	2.10E-07	2.10E-07
GGRE 28		P (uCi/cc)	3.10E-10	3.10E-10
		I	1.30E-11	1.30E-11
		G	2.00E-07	2.00E-07
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	8.00E-11	8.10E-11
		I	4.00E-12	1.10E-11
		G	6.60E-06	5.00E-03
GTRE 32		P (uCi/cc)	8.10E-11	8.1E-11
		I	4.00E-12	4.00E-11
		G	6.90E-06	5.00E-03
GTRE 59	Ctmt. High Range Rad.	(R/hr)	<1.00E+00	3.30E+00
GTRE 60		(R/hr)	<1.00E+00	3.40E+00

PROCESS MONITORS

AIRBORNE

Time	Real / Relative		0900 (H+01:00)	0915 (H+01:15)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	1.20E-09	9.30E-10
		I	1.00E-11	1.00E-11
		G	6.60E-07	6.20E-07
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	3.20E-10	3.20E-10
		I	1.40E-11	1.40E-11
		G	2.10E-07	2.10E-07
GGRE 28		P (uCi/cc)	3.10E-10	3.10E-10
		I	1.30E-11	1.30E-11
		G	2.00E-07	2.00E-07
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	5.00E-10	6.00E-10
		I	3.50E-11	4.00E-11
		G	6.20E-03	7.50E-03
GTRE 32		P (uCi/cc)	5.20E-10	6.20E-10
		I	3.40E-11	4.50E-11
		G	6.20E-03	7.60E-03
GTRE 59	Ctmt. High Range Rad.	(R/hr)	4.50E+00	4.80E+00
GTRE 60		(R/hr)	4.60E+00	4.90E+00

PROCESS MONITORS

AIRBORNE

Time Real Relative			0930 (H+01:30)	0945 (H+01:45)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	6.90E-10	4.70E-10
		I	1.10E-11	1.00E-11
		G	6.10E-07	5.80E-07
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	3.20E-10	3.20E-10
		I	1.40E-11	1.40E-11
		G	2.10E-07	2.10E-07
GGRE 28		P (uCi/cc)	3.10E-10	3.10E-10
		I	1.30E-11	1.30E-11
		G	2.00E-07	2.00E-07
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	7.00E-10	8.50E-10
		I	5.50E-11	6.60E-11
		G	8.70E-03	>1.00E+02
GTRE 32		P (uCi/cc)	7.10E-10	8.20E-10
		I	5.20E-11	6.50E-11
		G	8.80E-03	>1.00E+02
GTRE 59	Ctmt. High Range Rad.	(R/hr)	5.50E+00	9.00E+00
GTRE 60		(R/hr)	5.60E+00	9.10E+00

PROCESS MONITORS

AIRBORNE

Time Real / Relative			1000 (H+02:00)	1015 (H+02:15)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	3.80E-10	4.30E-10
		I	9.90E-12	1.30E-11
		G	5.30E-07	5.80E-07
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	3.20E-10	3.20E-10
		I	1.40E-11	1.40E-11
		G	2.10E-07	2.10E-07
GGRE 28		P (uCi/cc)	3.10E-10	3.10E-10
		I	1.30E-11	1.30E-11
		G	2.00E-07	2.00E-07
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	2.60E-08	OFF SCALE HIGH
		I	3.00E-09	
		G	>1.00E-02	
GTRE 32		P (uCi/cc)	2.40E-08	OFF SCALE HIGH
		I	2.50E-09	
		G	>1.00E-02	
GTRE 59	Ctmt. High Range Rad.	(R/hr)	1.30E+03	1.00E+05
GTRE 60		(R/hr)	1.30E+03	1.00E+05

PROCESS MONITORS

AIRBORNE

Time	Real / Relative		1030 (H+02:30)	1045 (H+02:45)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	4.50E-10	4.10E-10
		I	1.30E-11	1.30E-11
		G	5.60E-07	5.50E-07
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	3.20E-10	3.20E-10
		I	1.40E-11	1.40E-11
		G	2.10E-07	2.10E-07
GGRE 28		P (uCi/cc)	3.10E-10	3.10E-10
		I	1.30E-11	1.30E-11
		G	2.00E-07	2.00E-07
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 32		P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 59	Ctmt. High Range Rad.	(R/hr)	9.40E+05	8.50E+05
GTRE 60		(R/hr)	9.40E+05	8.50E+05

PROCESS MONITORS

AIRBORNE

Time	Real Relative		1100 (H+03:00)	1115 (H+03:15)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	5.10E-10	2.5E-09
		I	1.40E-11	1.0E-11
		G	5.60E-07	8.5E-07
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	3.20E-10	3.20E-10
		I	1.40E-11	1.40E-11
		G	2.10E-07	2.10E-07
GGRE 28		P (uCi/cc)	3.10E-10	3.10E-10
		I	1.30E-11	1.30E-11
		G	2.00E-07	2.00E-07
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 32		P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 59	Ctmt. High Range Rad.	(R/hr)	7.90E+05	7.40E+05
GTRE 60		(R/hr)	7.90E+05	7.40E+05

PROCESS MONITORS

AIRBORNE

Time Real Relative			1130 (H+03:30)	1145 (H+03:45)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	2.5E-09	2.5E-09
		I	1.0E-11	1.00E-11
		G	8.5E-07	5.80E-07
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	3.20E-10	3.20E-10
		I	1.40E-11	1.40E-11
		G	2.10E-07	2.10E-07
GGRE 28		P (uCi/cc)	3.10E-10	3.10E-10
		I	1.30E-11	1.30E-11
		G	2.00E-07	2.00E-07
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 32		P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 59	Ctmt. High Range Rad.	(R/hr)	6.90E+05	6.50E+05
GTRE 60		(R/hr)	6.90E+05	6.50E+05

PROCESS MONITORS

AIRBORNE

Time Real Relative			1200 (H+04:00)	1215 (H+04:15)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	2.5E-09	>1.0E-7
		I	1.0E-11	>1.0E-6
		G	8.5E-07	2.06E+03
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	>1.00E-07	>1.00E-07
		I	>1.00E-06	>1.00E-06
		G	>1.00E-02	>1.00E-02
GGRE 28		P (uCi/cc)	>1.00E-07	>1.00E-07
		I	>1.00E-06	>1.00E-06
		G	>1.00E-02	>1.00E-02
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 32		P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 59	Ctmt. High Range Rad.	(R/hr)	6.20E+05	5.80E+05
GTRE 60		(R/hr)	6.20E+05	5.80E+05

PROCESS MONITORS

AIRBORNE

Time Real Relative			1230 (H+04:30)	1245 (H+04:45)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	>1.00E-07	>1.00E-07
		I	>1.00E-06	>1.00E-06
		G	2.44E+03	2.58E+03
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	>1.00E-07	>1.00E-07
		I	>1.00E-06	>1.00E-06
		G	>1.00E-02	>1.00E-02
GGRE 28		P (uCi/cc)	>1.00E-07	>1.00E-07
		I	>1.00E-06	>1.00E-06
		G	>1.00E-02	>1.00E-02
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 32		P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 59	Ctmt. High Range Rad.	(R/hr)	5.50E+05	5.30E+05
GTRE 60		(R/hr)	5.50E+05	5.30E+05

PROCESS MONITORS

AIRBORNE

Time Real Relative			1300 (H+05:00)	1315 (H+05:15)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	>1.00E-07	>1.00E-07
		I	>1.00E-06	1.00E-06
		G	2.29E+03	2.62E+03
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	[illegible].00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	>1.00E-07	>1.00E-07
		I	>1.00E-06	>1.00E-06
		G	>1.00E-02	>1.00E-02
GGRE 28		P (uCi/cc)	>1.00E-07	>1.00E-07
		I	>1.00E-06	>1.00E-06
		G	>1.00E-02	>1.00E-02
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 32		P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 59	Ctmt. High Range Rad.	(R/hr)	5.00E+05	4.70E+05
GTRE 60		(R/hr)	5.00E+05	4.70E+05

PROCESS MONITORS

AIRBORNE

Time Real Relative			1330 (H+05:30)	1345 (H+05:45)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	>1.0E-07	>1.0E-07
		I	>1.0E-06	>1.0E-06
		G	1.95E+03	2.05E+03
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	>1.00E-07	>1.00E-07
		I	>1.00E-06	>1.00E-06
		G	>1.00E-02	>1.00E-02
GGRE 28		P (uCi/cc)	>1.00E-07	>1.00E-07
		I	>1.00E-06	>1.00E-06
		G	>1.00E-02	>1.00E-02
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 32		P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 59	Ctmt. High Range Rad.	(R/hr)	4.50E+05	4.30E+05
GTRE 60		(R/hr)	4.50E+05	4.30E+05

PROCESS MONITORS

AIRBORNE

Time	Real Relative		1400 (H+06:00)	1415 (H+06:15)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	5.15E-10	5.15E-10
		I	1.00E-11	1.00E-11
		G	7.80E-06	7.80E-06
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	2.40E-08	2.40E-08
		I	1.00E-08	1.00E-08
		G	1.60E-04	1.60E-04
GGRE 28		P (uCi/cc)	2.40E-08	2.40E-08
		I	1.00E-08	1.00E-08
		G	1.60E-04	1.60E-04
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 32		P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 59	Ctmt. High Range Rad.	(R/hr)	4.20E+05	4.20E+05
GTRE 60		(R/hr)	4.20E+05	4.20E+05

PROCESS MONITORS

AIRBORNE

Time Real Relative			1430 (H+06:30)	1445 (H+06:45)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	5.30E-10	5.30E-10
		I	1.10E-11	1.10E-11
		G	7.80E-06	7.80E-06
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	2.40E-08	2.40E-08
		I	1.00E-08	1.00E-08
		G	1.60E-05	1.60E-05
GGRE 28		P (uCi/cc)	2.40E-08	2.40E-08
		I	1.00E-09	1.00E-09
		G	1.60E-05	1.60E-05
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	OFF SCALE HIGH	OFF SCALE HIGH
		I		
		G		
GTRE 32		P (uCi/cc)	OFF SCALE HIGH	OFF SCALE HIGH
		I		
		G		
GTRE 59	Ctmt. High Range Rad.	(R/hr)	4.10E+05	4.10E+05
GTRE 60		(R/hr)	4.10E+05	4.10E+05

PROCESS MONITORS

AIRBORNE

Time Real Relative			1500 (H+07:00)	1515 (H+07:15)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	5.00E-10	5.10E-10
		I	1.00E-11	1.00E-11
		G	7.60E-06	7.50E-06
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	2.40E-10	2.40E-10
		I	1.00E-11	1.00E-11
		G	4.00E-06	4.00E-06
GGRE 28		P (uCi/cc)	2.40E-10	2.40E-10
		I	1.00E-11	1.00E-11
		G	4.00E-06	4.00E-06
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 32		P (uCi/cc)	OFF SCALE	OFF SCALE
		I	HIGH	HIGH
		G		
GTRE 59	Ctmt. High Range Rad.	(R/hr)	4.00E+05	3.90E+05
GTRE 60		(R/hr)	4.00E+05	3.90E+05

PROCESS MONITORS

AIRBORNE

Time Real Relative			1530 (H+07:30)	1545 (H+07:45)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	5.10E-10	5.15E-10
		I	1.00E-11	1.05E-11
		G	7.50E-06	7.40E-06
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-11	1.00E-11
		G	2.00E-06	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	2.40E-10	2.40E-10
		I	1.00E-11	1.00E-11
		G	4.00E-06	4.00E-06
GGRE 28		P (uCi/cc)	2.40E-10	2.40E-10
		I	1.00E-11	1.00E-11
		G	4.00E-06	4.00E-06
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12	1.00E-12
		I	1.00E-12	1.00E-12
		G	1.00E-07	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-11	2.00E-11
		G	4.00E-06	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12	3.00E-12
		I	2.00E-12	2.00E-12
		G	4.00E-06	4.00E-06
G'RE 31	Containment Atmosphere	P (uCi/cc)	OFF SCALE HIGH	OFF SCALE HIGH
		I		
		G		
GTRE 32		P (uCi/cc)	OFF SCALE HIGH	OFF SCALE HIGH
		I		
		G		
GTRE 59	Ctmt. High Range Rad.	(R/hr)	3.90E+05	3.80E+05
GTRE 60		(R/hr)	3.90E+05	3.80E+05

PROCESS MONITORS

AIRBORNE

Time	Real / Relative		1600 (H+08:00)
GTRE 21A/B	Unit Vent Eff.	P (uCi/cc)	5.15E-10
		I	1.05E-11
		G	7.40E-06
GHRE 10A/B	Radwaste Building Eff.	P (uCi/cc)	1.00E-12
		I	1.00E-11
		G	2.00E-06
ABRE 114	MSRV Monitor S/G A	(mR/hr)	4.00E-01
ABRE 113	S/G B	(mR/hr)	4.00E-01
ABRE 112	S/G C	(mR/hr)	4.00E-01
ABRE 111	S/G D	(mR/hr)	4.00E-01
FCRE 385	Aux. Feed Pump Turbine	(mR/hr)	4.00E-01
GLRE 60	Aux. Bldg. Vent Ex.	P (uCi/cc)	2.00E-11
GKRE 41	Access Control Vent	P (uCi/cc)	3.00E-11
GGRE 27	Fuel Bldg. Vent Ex.	P (uCi/cc)	2.40E-10
		I	1.00E-11
		G	4.00E-06
GGRE 28		P (uCi/cc)	2.40E-10
		I	1.00E-11
		G	4.00E-06
GERE 92	Condenser Air Discharge	G (uCi/cc)	2.00E-06
GTRE 22	Containment Purge Ex.	P (uCi/cc)	1.00E-12
		I	1.00E-12
		G	1.00E-07
GTRE 33		P (uCi/cc)	1.00E-12
		I	1.00E-12
		G	1.00E-07
GHRE 22	Radwaste Bldg. Vent	P (uCi/cc)	1.00E-11
GHRE 23	Gas Decay Tank Vent Ex.	G (uCi/cc)	2.20E-06
GKRE 04	Control Room Supply	P (uCi/cc)	3.00E-12
		I	2.00E-11
		G	4.00E-06
GKPE 05		P (uCi/cc)	3.00E-12
		I	2.00E-12
		G	4.00E-06
GTRE 31	Containment Atmosphere	P (uCi/cc)	OFF SCALE HIGH
		I	
		G	
GTRE 32		P (uCi/cc)	OFF SCALE HIGH
		I	
		G	
GTRE 59	Ctmt. High Range Rad.	(R/hr)	3.80E+05
GTRE 60		(R/hr)	3.80E+05

PROCESS MONITORS

LIQUID

Time-related inplant liquid process monitoring data is provided in the following subsection. Concentration data is provided in the units as indicated. The process monitors are identified by identification numbers as well as its common name.

PROCESS MONITORS

LIQUID

ID Number	Time: Real Relative		0800-1600 (H+0:00-H+08:00)
LFRE 59	Turbine Bldg. Drain	(uCi/cc)	6.00E-07
HFRE 45	Sec. Liquid Waste Discharge	(uCi/cc)	5.50E-09
FBRE 50	Aux. Steam Cond. Recovery Tank	(uCi/cc)	4.00E-07
AERE 4A	Service Water Return	(uCi/cc)	4.00E-08
AERE 4B		(uCi/cc)	4.00E-08
EFRE 35	Essential Water Service	(uCi/cc)	5.00E-07
EFRE 36		(uCi/cc)	5.00E-07
EGRE 09	CCW Train A	(uCi/cc)	4.50E-07
EGRE 10	CCW Train B	(uCi/cc)	4.50E-07
SJRE 02	S/G Liquid	(uCi/cc)	3.00E-07
BMRE 25	S/G Blowdown Process	(uCi/cc)	3.50E-07
BMRE 52	S/G Blowdown Discharge	(uCi/cc)	3.50E-07
SJRE 01	CVCS Letdown	(uCi/cc)	7.40E-02
HERE 16	Boron Recycle Distillate	(uCi/cc)	5.00E-07
HBRE 18	Liquid Waste Discharge	(uCi/cc)	3.00E-05

AREA RADIATION MONITORING

Time-related inplant area radiation monitoring data is provided in the following subsection. Radiation data is provided in the units as indicated. The area radiation montiors are identified by identification numbers as well as its common name.

Radiation levels indicated with a < or > sign indicate the monitor is reading offscale low or offscale high respectively.

The location of area radiation monitors within the auxiliary and fuel buildings are designated on the inplant radiation maps.

93-FE

7.35

MONITOR	LOCATION	TIME 0800 - 0845 DOSE RATE (mR/hR)	TIME 0900	TIME 0915	TIME 0930	TIME 0945	TIME 1000	TIME 1015	TIME 1030	TIME 1045 - 1145	TIME 1200
SDRE01	RADWASTE BLDG. CORRIDOR, BASEMENT (W)	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE02	RADWASTE BLDG. CORRIDOR, BASEMENT (Cntl)	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE03	RADWASTE BLDG. CORRIDOR, BASEMENT (E)	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE04	RADWASTE BLDG. CORRIDOR, 2000' (W)	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE05	RADWASTE BLDG. CORRIDOR, 2000' (Cntl)	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE06	SOLID RADWASTE AREA, 2000'	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE07	RADWASTE BLDG. TRUCK SPACE, 2000'	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE08	RADWASTE BLDG. SAMPLE LABORATORY	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE09	RADWASTE BLDG. VALVE ROOM CORRIDOR, 2047' (E)	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE10	RADWASTE BLDG. VALVE ROOM CORRIDOR, 2047' (W)	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE11	RADWASTE BLDG. HVAC FILTER UNIT	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE12	AUX. BLDG. CORRIDOR, BASEMENT 1974' (SE)	0.5	0.5	0.5	0.5	0.5	0.5	100.0	700.0	1200.0	1200.0
SDRE13	AUX. BLDG. CORRIDOR, BASEMENT 1974' (NE)	0.5	0.5	0.5	0.5	0.5	0.5	100.0	700.0	1200.0	1200.0
SDRE14	AUX. BLDG. CORRIDOR, BASEMENT 1974' (N)	0.5	0.5	0.5	0.5	0.5	0.5	50.0	300.0	600.0	600.0
SDRE15	AUX. BLDG. CORRIDOR, BASEMENT 1974' (W)	0.5	0.5	0.5	0.5	0.5	0.5	50.0	300.0	600.0	600.0
SDRE16	AUX. BLDG. CORRIDOR, BASEMENT 1974' (SW)	0.5	0.5	0.5	0.5	0.5	0.5	50.0	300.0	600.0	600.0
SDRE17	NON-RADIOACTIVE PIPE TUNNEL & PERSONNEL ACCESS 1974'	0.5	0.5	0.5	0.5	0.5	0.5	10.0	60.0	100.0	100.0
SDRE18	AUX. BLDG. GROUND FLOOR CORRIDOR 2000' (N)	1.0	1.0	1.0	1.0	1.0	1.0	100.0	700.0	1200.0	1200.0
SDRE19	AUX. BLDG. GROUND FLOOR CORRIDOR 2000' (SE)	0.5	0.5	0.5	0.5	0.5	0.5	75.0	500.0	900.0	900.0
SDRE20	AUX. BLDG. VALVE ROOM CORRIDOR 2000' (S)	5.0	5.0	5.0	5.0	5.0	5.0	50.0	300.0	500.0	500.0
SDRE21	AUX. BLDG. VALVE ROOM CORRIDOR 2000' (S)	10.0	10.0	10.0	10.0	10.0	10.0	50.0	300.0	500.0	500.0
SDRE22	AUX. BLDG. GROUND FLOOR 2000' (SW)	0.5	0.5	0.5	0.5	0.5	0.5	50.0	300.0	500.0	500.0
SDRE23	AUX. BLDG. GROUND FLOOR 2000' (W)	0.5	0.5	0.5	0.5	0.5	0.5	50.0	300.0	500.0	500.0
SDRE24	AUX. BLDG. SAMPLING ROOM 2000' (Cntl)	5.0	5.0	5.0	5.0	5.0	5.0	50.0	300.0	500.0	500.0
SDRE25	AUX. BLDG. VENT FILTER	0.5	0.5	0.5	0.5	0.5	0.5	100.0	700.0	1200.0	> 10000
SDRE26	RHR HEAT EXCHANGER OUTSIDE 2026'	0.5	0.5	0.5	0.5	0.5	0.5	75.0	400.0	700.0	700.0
SDRE27	CONT. PURGE EXHAUST FILTER UNIT 2047'	0.5	0.5	0.5	0.5	0.5	0.5	100.0	700.0	1200.0	> 10000
SDRE28	CONT. PERSONNEL HATCH 2047'	0.5	0.5	0.5	0.5	0.5	0.5	125.0	800.0	1400.0	> 10000
SDRE29	HOT MACHINE SHOP	0.2	0.2	0.2	0.2	0.2	0.2	0.5	0.5	0.5	0.5
SDRE30	HOT INSTRUMENT SHOP	0.2	0.2	0.2	0.2	0.2	0.2	0.5	0.5	0.5	0.5
SDRE31	CONTROL BLDG. HOT LAB	0.2	0.2	0.2	0.2	0.2	0.2	0.5	0.5	0.5	0.5
SDRE32	CONTROL BLDG. CORRIDOR	0.2	0.2	0.2	0.2	0.2	0.2	0.5	0.5	0.5	0.5
SDRE33	CONTROL ROOM 2047'	0.2	0.2	0.2	0.2	0.2	0.2	0.5	0.5	0.5	0.5
SDRE34	CASK HANDLING AREA 2000'	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5
SDRE35	NEW FUEL STORAGE 2026'	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5	20.0
SDRE36	NEW FUEL STORAGE CORRIDOR 2026'	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5	20.0
SDRE37	SPENT FUEL POOL AREA	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5
SDRE38	SPENT FUEL POOL AREA	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5	0.5
SDRE39	SEAL TABLE AREA 2026' (N)	30.0	3000	3900	4100	5100	8000	>10000	>10000	>10000	>10000
SDRE40	PERSONNEL ACCESS HATCH AREA INSIDE 2047' (SW)	15.0	3200	4100	4500	5300	9500	>10000	>10000	>10000	>10000
SDRE41	CONTAINMENT BLDG. 2047' (NW)	25.0	4100	5300	5500	5700	9700	>10000	>10000	>10000	>10000
SDRE42	CONTAINMENT BLDG. 2047' (E)	20.0	4150	5400	5600	5800	>10000	>10000	>10000	>10000	>10000
SDRE43	TECHNICAL SUPPORT CENTER	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE44	EMERGENCY OPERATIONS FACILITY	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2	0.2
SDRE47	PASS SAMPLING ROOM 2000'	10.0	10	10	10	10	10	400	400	400	400

93-FE

MONITOR	LOCATION	TIME 1215	TIME 1230	TIME 1245	TIME 1300	TIME 1315 - 1330	TIME 1345 - 1600
SDRE01	RADWASTE BLDG. CORRIDOR, BASEMENT (W)	0.2	0.2	0.2	0.2	0.2	10.0
SDRE02	RADWASTE BLDG. CORRIDOR, BASEMENT (Cntl)	0.2	0.2	0.2	0.2	0.2	7.0
SDRE03	RADWASTE BLDG. CORRIDOR, BASEMENT (E)	0.2	0.2	0.2	0.2	0.2	2.0
SDRE04	RADWASTE BLDG. CORRIDOR, 2000' (W)	0.2	0.2	0.2	0.2	0.2	10.0
SDRE05	RADWASTE BLDG. CORRIDOR, 2000' (Cntl)	0.2	0.2	0.2	0.2	0.2	7.0
SDRE06	SOLID RADWASTE AREA, 2000'	0.2	0.2	0.2	0.2	0.2	0.2
SDRE07	RADWASTE BLDG. TRUCK SPACE, 2000'	0.2	0.2	0.2	0.2	0.2	0.2
SDRE08	RADWASTE BLDG. SAMPLE LABORATORY	0.2	0.2	0.2	0.2	0.2	7.0
SDRE09	RADWASTE BLDG. VALVE ROOM CORRIDOR, 2047' (E)	0.2	0.2	0.2	0.2	0.2	0.2
SDRE10	RADWASTE BLDG. VALVE ROOM CORRIDOR, 2047' (W)	0.2	0.2	0.2	0.2	0.2	0.2
SDRE11	RADWASTE BLDG. HVAC FILTER UNIT	0.2	0.2	0.2	0.2	0.2	0.2
SDRE12	AUX. BLDG. CORRIDOR, BASEMENT 1974' (SE)	1200.0	1200.0	1200.0	1200.0	1200.0	>10000
SDRE13	AUX. BLDG. CORRIDOR, BASEMENT 1974' (NE)	1200.0	1200.0	1200.0	1200.0	1200.0	>10000
SDRE14	AUX. BLDG. CORRIDOR, BASEMENT 1974' (N)	600.0	600.0	600.0	600.0	600.0	>10000
SDRE15	AUX. BLDG. CORRIDOR, BASEMENT 1974' (W)	600.0	600.0	600.0	600.0	600.0	>10000
SDRE16	AUX. BLDG. CORRIDOR, BASEMENT 1974' (SW)	600.0	600.0	600.0	600.0	600.0	>10000
SDRE17	NON-RADIOACTIVE PIPE TUNNEL & PERSONNEL ACCESS 1974'	100.0	100.0	100.0	100.0	150.0	>10000
SDRE18	AUX. BLDG. GROUND FLOOR CORRIDOR 2000' (N)	1200.0	1200.0	1200.0	1200.0	1200.0	>10000
SDRE19	AUX. BLDG. GROUND FLOOR CORRIDOR 2000' (SE)	900.0	900.0	900.0	900.0	900.0	>10000
SDRE20	AUX. BLDG. VALVE ROOM CORRIDOR 2000' (S)	500.0	500.0	500.0	500.0	500.0	>10000
SDRE21	AUX. BLDG. VALVE ROOM CORRIDOR 2000' (S)	500.0	500.0	500.0	500.0	500.0	>10000
SDRE22	AUX. BLDG. GROUND FLOOR 2000' (SW)	500.0	500.0	500.0	500.0	500.0	>10000
SDRE23	AUX. BLDG. GROUND FLOOR 2000' (W)	500.0	500.0	500.0	500.0	500.0	>10000
SDRE24	AUX. BLDG. SAMPLING ROOM 2000' (Cntl)	500.0	500.0	500.0	500.0	500.0	>10000
SDRE25	AUX. BLDG. VENT FILTER	> 10000	> 10000	> 10000	> 10000	> 10000	>10000
SDRE26	RHR HEAT EXCHANGER OUTSIDE 2026'	700.0	700.0	700.0	700.0	700.0	>10000
SDRE27	CONT. PURGE EXHAUST FILTER UNIT 2047'	> 10000	> 10000	> 10000	> 10000	> 10000	>10000
SDRE28	CONT. PERSONNEL HATCH 2047'	> 10000	> 10000	> 10000	> 10000	> 10000	>10000
SDRE29	HOT MACHINE SHOP	1.0	1.0	1.0	1.0	1.0	1.0
SDRE30	HOT INSTRUMENT SHOP	1.0	1.0	1.0	1.0	1.0	1.0
SDRE31	CONTROL BLDG. HOT LAB	1.0	1.0	1.0	1.0	1.0	1.0
SDRE32	CONTROL BLDG. CORRIDOR	1.0	1.0	1.0	1.0	1.0	1.0
SDRE33	CONTROL ROOM 2047'	1.0	1.0	1.0	1.0	1.0	1.0
SDRE34	CASK HANDLING AREA 2000'	0.5	0.5	0.5	0.5	0.5	0.5
SDRE35	NEW FUEL STORAGE 2026'	100.0	800.0	4000.0	9000.0	9000.0	9000.0
SDRE36	NEW FUEL STORAGE CORRIDOR 2026'	100.0	800.0	4000.0	9000.0	9000.0	9000.0
SDRE37	SPENT FUEL POOL AREA	1000.0	4000.0	5000.0	7000.0	9000.0	9000.0
SDRE38	SPENT FUEL POOL AREA	> 10000	> 10000	> 10000	> 10000	> 10000	> 10000
SDRE39	SEAL TABLE AREA 2026' (N)	>10000	>10000	>10000	>10000	>10000	>10000
SDRE40	PERSONNEL ACCESS HATCH AREA INSIDE 2047' (SW)	>10000	>10000	>10000	>10000	>10000	>10000
SDRE41	CONTAINMENT BLDG. 2047' (NW)	>10000	>10000	>10000	>10000	>10000	>10000
SDRE42	CONTAINMENT BLDG. 2047' (E)	>10000	>10000	>10000	>10000	>10000	>10000
SDRE43	TECHNICAL SUPPORT CENTER	0.2	0.2	0.2	0.2	0.2	0.2
SDRE44	EMERGENCY OPERATIONS FACILITY	0.2	0.2	0.2	0.2	0.2	0.2
SDRE47	PASS SAMPLING ROOM 2000'	400	400	400	400	400	400

7. 36

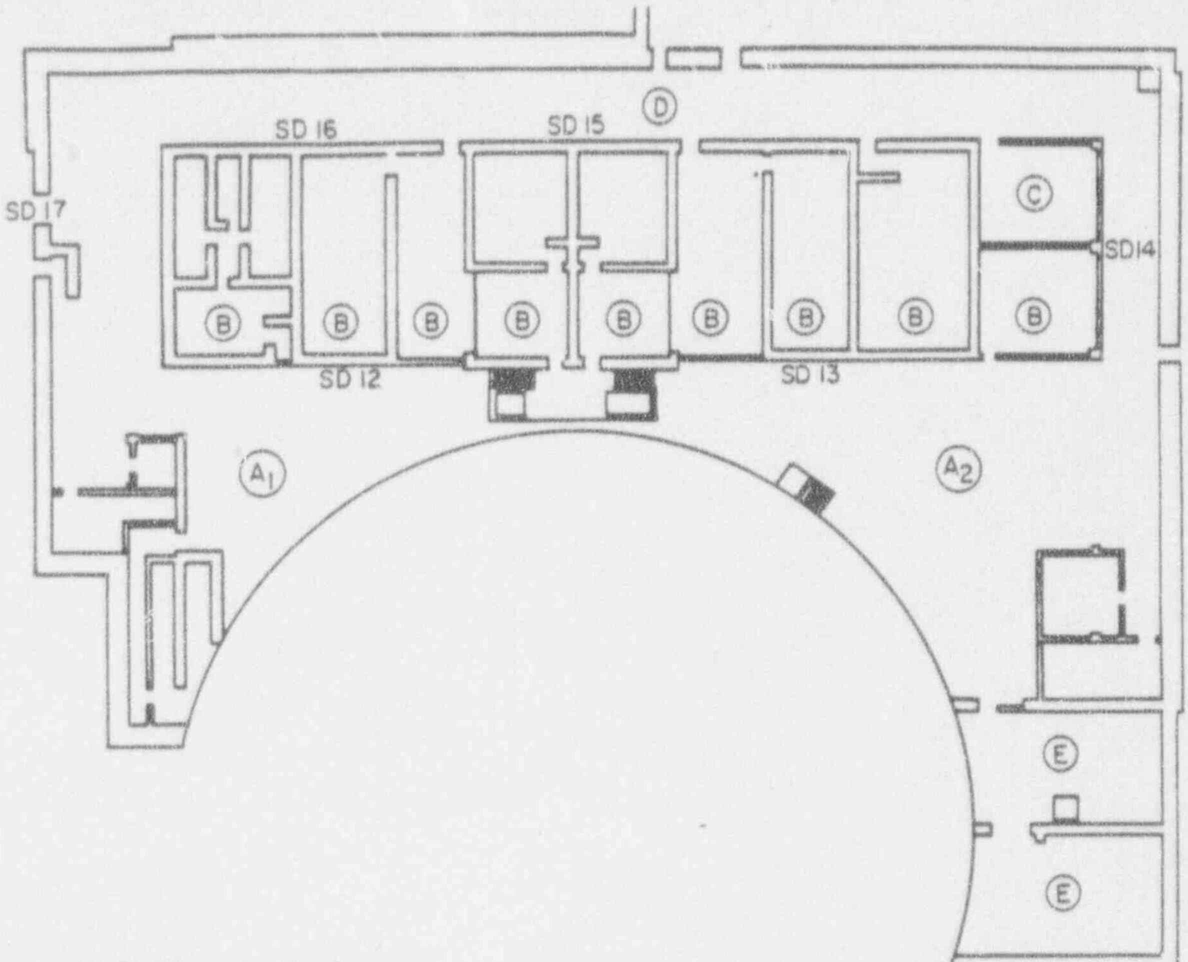
INPLANT SURVEYS

RADIATION

Time-related inplant survey map data is provided i the following subsection. Data is provided for each floor leve. .. the auxiliary, fuel and diesel generator buildings. Radiation data is provided in the units as indicated. The data is designated by a letter and corresponds to the circled letter zones on the map.

Radiation levels indicated with a < sign indicate areas where readings will generally be below the lower level of detectabiity for instruments used in determining radiation levels.

Area radiation monitors are designated on the maps as the monitor number preceeded by a "SD". The data for these monitors is provided in the area radiation monitoring data.



AUX. BLDG. 1974'-0"

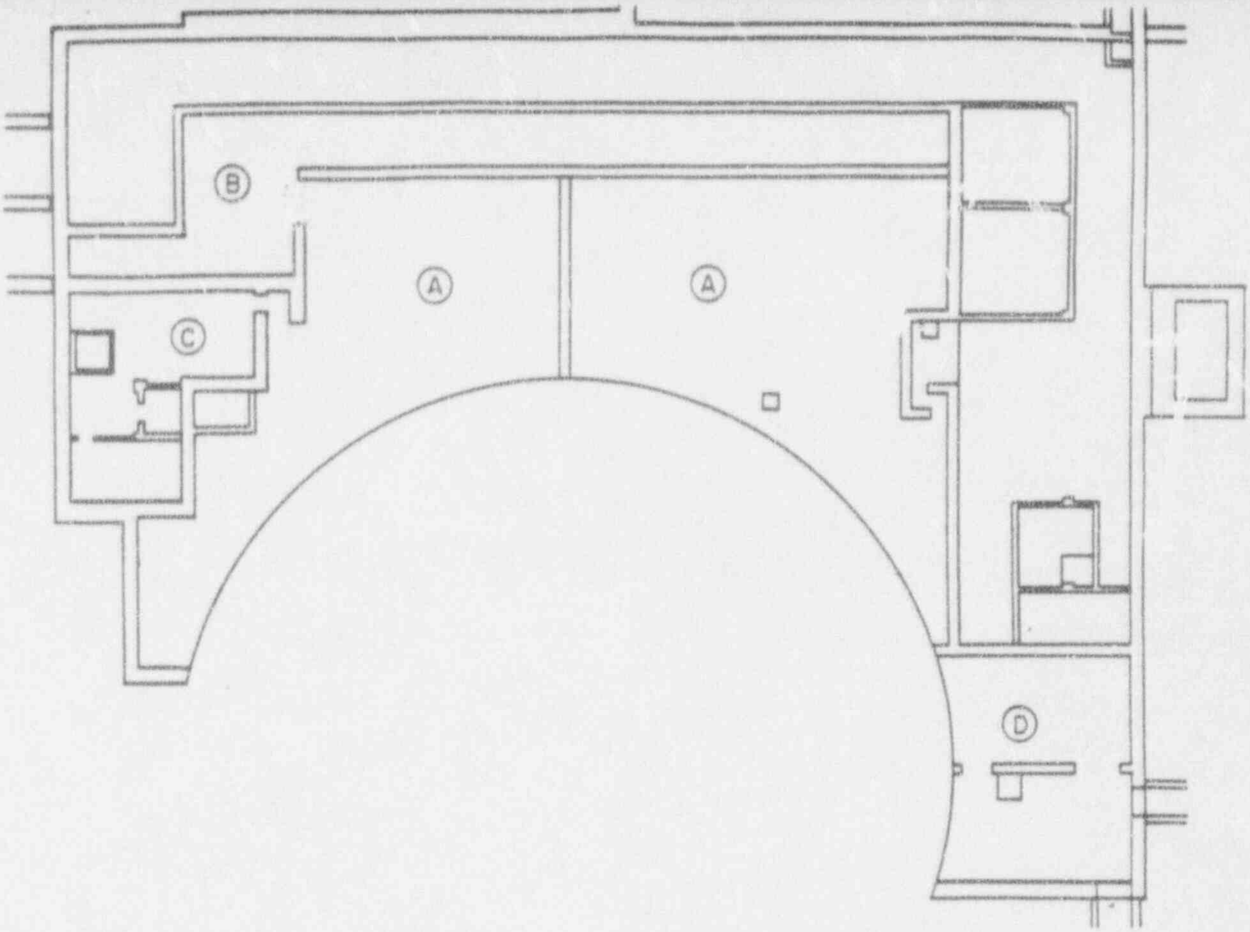
(RADIATION LEVELS INDICATED ARE IN R/HR)

Time	Real Relative	0800 (H+00:00)	0815 (H+00:15)	0830 (H+00:30)	0845 (H+00:45)	0900 (H+01:00)	0915 (H+01:15)	0930 (H+01:30)	0945 (H+01:45)	100 (H+02:0
(A)	Corridor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.00
(B)	Pump Room Areas	0.020	0.020	0.020	0.020	0.020	0.020	0.020	0.020	0.02
(C)	Boric Acid Tank Areas	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.00
(D)	Corridor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.00
(E)	Aux FW Pump Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.00

Time	Real Relative	1015 (H+02:15)	1030 (H+02:30)	1045 (H+02:45)	1100 (H+03:00)	1115 (H+03:15)	1130 (H+03:30)	1145 (H+03:45)	1200 (H+04:00)	121 (H+04:1
(A)	Corridor Area	0.100	0.100	0.100	0.700	1.200	1.200	1.200	1.200	1.20
(B)	Pump Room Areas	0.050	0.050	0.050	0.100	0.150	0.150	0.150	0.150	0.15
(C)	Boric Acid Tank Areas	0.050	0.050	0.050	0.075	0.100	0.100	0.100	0.100	0.10
(D)	Corridor Area	0.050	0.050	0.050	0.300	0.600	0.600	0.600	0.600	0.60
(E)	Aux FW Pump Area	0.100	0.100	0.100	0.700	1.200	1.200	1.200	1.200	1.20

Time	Real Relative	1230 (H+04:30)	1245 (H+04:45)	1300 (H+05:00)	1315 (H+05:15)	1330 (H+05:30)	1345 (H+05:45)	1400 (H+06:00)	1415 (H+06:15)	143 (H+06:3
(A)	Corridor Area	1.200	1.200	1.200	1.200	1.200	200.000	200.000	200.000	200.00
(B)	Pump Room Areas	0.150	0.150	0.150	0.150	0.150	1200.000	1200.000	1200.000	1200.00
(C)	Boric Acid Tank Areas	0.100	0.100	0.100	0.100	0.100	2.200	2.200	2.200	2.20
(D)	Corridor Area	0.600	0.600	0.600	0.600	0.600	200.000	200.000	200.000	200.00
(E)	Aux FW Pump Area	1.200	1.200	1.200	1.200	1.200	200.000	200.000	200.000	200.00

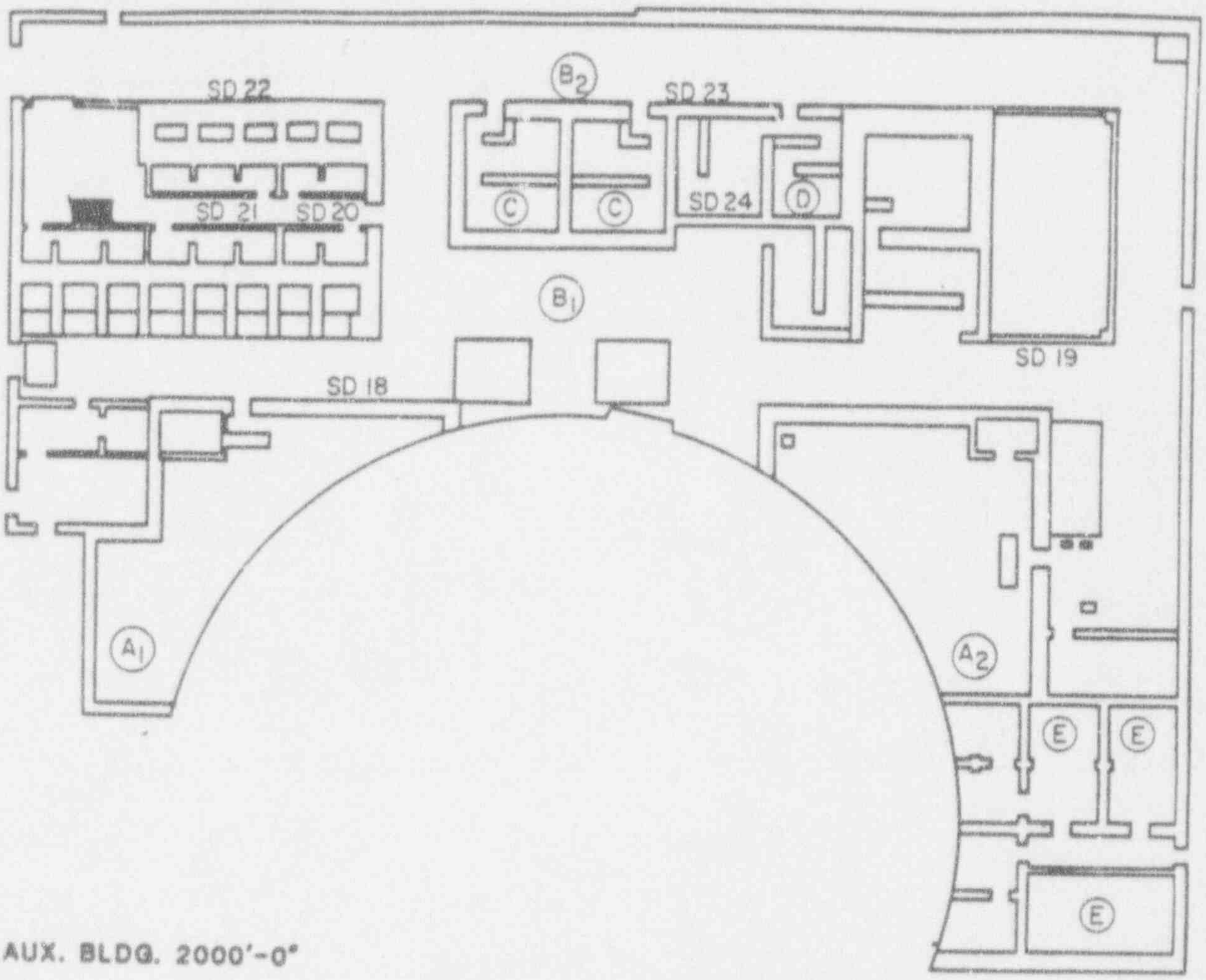
Time	Real Relative	1445 (H+06:45)	1500 (H+07:00)	1515 (H+07:15)	1530 (H+07:30)	1545 (H+07:45)	1600 (H+08:00)
(A)	Corridor Area	200.000	200.000	200.000	200.000	200.000	200.000
(B)	Pump Room Areas	1200.000	1200.000	1200.000	1200.000	1200.000	1200.000
(C)	Boric Acid Tank Areas	2.200	2.200	2.200	2.200	2.200	2.200
(D)	Corridor Area	200.000	200.000	200.000	200.000	200.000	200.000
(E)	Aux FW Pump Area	200.000	200.000	200.000	200.000	200.000	200.000



AUX. BLDG. 1988'-0" and 1989'-6"

(RADIATION LEVELS INDICATED ARE IN R/HR)

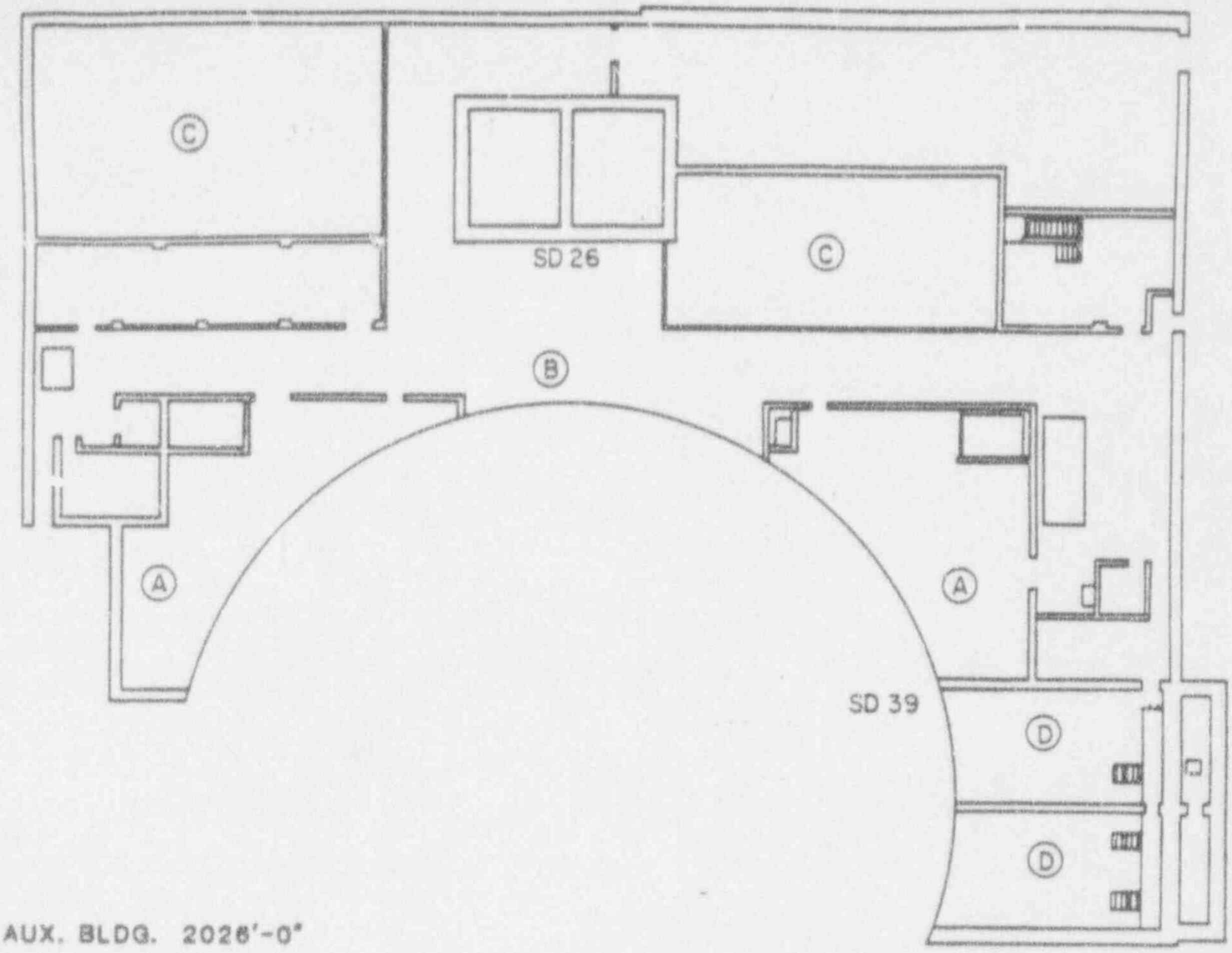
Time		Real Relative	0800 (E+00:00)	0815 (E+00:15)	0830 (E+00:30)	0845 (E+00:45)	0900 (E+01:00)	0915 (E+01:15)	0930 (E+01:30)	0945 (E+01:45)	1000 (E+02:00)
(A)	RHR Encapsulation Areas		0.015	0.015	0.015	0.015	0.015	0.015	0.015	0.015	0.015
(B)	Corridor Area		< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(C)	Corridor Area		< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
Time		Real Relative	1015 (E+02:15)	1030 (E+02:30)	1045 (E+02:45)	1100 (E+03:00)	1115 (E+03:15)	1130 (E+03:30)	[illegible] (E+0.	1200 +04:00)	1215 (E+04:15)
(A)	RHR Encapsulation Areas		0.100	0.100	0.100	0.700	1.200	1.200	1.200	1.200	1.200
(B)	Corridor Area		0.010	0.010	0.010	0.070	0.120	0.120	0.120	0.120	0.120
(C)	Corridor Area		0.005	0.005	0.005	0.035	0.600	0.600	0.600	0.600	0.600
Time		Real Relative	1230 (E+04:30)	1245 (E+04:45)	1300 (E+05:00)	1315 (E+05:15)	1330 (E+05:30)	1345 (E+05:45)	1400 (E+06:00)	1415 (E+06:15)	1430 (E+06:30)
(A)	RHR Encapsulation Areas		1.200	1.200	1.200	1.200	1.200	800.000	800.000	800.000	800.000
(B)	Corridor Area		0.120	0.120	0.120	0.120	0.120	50.000	50.000	50.000	50.000
(C)	Corridor Area		0.600	0.600	0.600	0.600	0.600	15.000	15.000	15.000	15.000
Time		Real Relative	1445 (E+06:45)	1500 (E+07:00)	1515 (E+07:15)	1530 (E+07:30)	1545 (E+07:45)	1600 (E+08:00)			
(A)	RHR Encapsulation Areas		800.000	800.000	800.000	800.000	800.000	800.000			
(B)	Corridor Area		50.000	50.000	50.000	50.000	50.000	50.000			
(C)	Corridor Area		15.000	15.000	15.000	15.000	15.000	15.000			



AUX. BLDG. 2000'-0"

(RADIATION LEVELS INDICATED ARE IN R/HR)

Time		0800 (B+00:00)	0815 (B+00:15)	0830 (B+00:30)	0845 (B+00:45)	0900 (B+01:00)	0915 (B+01:15)	0930 (B+01:30)	0945 (B+01:45)	1000 (B+02:00)
	Real / Relative	0800 (B+00:00)	0815 (B+00:15)	0830 (B+00:30)	0845 (B+00:45)	0900 (B+01:00)	0915 (B+01:15)	0930 (B+01:30)	0945 (B+01:45)	1000 (B+02:00)
A)	RHR Encapsulation Areas	0.015	0.015	0.015	0.015	0.015	0.015	0.015	0.015	0.015
B)	Corridor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
C)	Corridor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
Time	Real / Relative	1015 (B+02:15)	1030 (B+02:30)	1045 (B+02:45)	1100 (B+03:00)	1115 (B+03:15)	1130 (B+03:30)	1145 (B+03:45)	1200 (B+04:00)	1215 (B+04:15)
(A)	RHR Encapsulation Areas	0.100	0.100	0.100	0.700	1.200	1.200	1.200	1.200	1.200
(B)	Corridor Area	0.010	0.010	0.010	0.070	0.120	0.120	0.120	0.120	0.120
(C)	Corridor Area	0.005	0.005	0.005	0.035	0.600	0.600	0.600	0.600	0.600
Time	Real / Relative	1230 (B+04:30)	1245 (B+04:45)	1300 (B+05:00)	1315 (B+05:15)	1330 (B+05:30)	1345 (B+05:45)	1400 (B+06:00)	1415 (B+06:15)	1430 (B+06:30)
(A)	RHR Encapsulation Areas	1.200	1.200	1.200	1.200	1.200	800.000	800.000	800.000	800.000
(B)	Corridor Area	0.120	0.120	0.120	0.120	0.120	50.000	50.000	50.000	50.000
(C)	Corridor Area	0.600	0.600	0.600	0.600	0.600	15.000	15.000	15.000	15.000
Time	Real / Relative	1445 (B+06:45)	1300 (B+07:00)	1515 (B+07:15)	1530 (B+07:30)	1545 (B+07:45)	1600 (B+08:00)			
(A)	RHR Encapsulation Areas	800.000	800.000	800.000	800.000	800.000	800.000			
(B)	Corridor Area	50.000	50.000	50.000	50.000	50.000	50.000			
(C)	Corridor Area	15.000	15.000	15.000	15.000	15.000	15.000			



AUX. BLDG. 2026'-0"

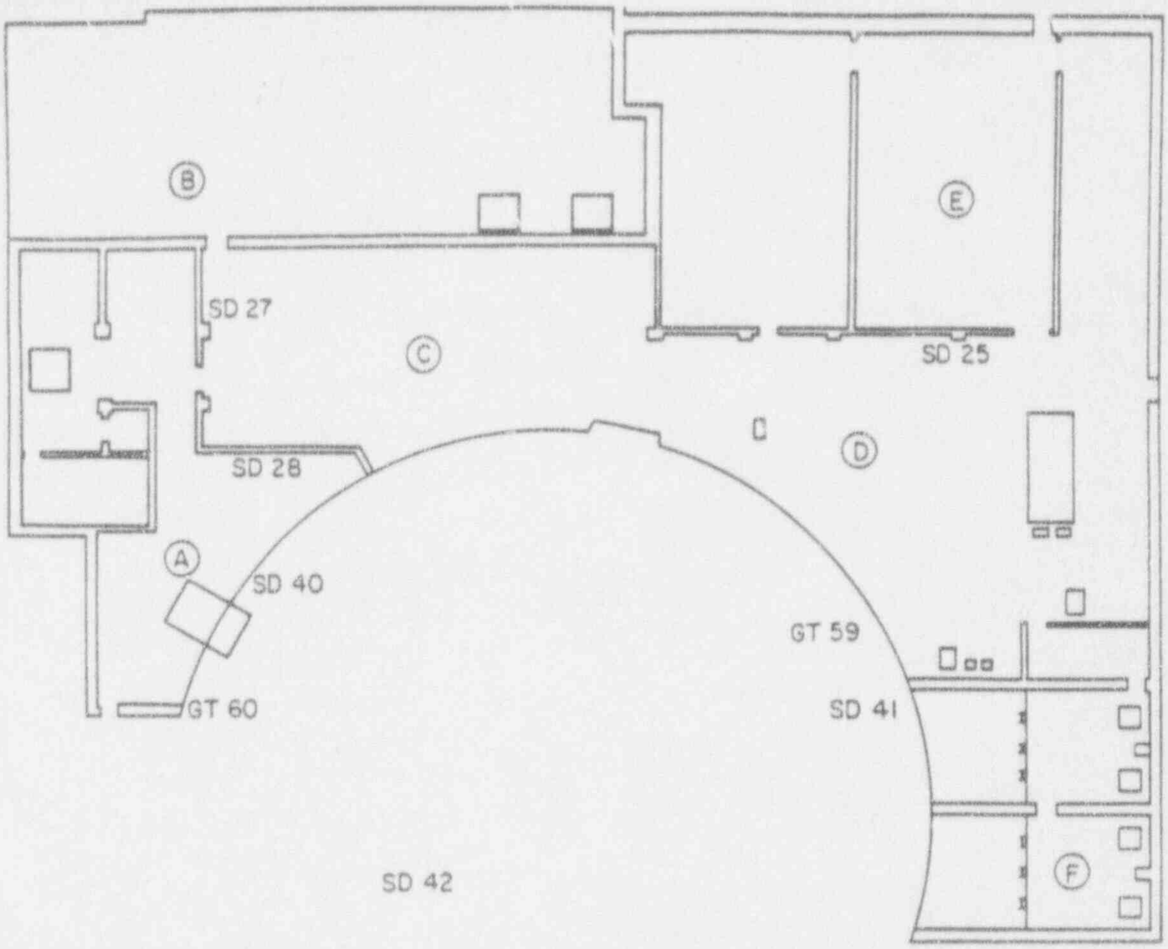
(RADIATION LEVELS INDICATED ARE IN R/HR)

Time	Real Relative	0800 (H+00:00)	0815 (H+00:15)	0830 (H+00:30)	0845 (H+00:45)	0900 (H+01:00)	0915 (H+01:15)	0930 (H+01:30)	0945 (H+01:45)	1000 (H+02:00)
(A)	Elect. Penetration Areas	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(B)	Corridor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(C)	CCW Pump Areas	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(D)	Main FW Pump Areas	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002

Time	Real Relative	1015 (H+02:15)	1030 (H+02:30)	1045 (H+02:45)	1100 (H+03:00)	1115 (H+03:15)	1130 (H+03:30)	1145 (H+03:45)	1200 (H+04:00)	1215 (H+04:15)
(A)	Elect. Penetration Areas	0.100	0.100	0.100	0.700	1.200	1.200	1.200	1.200	1.200
(B)	Corridor Area	0.100	0.100	0.100	0.700	1.200	1.200	1.200	1.200	1.200
(C)	CCW Pump Areas	0.010	0.010	0.010	0.070	0.120	0.120	0.120	0.120	0.120
(D)	Main FW Pump Areas	0.100	0.100	0.100	0.700	1.200	1.200	1.200	1.200	1.200

Time	Real Relative	1230 (H+04:30)	1245 (H+04:45)	1300 (H+05:00)	1315 (H+05:15)	1330 (H+05:30)	1345 (H+05:45)	1400 (H+06:00)	1415 (H+06:15)	1430 (H+06:30)
(A)	Elect. Penetration Areas	1.200	1.200	1.200	1.200	1.200	20.000	20.000	20.000	20.000
(B)	Corridor Area	1.200	1.200	1.200	1.200	1.200	20.000	20.000	20.000	20.000
(C)	CCW Pump Areas	0.120	0.120	0.120	0.120	0.120	2.000	2.000	2.000	2.000
(D)	Main FW Pump Areas	1.200	1.200	1.200	1.200	1.200	20.000	20.000	20.000	20.000

Time	Real Relative	1445 (H+06:45)	1500 (H+07:00)	1515 (H+07:15)	1530 (H+07:30)	1545 (H+07:45)	1600 (H+08:00)
(A)	Elect. Penetration Areas	20.000	20.000	20.000	20.000	20.000	20.000
(B)	Corridor Area	20.000	20.000	20.000	20.000	20.900	20.000
(C)	CCW Pump Areas	2.000	2.000	2.000	2.000	2.000	2.000
(D)	Main FW Pump Areas	20.000	20.000	20.000	20.000	20.000	20.000



AUX. BLDG. 2047'-6"

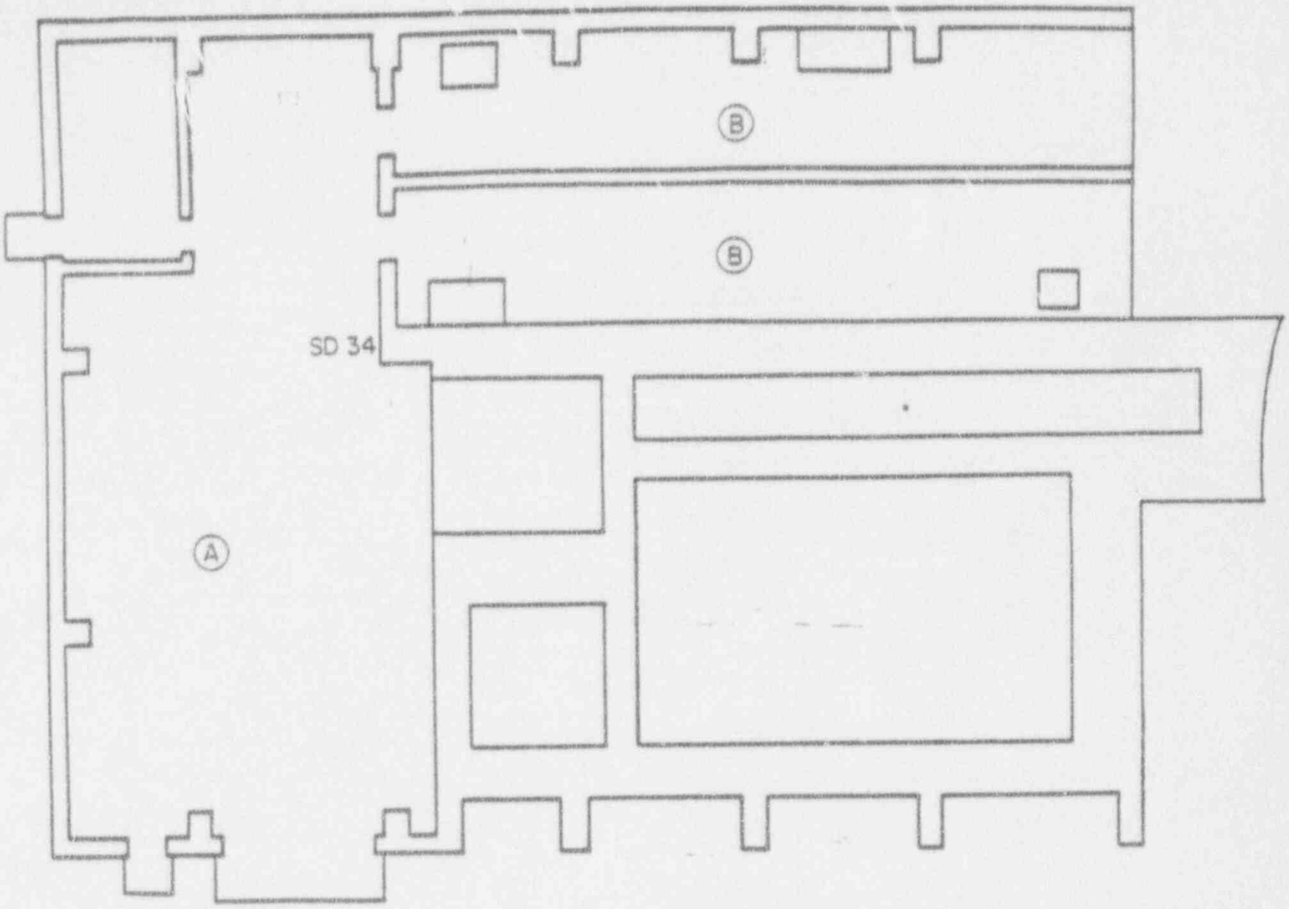
(RADIATION LEVELS INDICATED ARE IN R/HR)

Time	Real / Relative	0800 (H+00:00)	0815 (H+00:15)	0830 (H+00:30)	0845 (H+00:45)	0900 (H+01:00)	0915 (H+01:15)	0930 (H+01:30)	0945 (H+01:45)	1000 (H+02:00)
(A)	Personnel Hatch Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(B)	Aux Bldg Roof Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(C)	Ctmt Purge Exhaust Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(D)	Ctmt Purge Supply Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(E)	CR Filtration Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002

Time	Real / Relative	1015 (H+02:15)	1030 (H+02:30)	1045 (H+02:45)	1100 (H+03:00)	1115 (H+03:15)	1130 (H+03:30)	1145 (H+03:45)	1200 (H+04:00)	1215 (H+04:15)
(A)	Personnel Hatch Area	0.125	0.125	0.125	0.800	1.400	1.400	1.400	1.400	7000.000
(B)	Aux Bldg Roof Area	0.002	0.002	0.002	0.002	0.002	0.002	0.002	0.002	0.100
(C)	Ctmt Purge Exhaust Area	0.100	0.100	0.100	0.700	1.200	1.200	1.200	1.200	6000.000
(D)	Ctmt Purge Supply Area	0.100	0.100	0.100	0.700	1.200	1.200	1.200	1.200	6000.000
(E)	CR Filtration Area	0.002	0.002	0.002	0.002	0.002	0.002	0.002	0.002	0.100

Time	Real / Relative	1230 (H+04:30)	1245 (H+04:45)	1300 (H+05:00)	1315 (H+05:15)	1330 (H+05:30)	1345 (H+05:45)	1400 (H+06:00)	1415 (H+06:15)	1430 (H+06:30)
(A)	Personnel Hatch Area	9000.000	9000.000	12500.000	12500.000	500.000	300.000	220.000	180.000	180.000
(B)	Aux Bldg Roof Area	0.150	0.150	0.200	0.175	0.010	0.005	0.005	0.005	0.005
(C)	Ctmt Purge Exhaust Area	8000.000	8000.000	10500.000	10500.000	400.000	200.000	170.000	130.000	130.000
(D)	Ctmt Purge Supply Area	8000.000	8000.000	10500.000	10500.000	400.000	200.000	170.000	130.000	130.000
(E)	CR Filtration Area	0.150	0.150	0.200	0.175	0.010	0.005	0.005	0.005	0.005

Time	Real / Relative	1445 (H+06:45)	1500 (H+07:00)	1515 (H+07:15)	1530 (H+07:30)	1545 (H+07:45)	1600 (H+08:00)
(A)	Personnel Hatch Area	180.000	180.000	180.000	180.000	180.000	180.000
(B)	Aux Bldg Roof Area	0.005	0.005	0.005	0.005	0.005	0.005
(C)	Ctmt Purge Exhaust Area	130.000	130.000	130.000	130.000	130.000	130.000
(D)	Ctmt Purge Supply Area	130.000	130.000	130.000	130.000	130.000	130.000
(E)	CR Filtration Area	0.005	0.005	0.005	0.005	0.005	0.005



FUEL BLDG. 2000'-0"

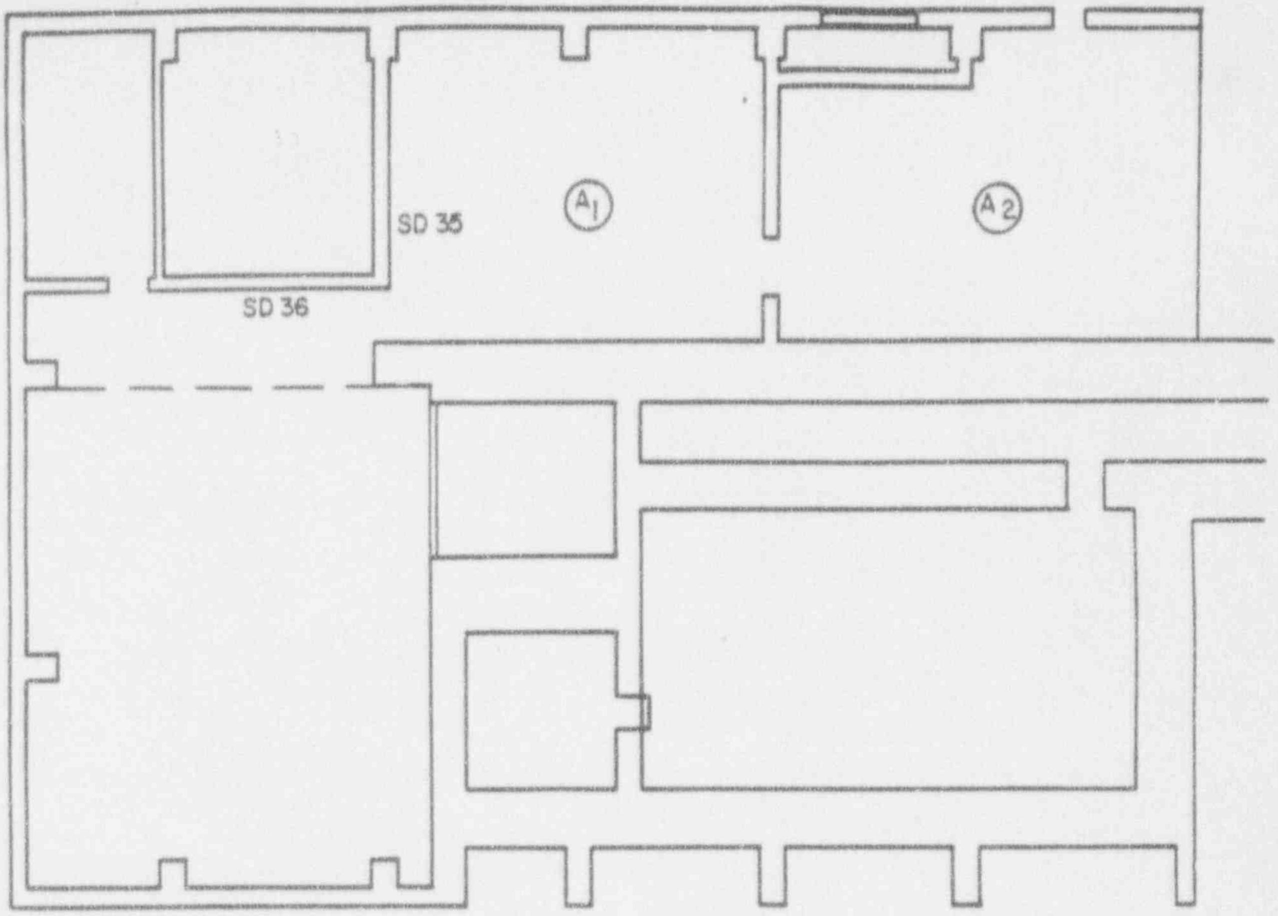
(RADIATION LEVELS INDICATED ARE IN R/HR)

Time Real / Relative	0800 (H+00:00)	0815 (H+00:15)	0830 (H+00:30)	0845 (H+00:45)	0900 (H+01:00)	0913 (H+01:15)	0930 (H+01:30)	0945 (H+01:45)	1000 (H+02:00)
(A) Laydown Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(B) Heat Exchange Area	0.005	0.005	0.005	0.005	0.005	0.005	0.005	0.005	0.005

Time Real / Relative	1015 (H+02:15)	1030 (H+02:30)	1045 (H+02:45)	1100 (H+03:00)	1115 (H+03:15)	1130 (H+03:30)	1145 (H+03:45)	1200 (H+04:00)	1215 (H+04:15)
(A) Laydown Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(B) Heat Exchange Area	0.005	0.005	0.005	0.005	0.005	0.005	0.005	0.005	0.005

Time Real / Relative	1230 (H+04:30)	1245 (H+04:45)	1300 (H+05:00)	1315 (H+05:15)	1330 (H+05:30)	1345 (H+05:45)	1400 (H+06:00)	1415 (H+06:15)	1430 (H+06:30)
(A) Laydown Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(B) Heat Exchange Area	0.005	0.005	0.005	0.005	0.005	0.005	0.005	0.005	0.005

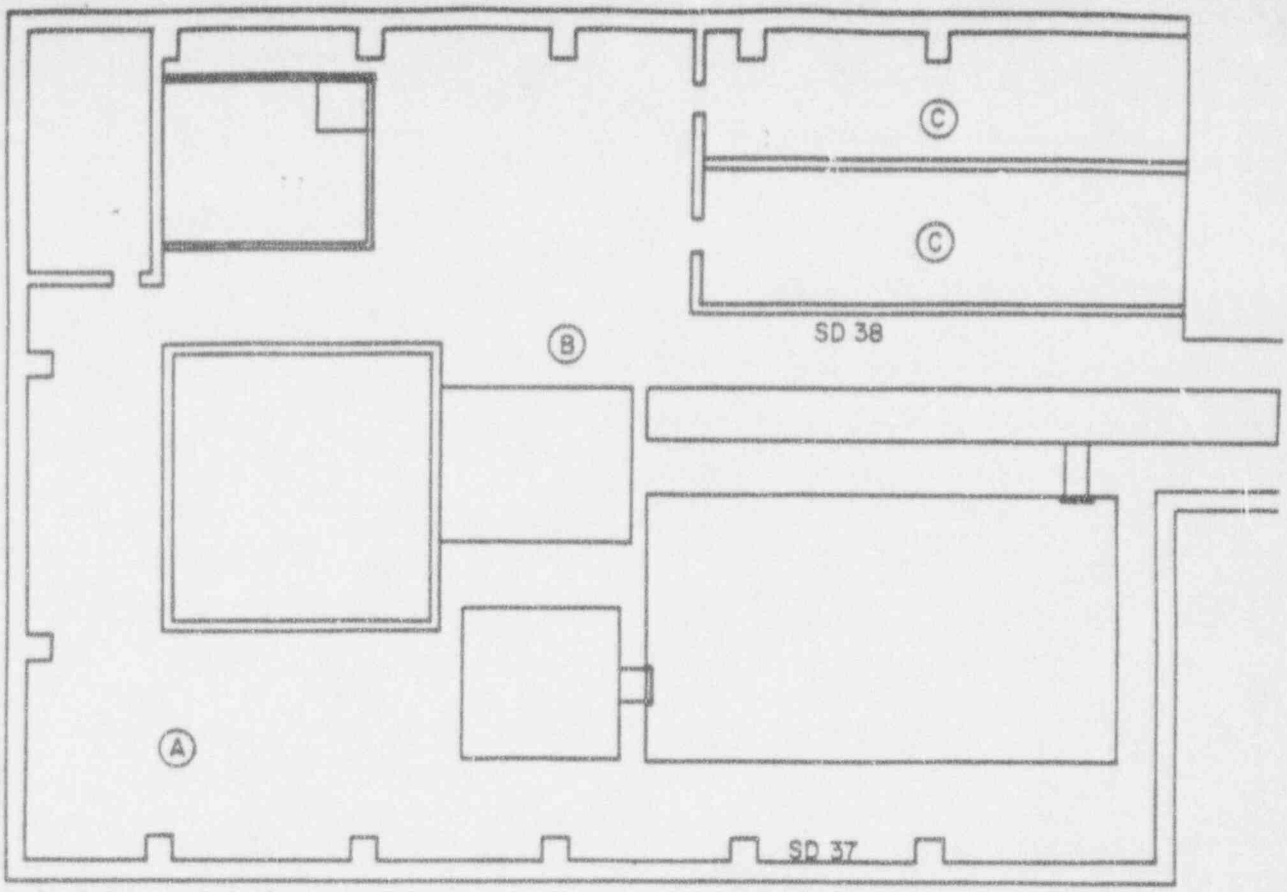
Time Real / Relative	1445 (H+06:45)	1500 (H+07:00)	1515 (H+07:15)	1530 (H+07:30)	1545 (H+07:45)	1600 (H+08:00)
(A) Laydown Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(B) Heat Exchange Area	0.005	0.005	0.005	0.005	0.005	0.005



FUEL BLDG. 2026'-0"

(RADIATION LEVELS INDICATED ARE IN R/HR)

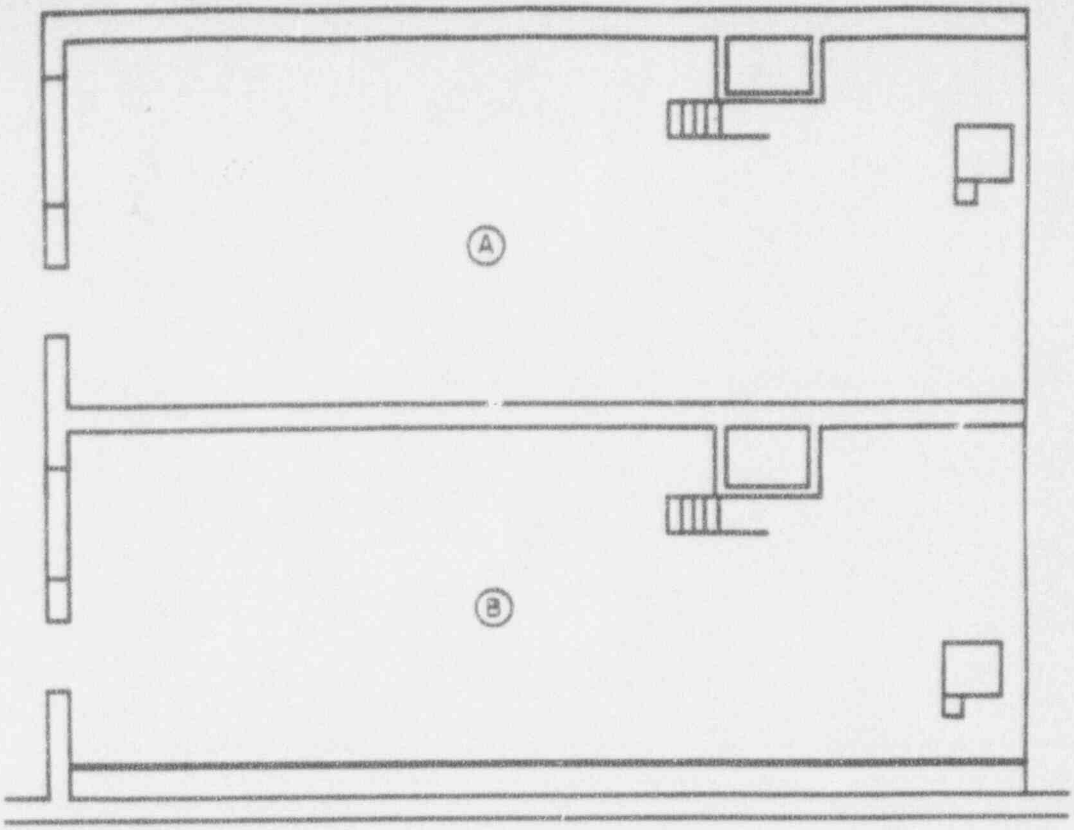
Time Real Relative	0800 (H+00:00)	0815 (H+00:15)	0830 (H+00:30)	0845 (H+00:45)	0900 (H+01:00)	0915 (H+01:15)	0930 (H+01:30)	0945 (H+01:45)	1000 (H+02:00)
(A) Fuel Bldg AC Areas	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
Time Real Relative	1015 (H+02:15)	1030 (H+02:30)	1045 (H+02:45)	1100 (H+03:00)	1115 (H+03:15)	1130 (H+03:30)	1145 (H+03:45)	1200 (H+04:00)	1215 (H+04:15)
(A) Fuel Bldg AC Areas	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	0.020
Time Real Relative	1230 (H+04:30)	1245 (H+04:45)	1300 (H+05:00)	1315 (H+05:15)	1330 (H+05:30)	1345 (H+05:45)	1400 (H+06:00)	1415 (H+06:15)	1430 (H+06:30)
(A) Fuel Bldg AC Areas	0.100	0.800	4.000	9.000	9.000	9.000	9.000	9.000	9.000
Time Real Relative	1445 (H+06:45)	1500 (H+07:00)	1515 (H+07:15)	1530 (H+07:30)	1545 (H+07:45)	1600 (H+08:00)			
(A) Fuel Bldg AC Areas	9.000	9.000	9.000	9.000	9.000	9.000			



FUEL BLDG. 2047'-6"

(RADIATION LEVELS INDICATED ARE IN R/HR)

Time	Real / Relative								
Time Real	0800	0815	0830	0845	0900	0915	0930	0945	1000
Relative	(H+00:00)	(H+00:15)	(H+00:30)	(H+00:45)	(H+01:00)	(H+01:15)	(H+01:30)	(H+01:45)	(H+02:00)
(A) Laydown Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(B) Corridor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(C) Emergency Filtration Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
Time Real	1015	1030	1045	1100	1115	1130	1145	1200	1215
Relative	(H+02:15)	(H+02:30)	(H+02:45)	(H+03:00)	(H+03:15)	(H+03:30)	(H+03:45)	(H+04:00)	(H+04:15)
(A) Laydown Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 2.000
(B) Corridor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 2.000
(C) Emergency Filtration Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	1.500
Time Real	1230	1245	1300	1315	1330	1345	1400	1415	1430
Relative	(H+04:30)	(H+04:45)	(H+05:00)	(H+05:15)	(H+05:30)	(H+05:45)	(H+06:00)	(H+06:15)	(H+06:30)
(A) Laydown Area	1.000	1.000	4.000	5.000	7.000	9.000	9.000	9.000	9.000
(B) Corridor Area	100.000	100.000	300.000	500.000	400.000	100.000	90.000	50.000	50.000
(C) Emergency Filtration Area	1000.000	1000.000	3000.000	5000.000	4000.000	1000.000	9000.000	5000.000	5000.000
Time Real	1445	1500	1515	1530	1545	1600			
Relative	(H+06:45)	(H+07:00)	(H+07:15)	(H+07:30)	(H+07:45)	(H+08:00)			
(A) Laydown Area	9.000	9.000	9.000	9.000	9.000	9.000			
(B) Corridor Area	50.000	50.000	50.000	50.000	50.000	50.000			
(C) Emergency Filtration Area	5000.000	5000.000	5000.000	5000.000	5000.000	5000.000			



DIESEL GEN. BLDG. 2000'-0"

(RADIATION LEVELS INDICATED ARE IN R/HR)

Time Real / Relative	0800 (E+00:00)	0815 (E+00:15)	0830 (E+00:30)	0845 (E+00:45)	0900 (E+01:00)	0915 (E+01:15)	0930 (E+01:30)	0945 (E+01:45)	1000 (E+02:00)
(A) General Floor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(B) General Floor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002

Time Real / Relative	1015 (E+02:15)	1030 (E+02:30)	1045 (E+02:45)	1100 (E+03:00)	1115 (E+03:15)	1130 (E+03:30)	1145 (E+03:45)	1200 (E+04:00)	1215 (E+04:15)
(A) General Floor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(B) General Floor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002

Time Real / Relative	1230 (E+04:30)	1245 (E+04:45)	1300 (E+05:00)	1315 (E+05:15)	1330 (E+05:30)	1345 (E+05:45)	1400 (E+06:00)	1415 (E+06:15)	1430 (E+06:30)
(A) General Floor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(B) General Floor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002

Time Real / Relative	1445 (E+06:45)	1500 (E+07:00)	1515 (E+07:15)	1530 (E+07:30)	1545 (E+07:45)	1600 (E+08:00)
(A) General Floor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002
(B) General Floor Area	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002	< 0.002

ONSITE PLUME MONITORING

Time-related onsite plume monitoring data is provided in the following subsection. Radiological data is provided in the units as indicated. The data is designated by a letter and corresponds to the lettered isopleths on the map.

Count rates do not include background. Controllers should determine what the background reading is and add the value to the ncpm prior to passing the data to the survey team members.

Radiological data used in determining beta dose rates takes into account a beta correction factor of 4.0. Field team members will be required to calculate beta dose rates from the data passed to them.

e.g. To determine the beta dose rate at H+04:30 on isopleth line "B":

Closed Window Reading (gamma) = 285 mR/hr
Open Window Reading (gamma + beta) = 340 mR/hr

Beta Dose Rate = 4 X (42 R/hr - 32 R/hr) = 40 R/hr

Iodine and particulate count rates of air samples whose volumes differ from 20 ft^3 should be multiplied by an appropriate factor as follows:

$$\frac{\text{Actual Sample Volume } (ft^3)}{20\ ft^3} \times \frac{\text{Sample ncpm at } 20\ ft^3}{1} = \text{Sample ncpm at Actual Volume}$$

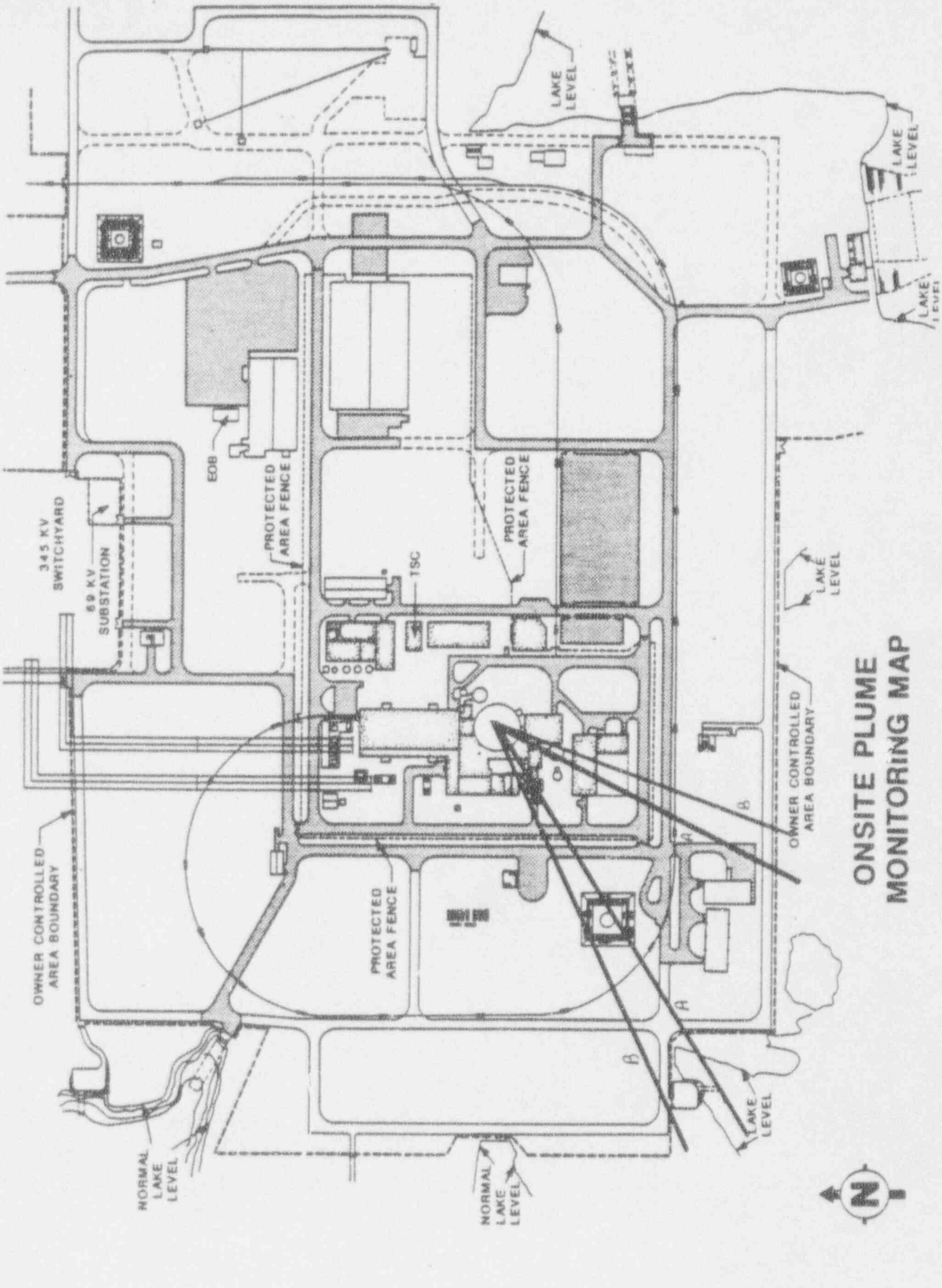
Iodine count rates are based on gross iodine. Iodine 131 count rate is determined by multiplying by the isotopic ratio of I-131 to total iodine listed under "Aux Bldg" on pages 7.2 - 7.13 for the same time period. For example, at H+04:30 for isopleth line "B":

Gross Iodine Concentration = 902 ncpm/20 ft^3

$$\text{I-131 Conc. } (\text{ncpm}/20\ ft^3) = (902\ \text{ncpm}/20\ ft^3) \times \frac{4.75E+01\ \text{mCi/cc}}{2.09E+02\ \text{mCi/cc}}$$

$$= 205\ \text{ncpm}/20\ ft^3$$

Gross particulate and iodine count rates determined with an HP-210 probe or equivalent.



ONSITE PLUME MONITORING MAP

93-FE

ONSITE PLUME DATA

TIME	ISOPLETH	DIST FT. (MI)	X/Q	OPEN	CLOSED R/HR	BETA R/HR	PARTICULATE CPM	PARTICULATE mCi/cc	Iodine CPM	Iodine uCi/cc	Iodine R/hr
1218-1350 (H+04:18 - 05:50)	A	500 (0.0947)	2.97E-04	N/A	1035	N/A	27219	2.16E-07	28861	1.24E-05	7.455
	B	3500 (0.6629)	9.28E-06	42.028	32.329	38.795	850	6.76E-09	902	3.89E-07	0.233

ONSITE SURFACE DEPOSITION DATA

TIME	ISOPLETH	SURFACE (mR/hr -contact, RO-2)	100 cm^2 smear mR/hr - contact, RO-2
All Times	A	-	-
	B	-	-

SECTION 8.0

OFFSITE RADIOLOGICAL DATA

PLUME CONCENTRATIONS AND DOSE RATES

Time-related offsite plume monitoring data is provided in the following section. The data is presented in the units in which it would normally be available to the field teams from the instruments used to monitor the plume.

Data is presented for radiation exposure rates (open and closed window values) and for airborne concentrations of gross iodine and particulates. The airborne iodine and particulate concentrations do not include the background count rate, which should be added by the controller as appropriate.

Airborne concentrations are expressed as net counts per minute (NCPM) per a 20 cubic foot sample. Particulate and iodine count rates are based on gross particulate and iodine activities. In determining count rates for a specifc nuclide, the nuclide ratios (pages 7.1-7.21) listed under the heading "AUX BLDG" for the same time period as the air concentration and dose rates map of interest must be used along with the formulas. For sample sizes other than 20 cubic feet, the concentrations should be adjusted as follows:

$$\text{NCPM for a } 20\ ft^3 \text{ sample from MAP I table} \times \frac{\text{Actual sample volume in } ft^3}{20} = \text{Adjusted NCPM}$$

Data is presented as follows:

Plume Isopleths (A, B, C, etc.): The controller should use the value for the nearest isopleth or should interpolate values between isopleths as appropriate.

Predesignated Monitoring Locations (X1, X2, etc.): For a field team located at a predesignated monitoring location, the controller may read the values directly from the table below the map.

Example:

What are the dose rate and airborne iodine and particulate concentations at predesignated location X2 at time 11:30? Assume a 15 cubic feet air sample is collected.

Solution:

° The following values for location X2 may be read directly from the table below.

Closed window dose rate: 1655 mR/hr

Open window dose rate: 2151 mR/hr

Gamma dose rate: <u>1655 mR/hr</u>

Beta dose rate: <u>(2151-1655) mR/hr</u> x appropriate beta correction <u>factor for the instrument used</u> (i.e. 4) = 1984

° Iodine cartridge count rate (excluding background) for location X1 is 108 NCPM for a 20 cubic feet sample. The corrected value for a 15 cubic feet sample would therefore be:

Airborne iodine: $108 \times \frac{15}{20}$ = <u>81 NCPM</u>

° If the count rate for the particulate filter is above the maximum scale for the RM-14; then an RO-2 detector is used to measure the contact dose rate from the filter. If the reading is 17 mR/hr for 20 cu. ft. then adjustments shall be made as follows:

Particulate concentration: 17 mR/hr x $\frac{15}{20}$ = <u>12.75 mR/hr.</u>

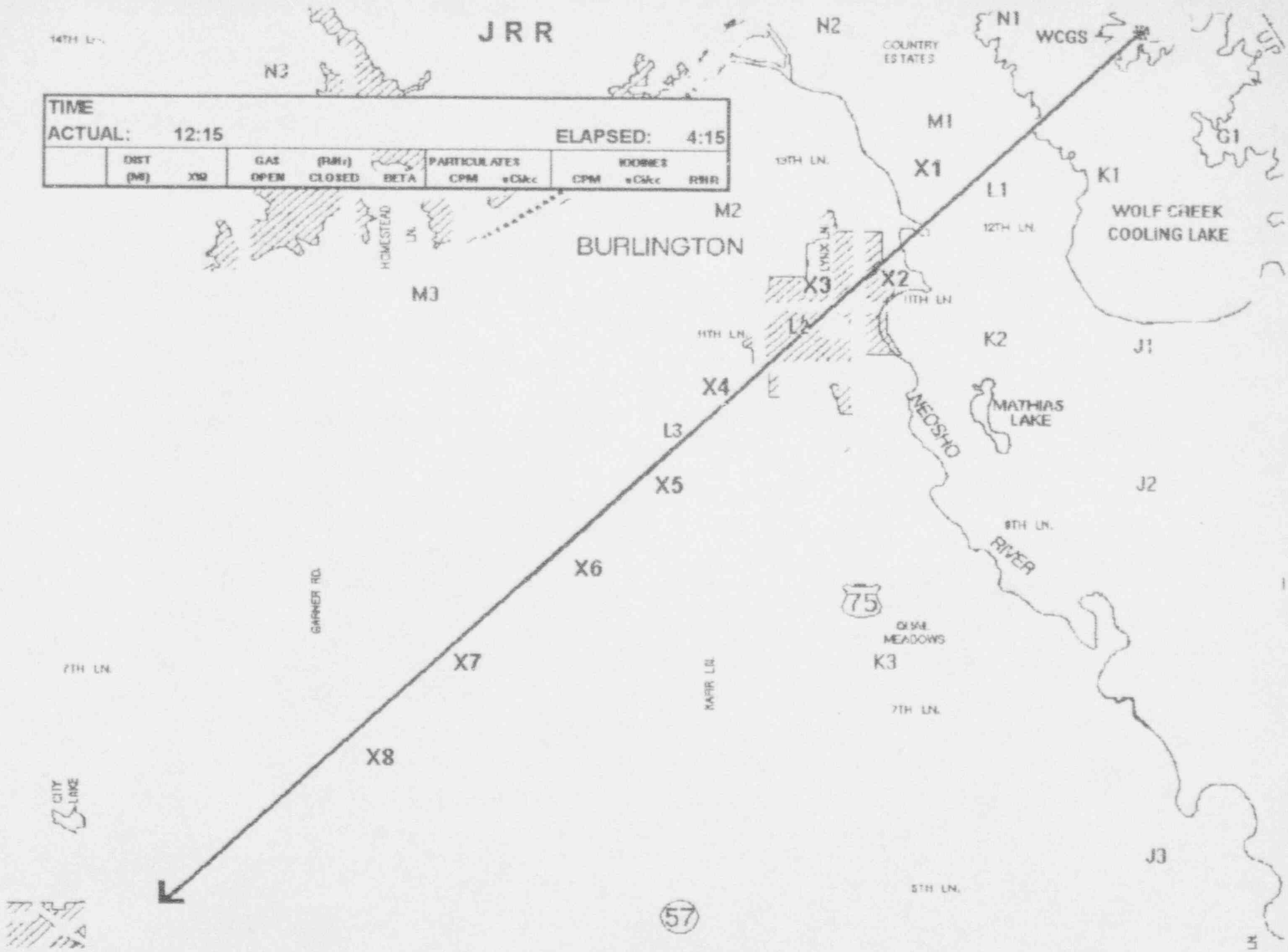
TABLE.XLS

TIME ACTUAL: 12:00								ELAPSED: 6:00		
	DIST (MI)	X\Q	GAS (R/Hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R\HR
EAB	0.75	7.45E-06	33.740	25.954	31.145	683.	5.43E-09	725.	3.12E-07	0.187
AA	1.8	1.57E-06	7.110	5.470	6.563	144.	1.14E-09	153.	6.58E-08	0.039
2 MI	2	1.30E-06	5.888	4.529	5.435	119.	9.48E-10	126.	5.45E-08	0.033
X1	2.18	1.11E-06	5.027	3.867	4.640	102.	8.09E-10	108.	4.65E-08	0.028
X2	3.5	4.75E-07	2.151	1.655	1.986	44.	3.46E-10	46.	1.99E-08	0.012
X3	3.77	4.15E-07	1.879	1.446	1.735	38.	3.02E-10	40.	1.74E-08	0.010
BB	4.3	3.27E-07	1.481	1.139	1.367	30.	2.38E-10	32.	1.37E-08	0.008
X4	4.66	2.83E-07	1.282	0.986	1.183	26.	2.06E-10	28.	1.19E-08	0.007
5 MI	5	2.49E-07	1.128	0.867	1.041	23.	1.81E-10	24.	1.04E-08	0.006
X5	5.89	1.85E-07	0.838	0.644	0.773	17.	1.35E-10	18.	7.76E-09	0.005
CC	6.8	1.43E-07	0.648	0.498	0.598	13.	1.04E-10	14.	6.00E-09	0.004
X6	7.12	1.31E-07	0.593	0.456	0.546	12.	9.55E-11	13.	5.49E-09	0.003
X7	8.39	9.75E-08	0.442	0.340	0.408	9.	7.11E-11	9.	4.09E-09	0.002
DD	9.3	8.09E-08	0.366	0.282	0.338	7.	5.90E-11	8.	3.39E-09	0.002
X8	9.7	7.49E-08	0.339	0.261	0.313	7.	5.46E-11	7.	3.14E-09	0.002
10 MI	10	7.09E-08	0.321	0.247	0.296	6.	5.17E-11	7.	2.97E-09	0.002

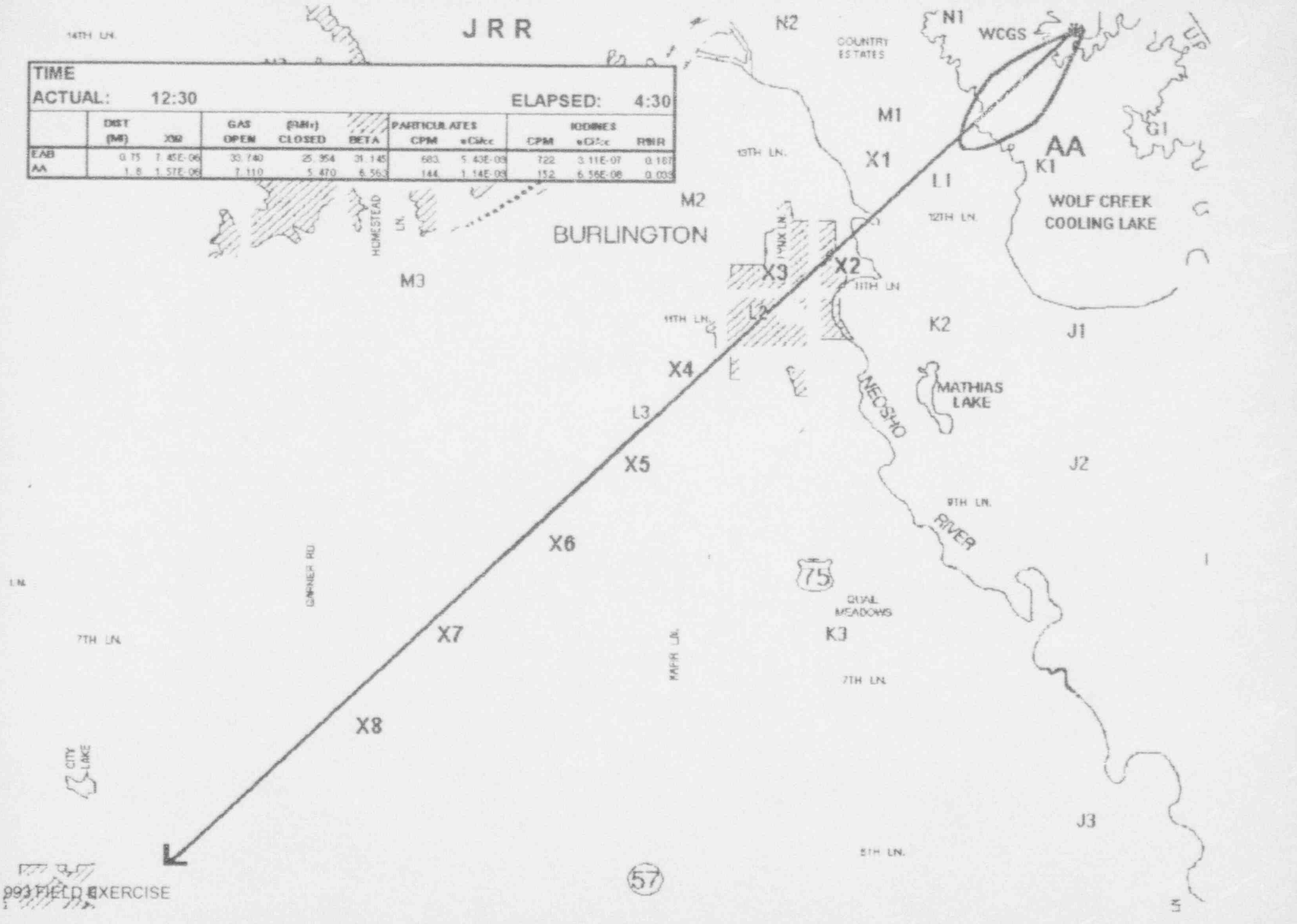
TIME

ACTUAL: 12:15 ELAPSED: 4:15

	DIST (MI)	X/Q	GAS (R/Hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR



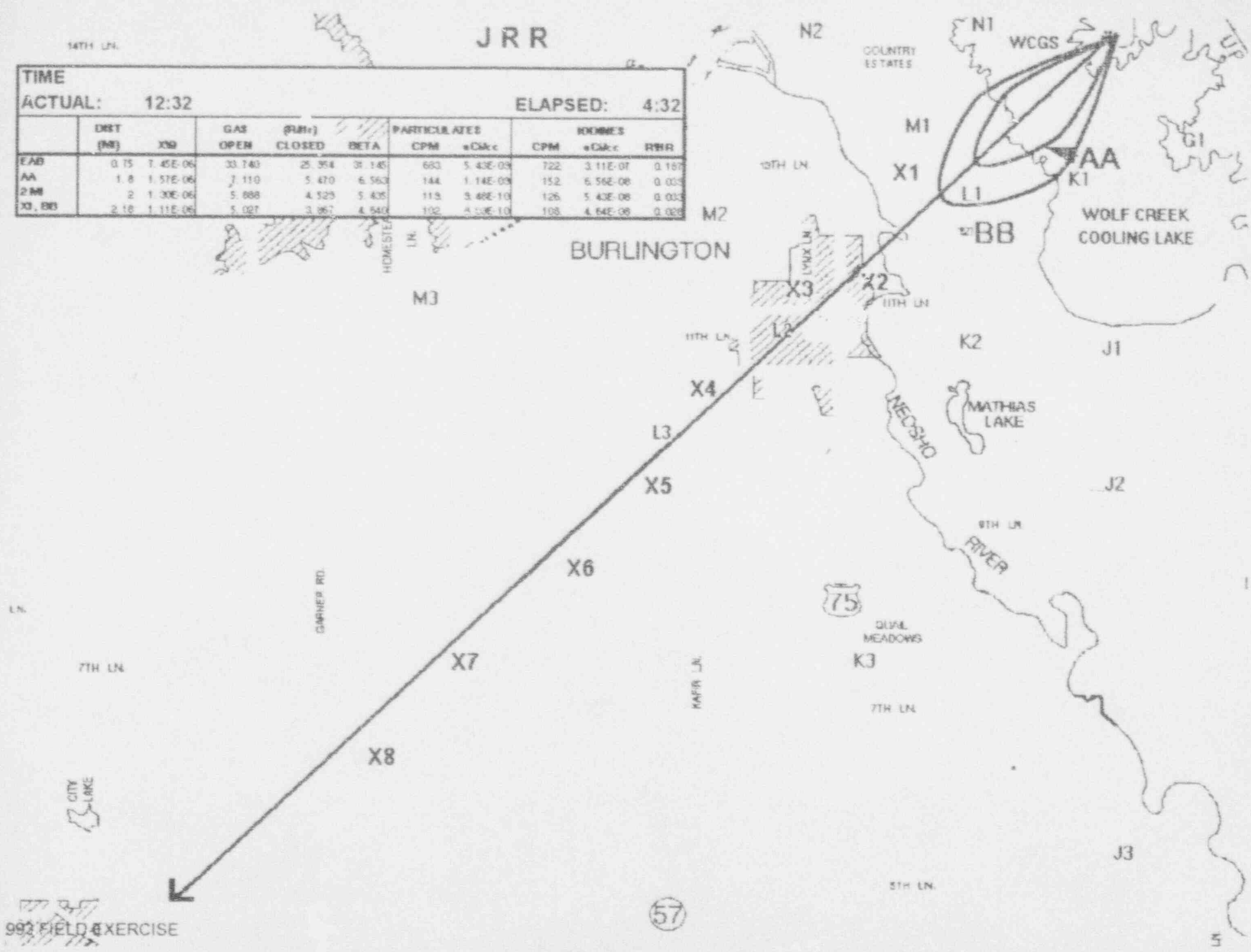
1993 FIELD EXERCISE



TIME ACTUAL: 12:30										ELAPSED: 4:30
	DIST (MI)	X/Q	GAS OPEN	(R/Hr) CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR
EAB	0.75	7.45E-06	33.740	25.354	31.145	683.	5.43E-09	722	3.11E-07	0.187
AA	1.8	1.57E-06	7.110	5.470	6.563	144.	1.14E-09	152	6.56E-08	0.039

993 FIELD EXERCISE

TIME ACTUAL: 12:32										ELAPSED: 4:32
	DIST (MI)	X/Q	GAS OPEN	(R/Hr) CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR
EAB	0.75	7.45E-06	33.740	25.354	31.145	683	5.43E-09	722	3.11E-07	0.167
AA	1.8	1.57E-06	7.110	5.470	6.563	144	1.14E-09	152	6.56E-08	0.035
2 MI	2	1.30E-06	5.888	4.529	5.435	119	9.48E-10	126	5.43E-08	0.033
X1, BB	2.18	1.11E-06	5.027	3.867	4.540	102	8.08E-10	108	4.64E-08	0.028



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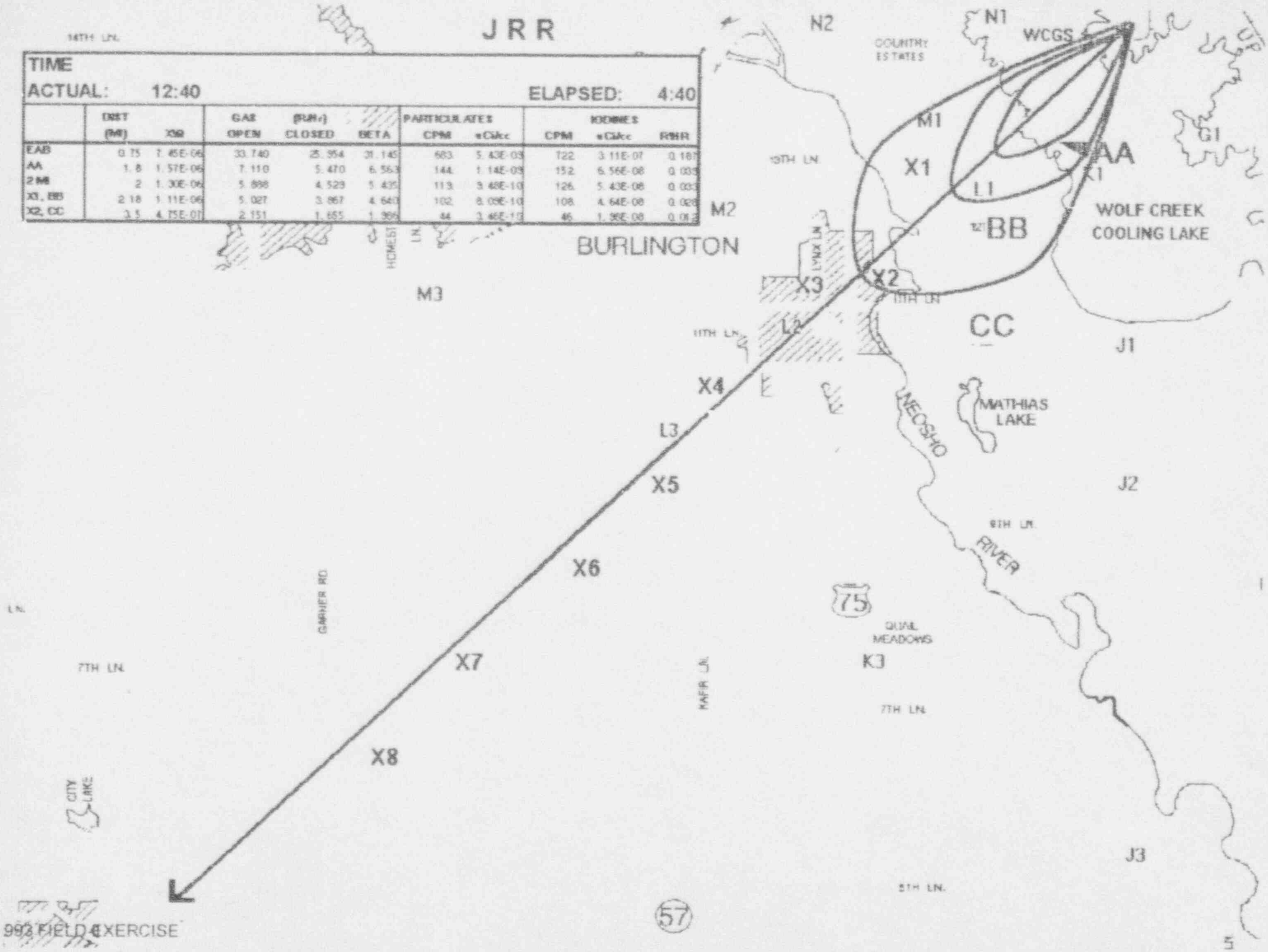
99? FIELD EXERCISE



TIME
ACTUAL: 12:40 ELAPSED: 4:40

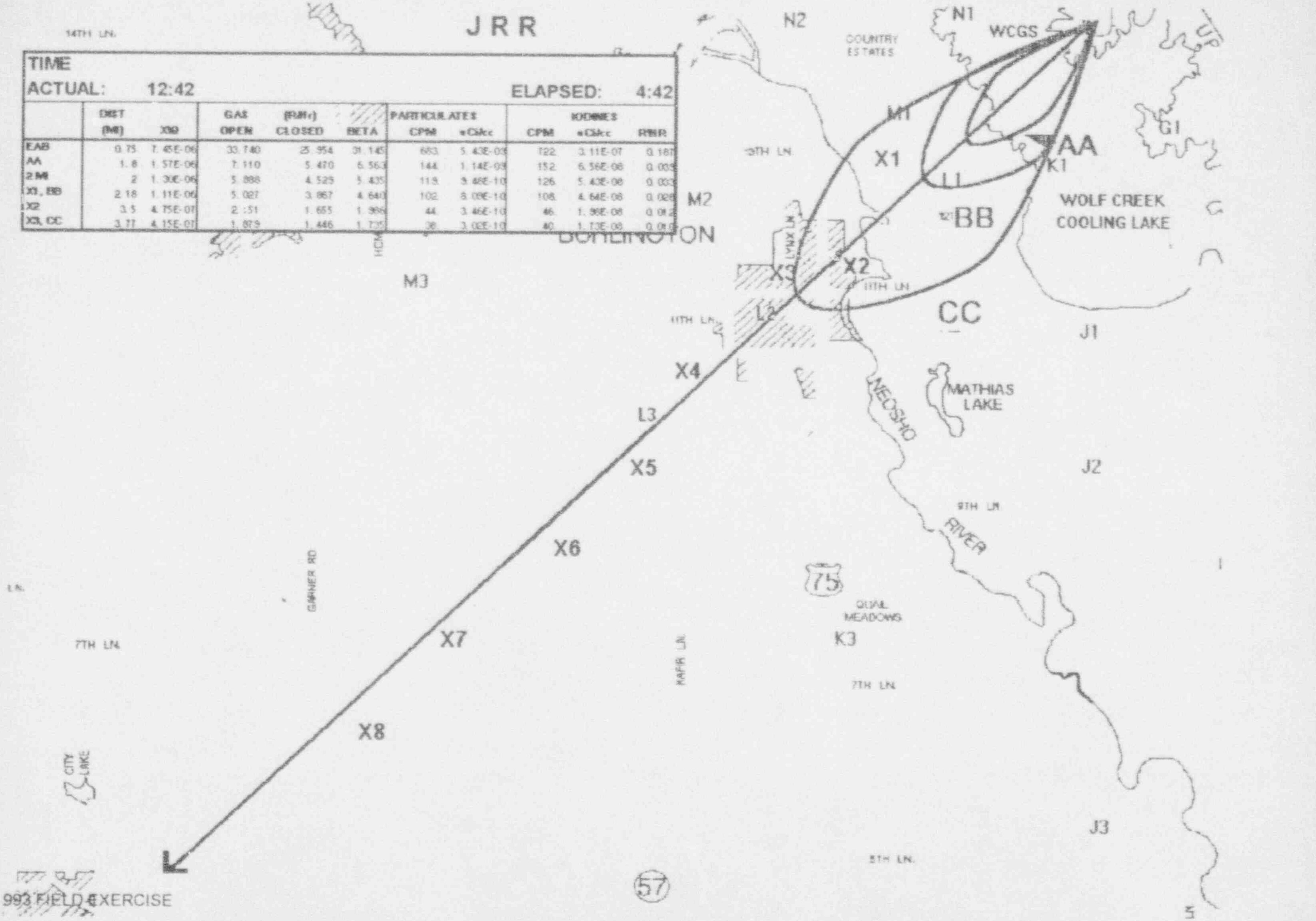
	DIST (MI)	X/Q	GAS (R/hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR
EAB	0.75	7.45E-06	33.740	25.354	31.145	683.	5.43E-09	722.	3.11E-07	0.181
AA	1.8	1.57E-06	7.110	5.470	6.563	144.	1.14E-09	152.	6.56E-08	0.039
2 MI	2	1.30E-06	5.898	4.529	5.435	113.	9.48E-10	126.	5.43E-08	0.033
X1, BB	2.18	1.11E-06	5.027	3.867	4.640	102.	8.09E-10	108.	4.64E-08	0.028
X2, CC	3.5	4.75E-07	2.151	1.655	1.986	44.	3.46E-10	46.	1.98E-08	0.012

993 FIELD EXERCISE

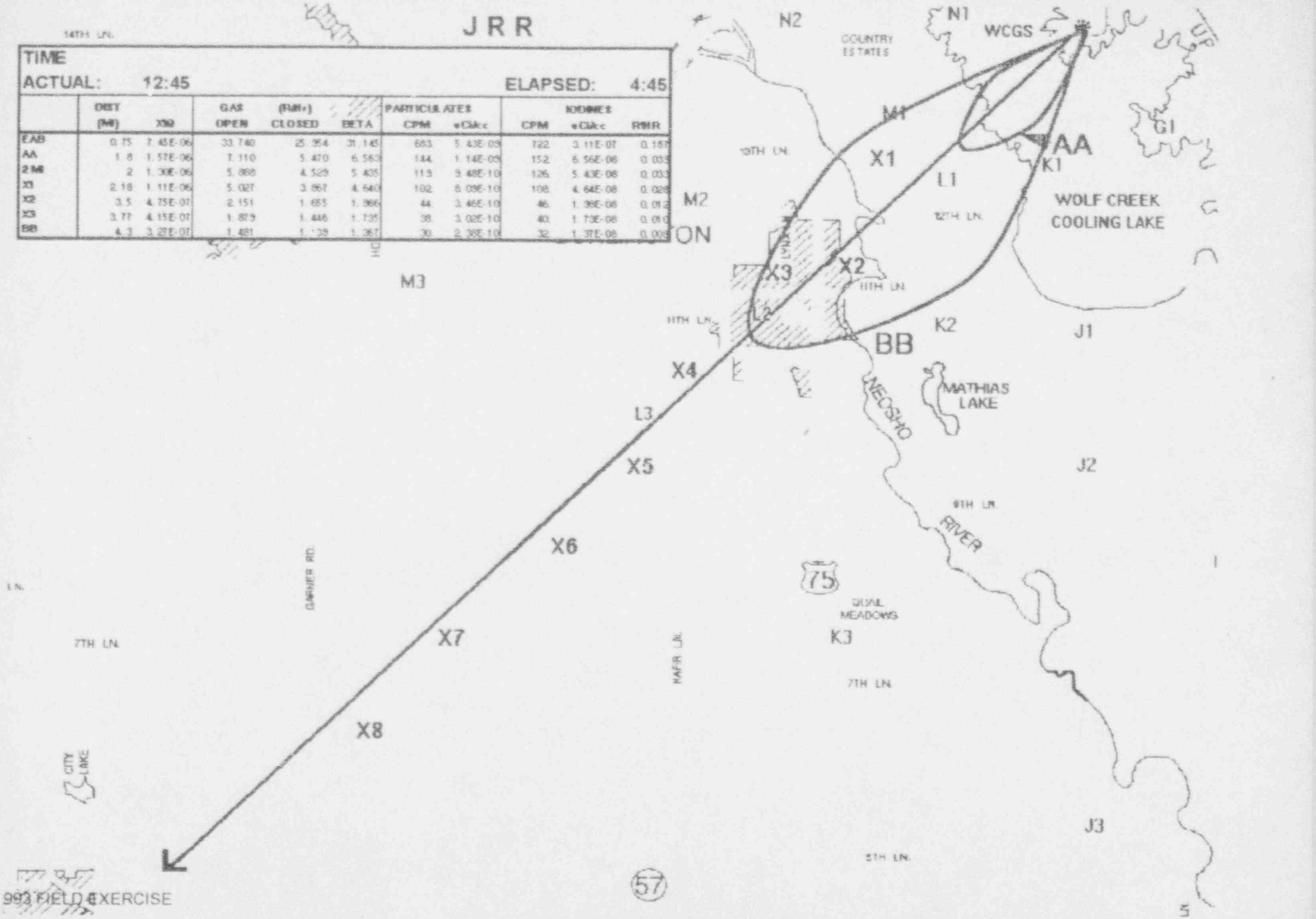
TIME

ACTUAL: 12:42 ELAPSED: 4:42

	DIST (MI)	X/Q	GAS (R/Hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR
EAB	0.75	7.45E-06	33.740	25.954	31.145	663.	5.43E-09	722.	3.11E-07	0.187
AA	1.8	1.57E-06	7.110	5.470	6.563	144.	1.14E-09	152.	6.56E-08	0.005
2 MI	2	1.30E-06	5.888	4.529	5.435	119.	9.48E-10	126.	5.43E-08	0.003
X1, BB	2.18	1.11E-06	5.027	3.867	4.640	102.	8.09E-10	108.	4.64E-08	0.026
X2	3.5	4.75E-07	2.151	1.655	1.986	44.	3.46E-10	46.	1.98E-08	0.012
X3, CC	3.77	4.15E-07	1.879	1.446	1.735	38.	3.02E-10	40.	1.73E-08	0.010



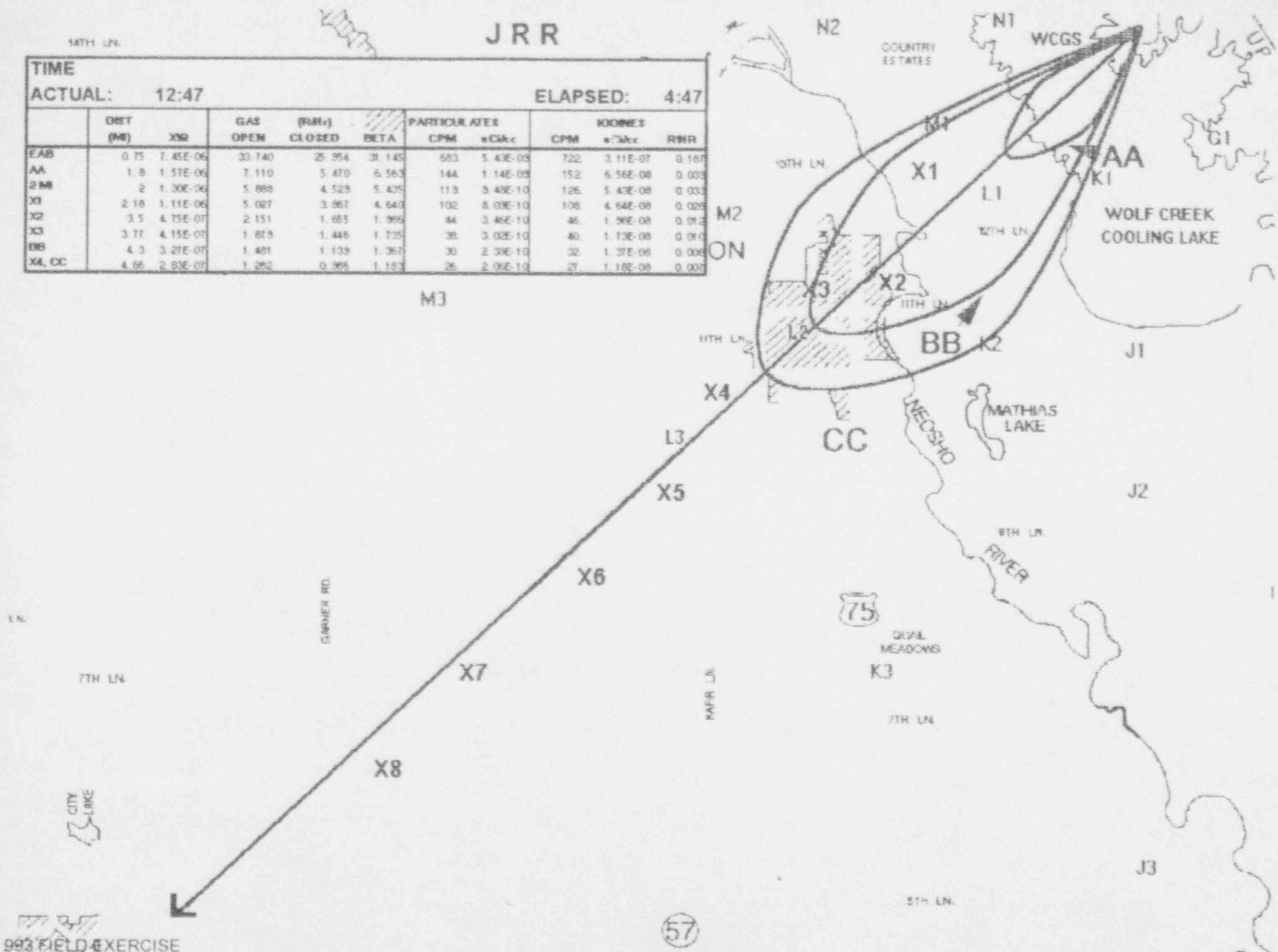
TIME										
ACTUAL: 12:45								ELAPSED: 4:45		
	DIST (MI)	X/Q	GAS (R/Hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	RMR
EAB	0.75	7.45E-06	33.740	25.354	31.145	683	5.43E-09	722	3.11E-07	0.187
AA	1.8	1.57E-06	7.110	5.470	6.563	144	1.14E-09	152	6.56E-08	0.035
2 MI	2	1.30E-06	5.868	4.529	5.435	119	9.48E-10	126	5.43E-08	0.033
X1	2.18	1.11E-06	5.027	3.867	4.640	102	8.09E-10	108	4.64E-08	0.028
X2	3.5	4.75E-07	2.151	1.655	1.986	44	3.46E-10	46	1.98E-08	0.012
X3	3.77	4.15E-07	1.879	1.446	1.735	38	3.02E-10	40	1.73E-08	0.010
BB	4.3	3.27E-07	1.481	1.139	1.367	30	2.38E-10	32	1.37E-08	0.008



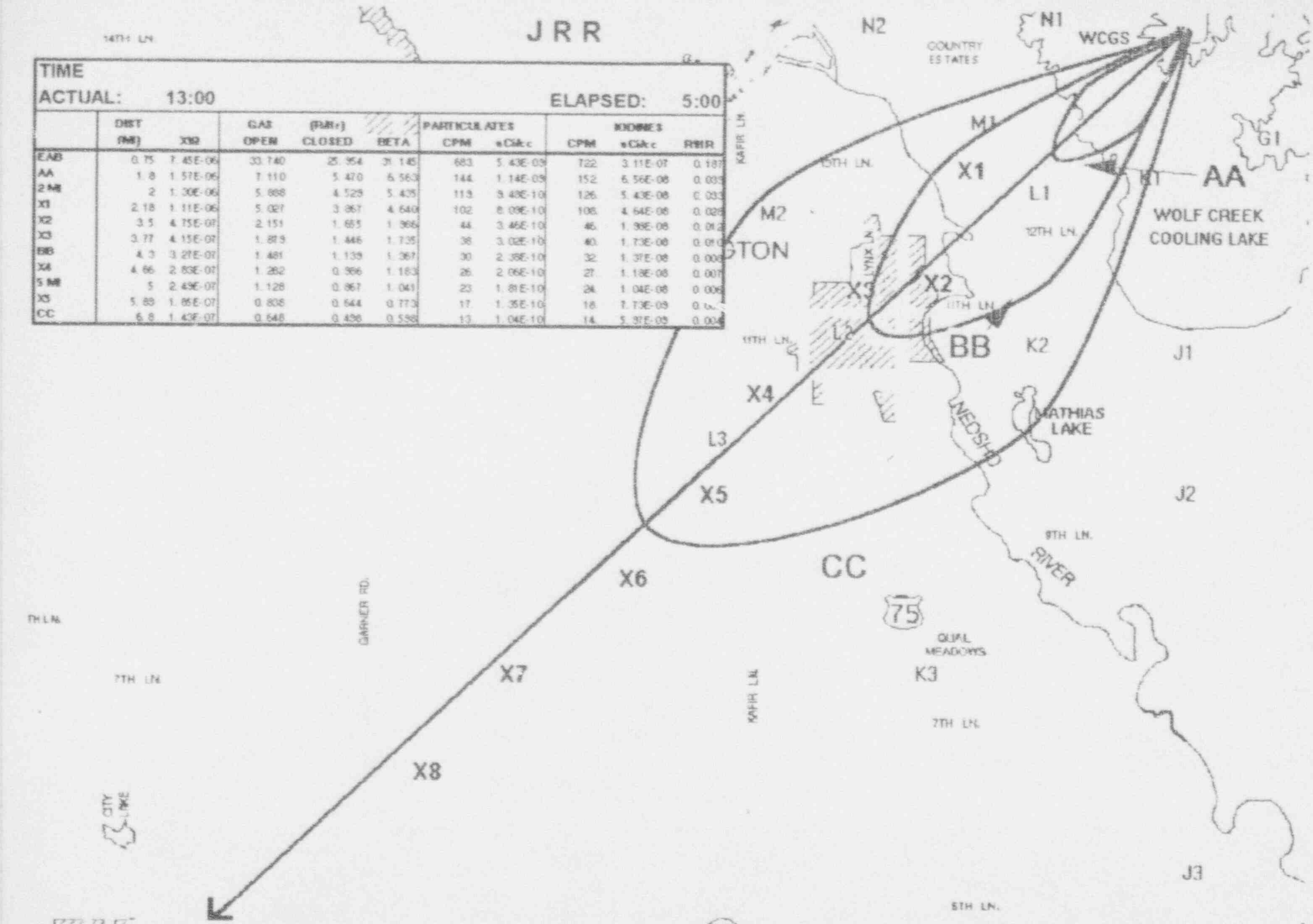
TIME

ACTUAL: 12:47 ELAPSED: 4:47

	DIST (MI)	X/Q	GAS (R/Hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR
EAB	0.75	1.45E-06	33.740	25.954	31.145	683	5.43E-09	722	3.11E-07	0.187
AA	1.8	1.57E-06	7.110	5.470	6.563	144	1.14E-09	152	6.56E-08	0.039
2 MI	2	1.30E-06	5.888	4.529	5.435	113	9.48E-10	126	5.43E-08	0.033
X1	2.18	1.11E-06	5.027	3.867	4.640	102	8.09E-10	108	4.64E-08	0.028
X2	3.5	4.75E-07	2.151	1.655	1.986	44	3.46E-10	46	1.98E-08	0.012
X3	3.77	4.15E-07	1.879	1.446	1.735	38	3.02E-10	40	1.73E-08	0.010
BB	4.3	3.27E-07	1.481	1.139	1.367	30	2.39E-10	32	1.37E-08	0.008
X4, CC	4.66	2.83E-07	1.282	0.985	1.183	26	2.06E-10	27	1.18E-08	0.007



993 FIELD EXERCISE



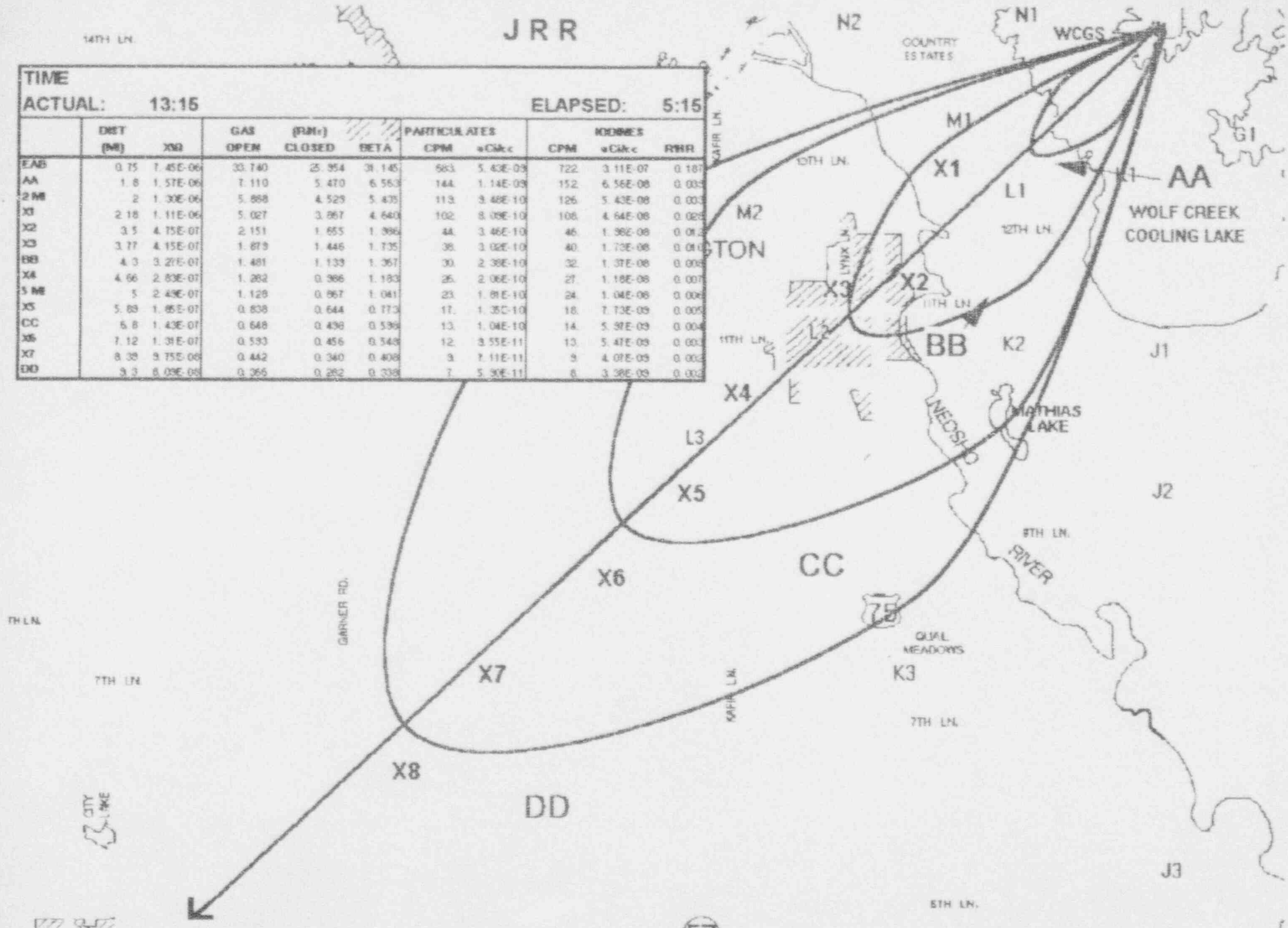
TIME
ACTUAL: 13:00 ELAPSED: 5:00

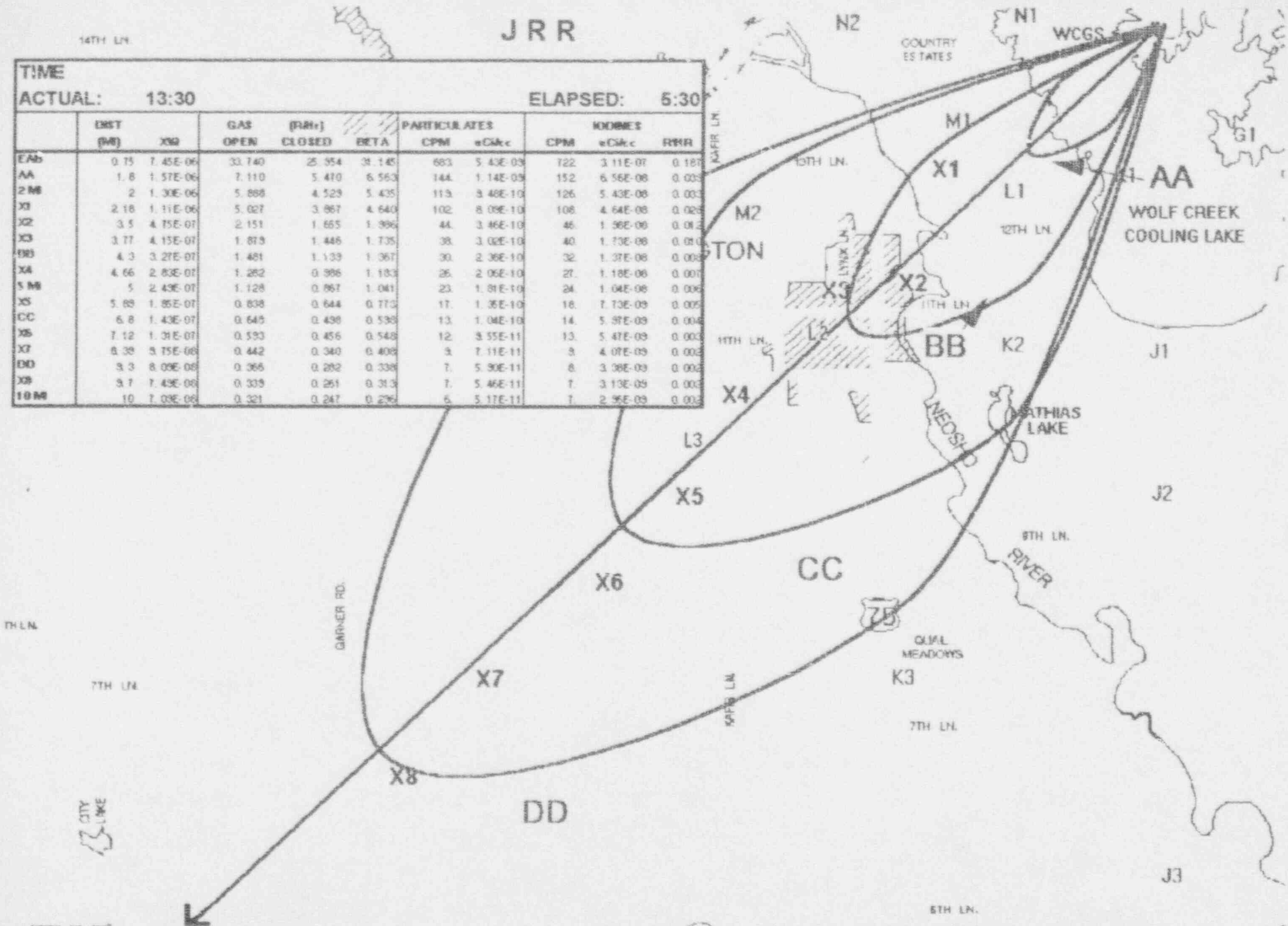
	DIST (MI)	X/Q	GAS (R/Hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR
EAB	0.75	7.45E-06	33.740	25.954	31.145	683	5.43E-09	722	3.11E-07	0.187
AA	1.8	1.57E-06	7.110	5.470	6.563	144	1.14E-09	152	6.56E-08	0.039
2 MI	2	1.30E-06	5.888	4.529	5.435	113	9.48E-10	126	5.43E-08	0.033
X1	2.18	1.11E-06	5.027	3.867	4.640	102	8.09E-10	106	4.64E-08	0.028
X2	3.5	4.75E-07	2.151	1.655	1.986	44	3.46E-10	46	1.98E-08	0.012
X3	3.77	4.15E-07	1.879	1.446	1.735	38	3.02E-10	40	1.73E-08	0.010
BB	4.3	3.27E-07	1.481	1.139	1.367	30	2.35E-10	32	1.37E-08	0.008
X4	4.66	2.83E-07	1.282	0.986	1.183	26	2.06E-10	27	1.18E-08	0.007
5 MI	5	2.49E-07	1.128	0.867	1.041	23	1.81E-10	24	1.04E-08	0.006
X5	5.89	1.86E-07	0.808	0.644	0.773	17	1.35E-10	18	7.73E-09	[illegible]
CC	6.8	1.43E-07	0.648	0.498	0.598	13	1.04E-10	14	5.97E-09	0.004

TIME

ACTUAL: 13:15 ELAPSED: 5:15

	DIST (MI)	X/Q	GAS (R/Hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR
EAB	0.75	7.45E-06	33.740	25.354	31.145	583.	5.43E-09	722.	3.11E-07	0.187
AA	1.8	1.57E-06	7.110	5.470	6.563	144.	1.14E-09	152.	6.56E-08	0.035
2 MI	2	1.30E-06	5.868	4.529	5.435	113.	3.48E-10	126.	5.43E-08	0.003
X1	2.18	1.11E-06	5.027	3.867	4.640	102.	8.09E-10	108.	4.64E-08	0.025
X2	3.5	4.75E-07	2.151	1.655	1.986	44.	3.46E-10	46.	1.98E-08	0.012
X3	3.77	4.15E-07	1.879	1.446	1.735	38.	3.02E-10	40.	1.73E-08	0.010
BB	4.3	3.27E-07	1.481	1.139	1.367	30.	2.38E-10	32.	1.37E-08	0.008
X4	4.66	2.83E-07	1.282	0.986	1.183	26.	2.06E-10	27.	1.18E-08	0.007
5 MI	5	2.49E-07	1.128	0.867	1.041	23.	1.81E-10	24.	1.04E-08	0.006
X5	5.89	1.85E-07	0.838	0.644	0.773	17.	1.35E-10	18.	7.73E-09	0.005
CC	6.8	1.43E-07	0.648	0.498	0.598	13.	1.04E-10	14.	5.97E-09	0.004
X6	7.12	1.31E-07	0.593	0.456	0.548	12.	9.55E-11	13.	5.47E-09	0.003
X7	8.38	9.75E-08	0.442	0.340	0.408	9.	7.11E-11	9.	4.07E-09	0.002
DD	9.3	8.09E-08	0.366	0.282	0.338	7.	5.30E-11	8.	3.38E-09	0.002

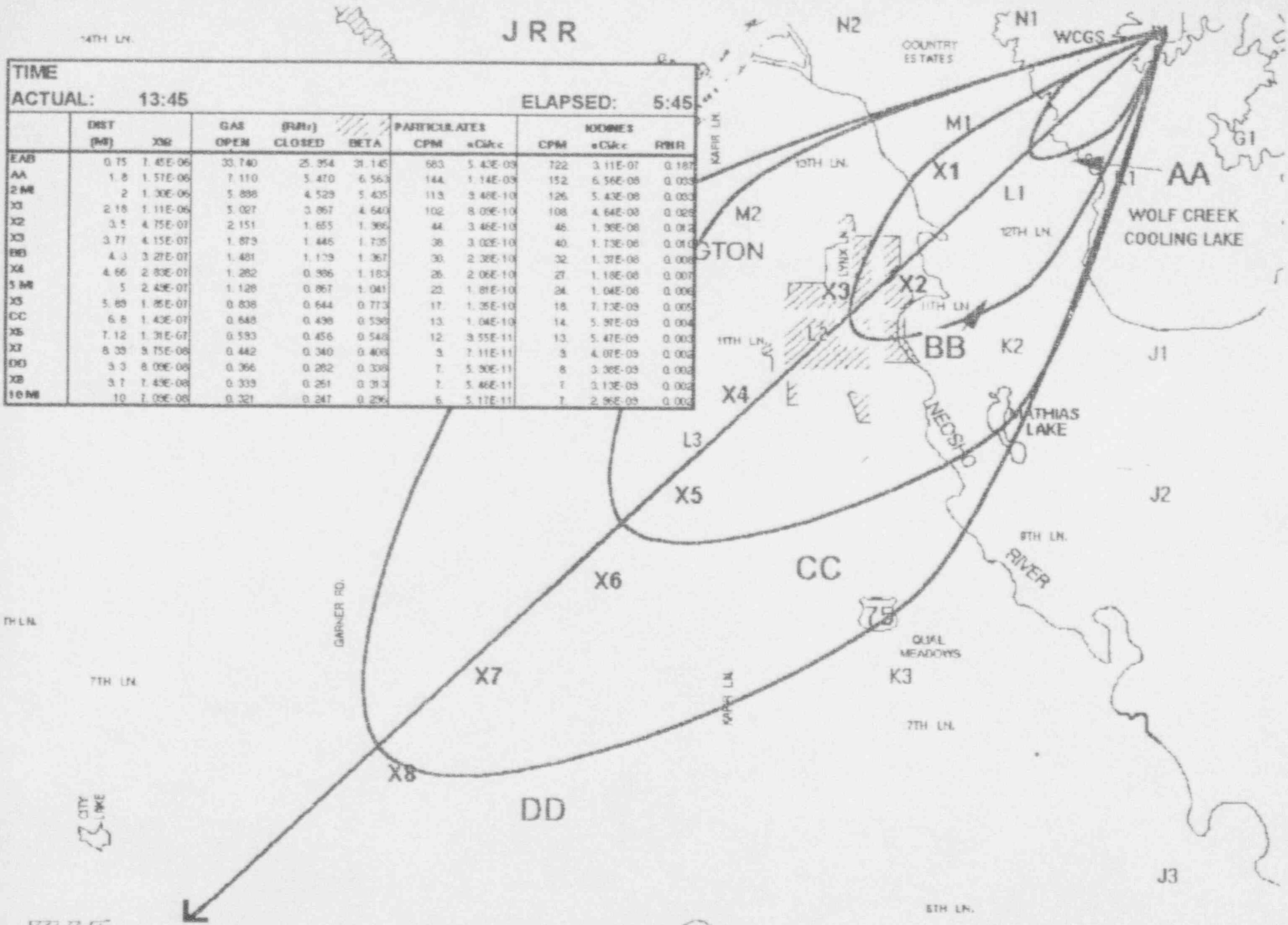




TIME

ACTUAL: 13:30 ELAPSED: 5:30

	DIST (MI)	X/Q	GAS (R/Hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR
EAb	0.75	7.45E-06	33.740	25.354	31.145	683.	5.43E-09	722.	3.11E-07	0.187
AA	1.8	1.57E-06	7.110	5.470	6.563	144.	1.14E-09	152.	6.56E-08	0.039
2 MI	2	1.30E-06	5.888	4.529	5.435	119.	9.48E-10	126.	5.43E-08	0.033
X1	2.18	1.11E-06	5.027	3.867	4.640	102.	8.09E-10	108.	4.64E-08	0.028
X2	3.5	4.75E-07	2.151	1.665	1.986	44.	3.46E-10	46.	1.98E-08	0.012
X3	3.77	4.15E-07	1.879	1.446	1.735	38.	3.02E-10	40.	1.73E-08	0.010
BB	4.3	3.27E-07	1.481	1.139	1.367	30.	2.38E-10	32.	1.37E-08	0.008
X4	4.66	2.83E-07	1.282	0.986	1.183	26.	2.06E-10	27.	1.18E-08	0.007
5 MI	5	2.49E-07	1.128	0.867	1.041	23.	1.81E-10	24.	1.04E-08	0.006
X5	5.89	1.85E-07	0.838	0.644	0.773	17.	1.35E-10	18.	7.73E-09	0.005
CC	6.8	1.43E-07	0.648	0.498	0.598	13.	1.04E-10	14.	5.97E-09	0.004
X6	7.12	1.31E-07	0.593	0.456	0.548	12.	9.55E-11	13.	5.47E-09	0.003
X7	8.39	9.75E-08	0.442	0.340	0.408	9.	7.11E-11	9.	4.07E-09	0.002
DD	9.3	8.09E-08	0.366	0.282	0.338	7.	5.90E-11	8.	3.38E-09	0.002
X8	9.7	7.49E-08	0.339	0.261	0.313	7.	5.46E-11	7.	3.13E-09	0.003
10 MI	10	7.09E-08	0.321	0.247	0.296	6.	5.17E-11	7.	2.95E-09	0.003



TIME

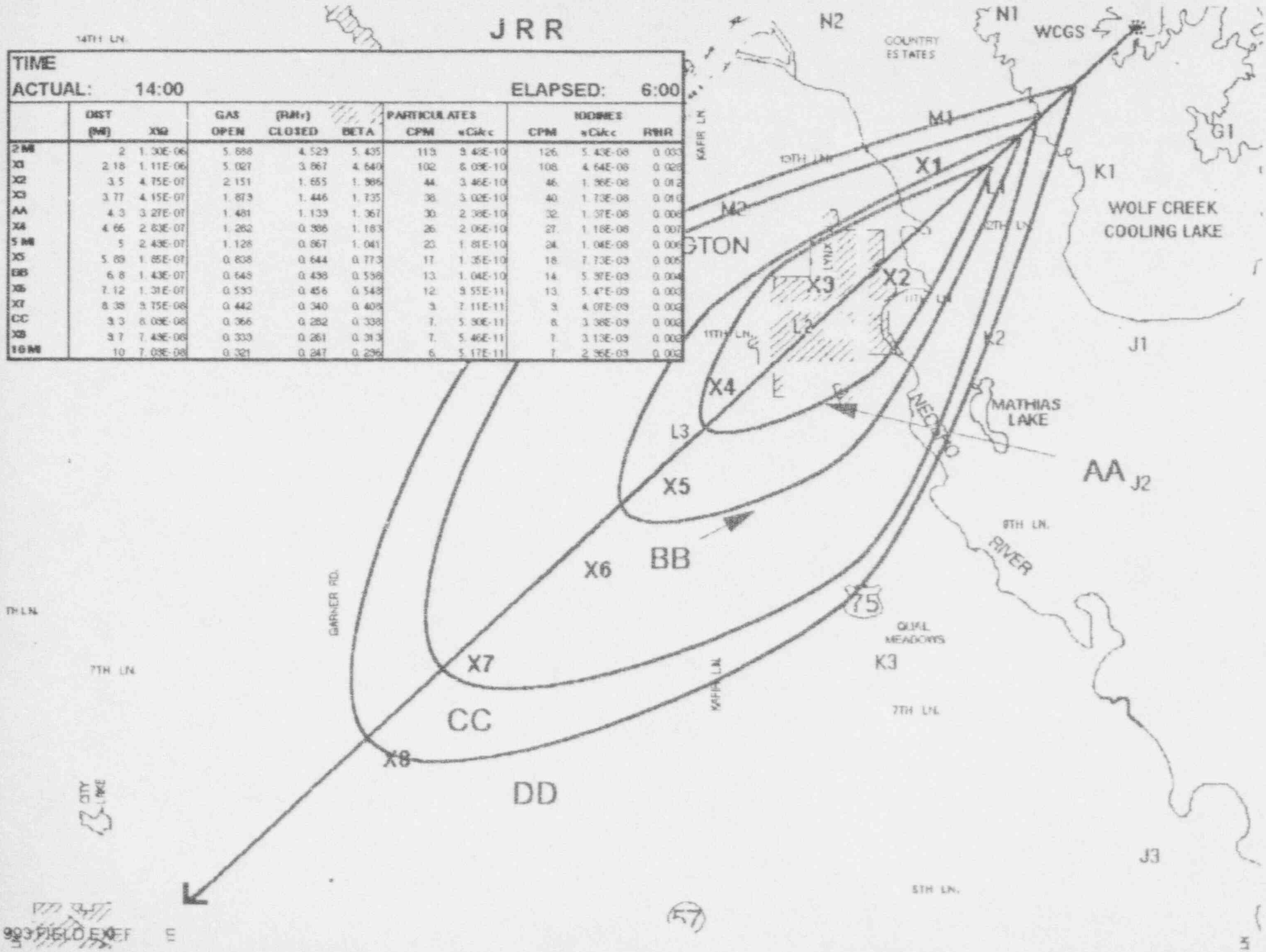
ACTUAL: 13:45 ELAPSED: 5:45

	DIST (MI)	X/Q	GAS (R/Hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	RMR
EAB	0.75	7.45E-06	33.740	25.954	31.145	683.	5.43E-09	722.	3.11E-07	0.187
AA	1.8	1.57E-06	7.110	5.470	6.563	144.	1.14E-09	152.	6.56E-08	0.039
2 MI	2	1.30E-06	5.888	4.529	5.435	119.	9.48E-10	126.	5.43E-08	0.033
X1	2.18	1.11E-06	5.027	3.867	4.640	102.	8.09E-10	108.	4.64E-08	0.028
X2	3.5	4.75E-07	2.151	1.655	1.986	44.	3.46E-10	46.	1.98E-08	0.012
X3	3.77	4.15E-07	1.879	1.446	1.735	38.	3.02E-10	40.	1.73E-08	0.010
BB	4.3	3.27E-07	1.481	1.139	1.367	30.	2.38E-10	32.	1.37E-08	0.008
X4	4.66	2.83E-07	1.282	0.986	1.183	26.	2.06E-10	27.	1.18E-08	0.007
5 MI	5	2.49E-07	1.128	0.867	1.041	23.	1.81E-10	24.	1.04E-08	0.006
X5	5.89	1.85E-07	0.838	0.644	0.773	17.	1.35E-10	18.	7.73E-09	0.005
CC	6.8	1.43E-07	0.648	0.498	0.598	13.	1.04E-10	14.	5.97E-09	0.004
X6	7.12	1.31E-07	0.593	0.456	0.548	12.	9.55E-11	13.	5.47E-09	0.003
X7	8.39	9.75E-08	0.442	0.340	0.408	9.	7.11E-11	9.	4.07E-09	0.002
DD	9.3	8.09E-08	0.366	0.282	0.338	7.	5.90E-11	8.	3.38E-09	0.002
X8	9.7	7.43E-08	0.339	0.261	0.313	7.	5.46E-11	7.	3.13E-09	0.002
10 MI	10	7.09E-08	0.321	0.247	0.296	6.	5.17E-11	7.	2.96E-09	0.002

TIME

ACTUAL: 14:00 ELAPSED: 6:00

	DIST (MI)	X/Q	GAS (R/Hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR
2 MI	2	1.30E-06	5.888	4.529	5.435	113.	9.48E-10	126.	5.43E-08	0.033
X1	2.18	1.11E-06	5.027	3.867	4.640	102.	8.09E-10	108.	4.64E-08	0.028
X2	3.5	4.75E-07	2.151	1.655	1.986	44.	3.46E-10	46.	1.98E-08	0.012
X3	3.77	4.15E-07	1.873	1.446	1.735	38.	3.02E-10	40.	1.73E-08	0.010
AA	4.3	3.27E-07	1.481	1.139	1.367	30.	2.38E-10	32.	1.37E-08	0.008
X4	4.66	2.83E-07	1.282	0.986	1.183	26.	2.06E-10	27.	1.18E-08	0.007
5 MI	5	2.43E-07	1.128	0.867	1.041	23.	1.81E-10	24.	1.04E-08	0.006
X5	5.89	1.85E-07	0.838	0.644	0.773	17.	1.35E-10	18.	7.73E-09	0.005
BB	6.8	1.43E-07	0.648	0.498	0.598	13.	1.04E-10	14.	5.97E-09	0.004
X6	7.12	1.31E-07	0.593	0.456	0.548	12.	9.55E-11	13.	5.47E-09	0.003
X7	8.38	9.75E-08	0.442	0.340	0.408	9.	7.11E-11	9.	4.07E-09	0.002
CC	9.3	8.09E-08	0.366	0.282	0.338	7.	5.90E-11	8.	3.38E-09	0.002
X8	9.7	7.49E-08	0.339	0.261	0.313	7.	5.46E-11	7.	3.13E-09	0.002
10 MI	10	7.09E-08	0.321	0.247	0.296	6.	5.17E-11	7.	2.96E-09	0.002

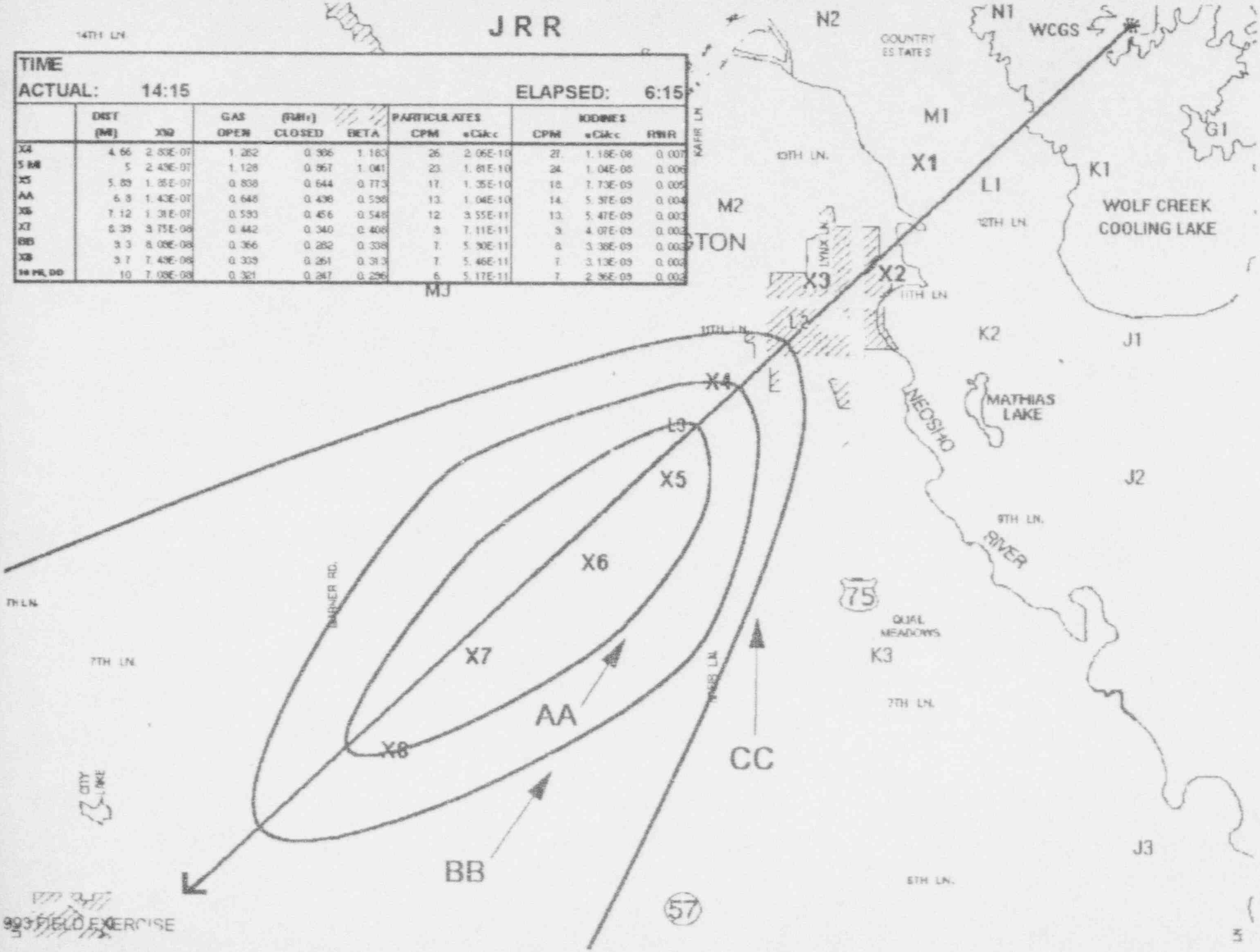


993 FIELD EXEF

TIME

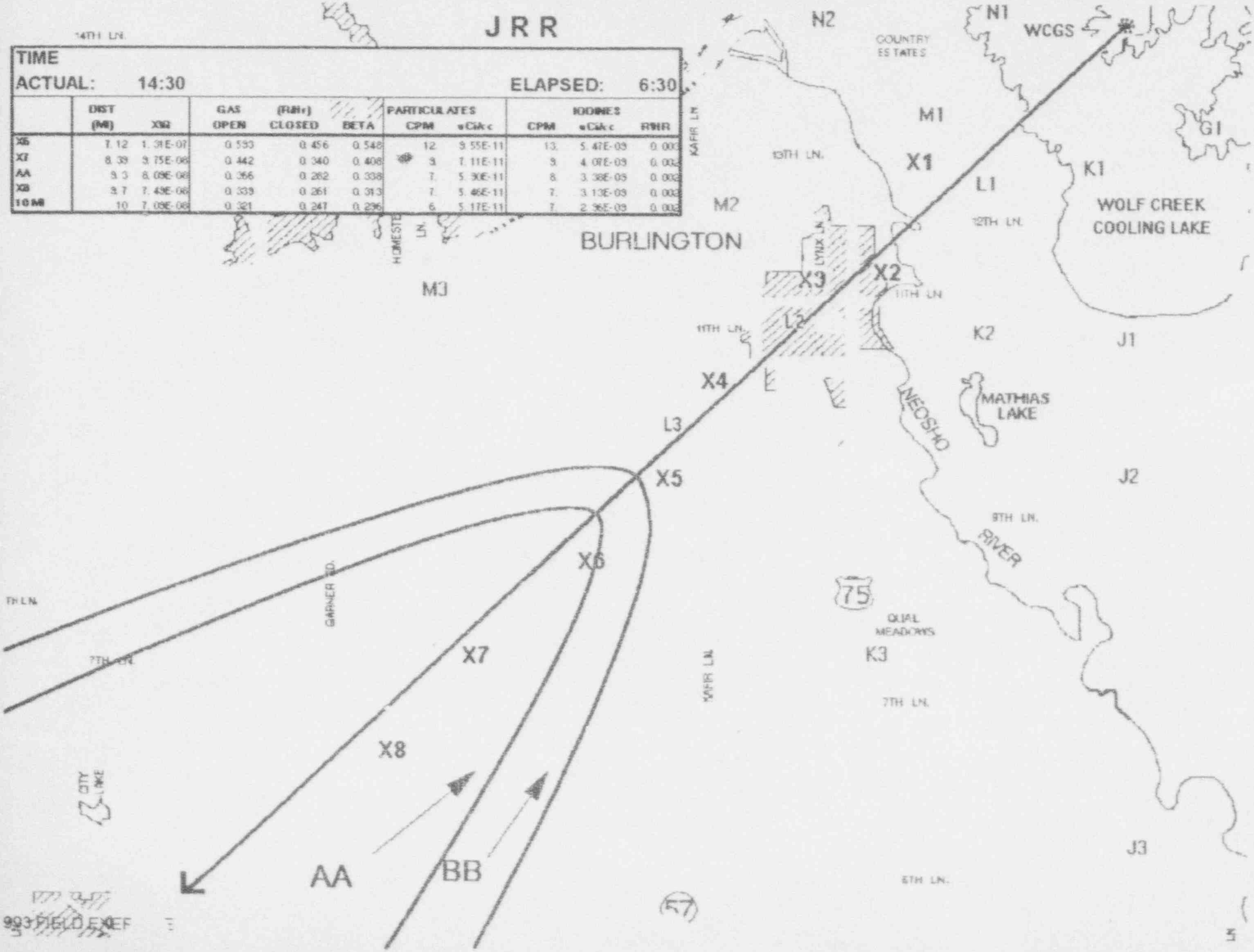
ACTUAL: 14:15 ELAPSED: 6:15

	DIST (MI)	X/Q	GAS (R/Hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR
X4	4.66	2.80E-07	1.282	0.986	1.183	26.	2.06E-10	27.	1.18E-08	0.007
5 MI	5	2.49E-07	1.128	0.867	1.041	23.	1.61E-10	24.	1.04E-08	0.006
X5	5.89	1.85E-07	0.838	0.644	0.773	17.	1.35E-10	18.	7.73E-09	0.005
AA	6.8	1.43E-07	0.648	0.498	0.598	13.	1.04E-10	14.	5.97E-09	0.004
X6	7.12	1.31E-07	0.593	0.456	0.548	12.	8.55E-11	13.	5.47E-09	0.003
X7	8.39	9.75E-08	0.442	0.340	0.408	9.	7.11E-11	9.	4.07E-09	0.002
BB	9.3	8.08E-08	0.366	0.282	0.338	7.	5.90E-11	8.	3.38E-09	0.002
X8	9.7	7.49E-08	0.339	0.261	0.313	7.	5.46E-11	7.	3.13E-09	0.002
10 MI, DD	10	7.08E-08	0.321	0.247	0.296	6.	5.17E-11	7.	2.36E-09	0.002



J R R

TIME										
ACTUAL: 14:30									ELAPSED:	6:30
	DIST (MI)	X/Q	GAS OPEN	(R/Hr) CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR
X6	7.12	1.31E-07	0.593	0.456	0.548	12	9.55E-11	13.	5.47E-09	0.003
X7	8.39	9.75E-08	0.442	0.340	0.408	9.	7.11E-11	9.	4.07E-09	0.002
AA	9.3	8.09E-08	0.366	0.282	0.338	7.	5.90E-11	8.	3.38E-09	0.002
X8	9.7	7.49E-08	0.339	0.261	0.313	7.	5.46E-11	7.	3.13E-09	0.002
10 MI	10	7.09E-08	0.321	0.247	0.296	6.	5.17E-11	7.	2.96E-09	0.002



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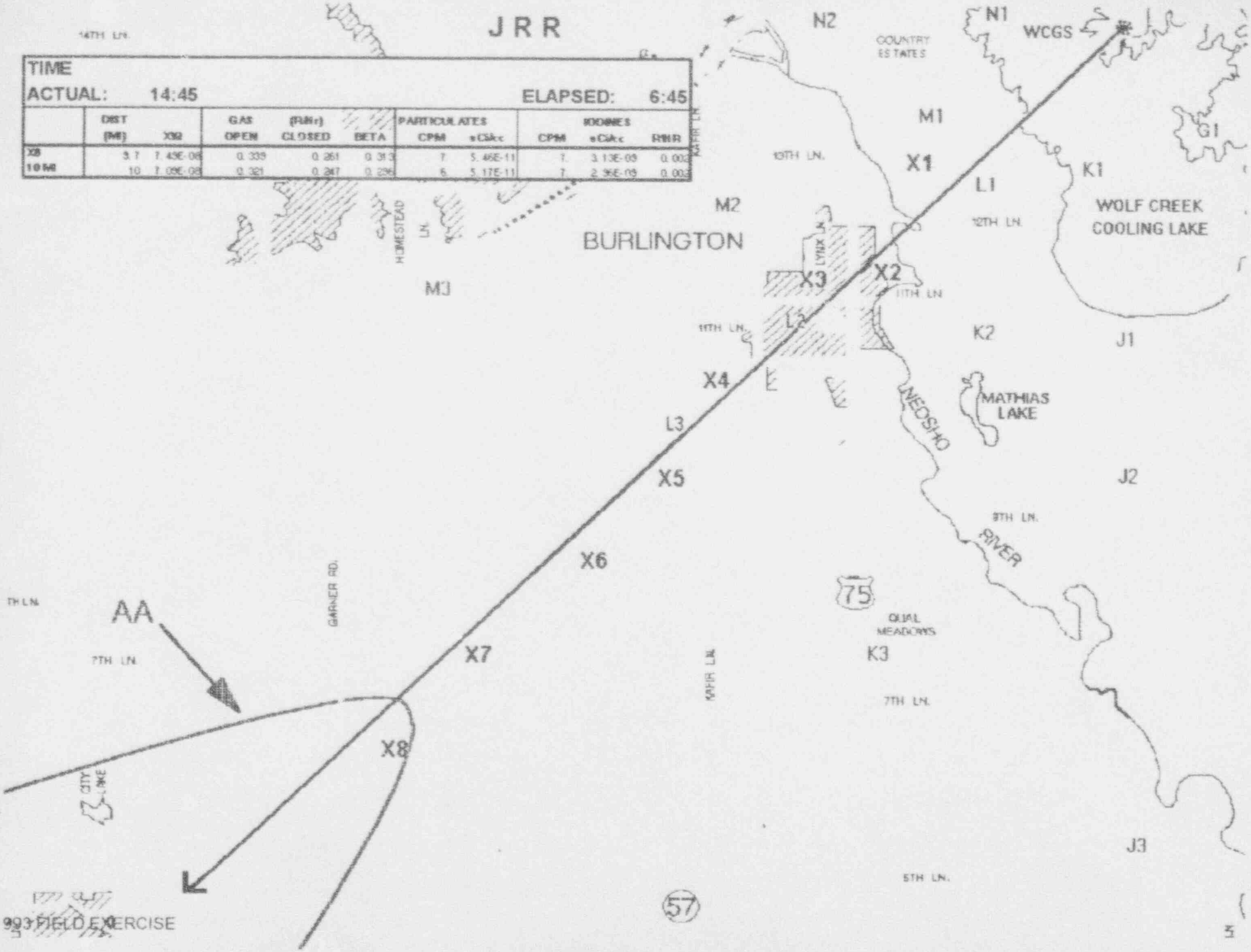
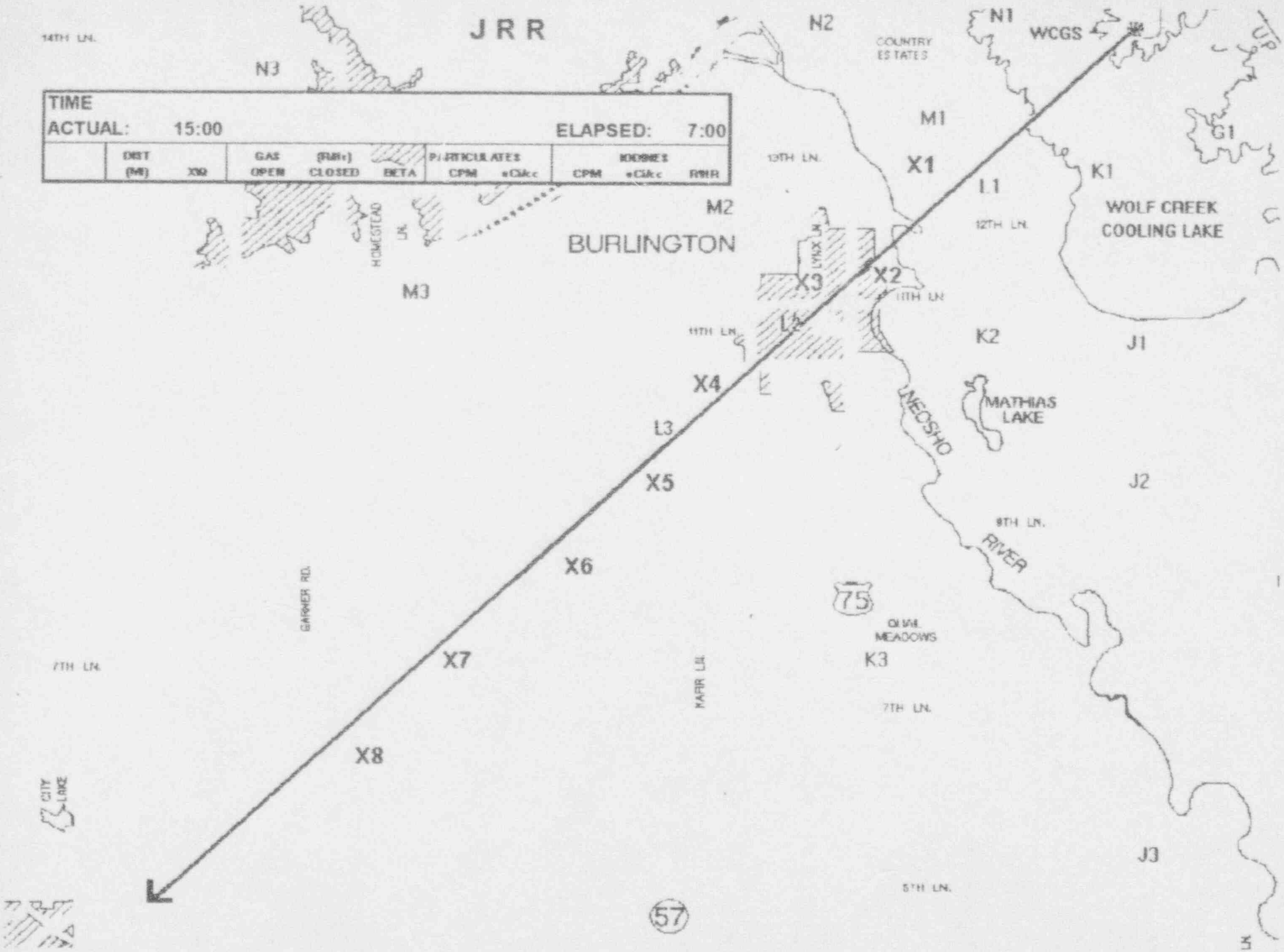
J R R

TIME										
ACTUAL: 14:45									ELAPSED:	6:45
	DIST (MI)	X/Q	GAS (R/Hr) OPEN	CLOSED	BETA	PARTICULATES CPM	uCi/cc	IODINES CPM	uCi/cc	R/HR
X8	3.7	7.43E-06	0.339	0.261	0.313	7.	5.46E-11	7.	3.13E-09	0.002
10 MI	10	7.09E-08	0.321	0.247	0.296	6.	5.17E-11	7.	2.96E-09	0.003





993 FIELD EXERCISE

SECTION 9.0

CONTROLLER ASSIGNMENTS AND INSTRUCTIONS

CONTROLLER ASSIGNMENTS

ASSIGNMENT	NAME
Lead Controller (3)	
CR (Simulator) Controller	
CR HP/Chemistry	
CR Communications	
CR Plant Teams (dispatched from computer room)	
Security	
TSC Lead Controller	
TSC Dose Assessment	
TSC Rad. Assessment	
TSC Engineering	
TSC Communications	
OSC Lead Controller	
OSC Health Physics	
OSC Onsite Teams (4)	
PASS Team	
Offsite Monitoring Teams and Joint Radiological Monitoring Teams (4)	
EOF Lead Controller	
EOF Engineering	
EOF Dose Assessment	
EOF Rad Team	
EOF Communications	
EOF State Dose Assessment	
EOF State Coordination	
Information Clearinghouse	
Media Release Center	
Rumor Control - Wichita	
Rumor Control - KCPL	

CONTROLLER ASSIGNMENTS (con't)

State EOC ______________________

____

State Forward Staging Area ______________________

Coffey County EOC ______________________

____

The Control Room Simulator and NPIS computer will be operated by Dale Moses, Dave Fehr, and Glenn Reeves.

CONTROLLER INSTRUCTIONS

Controller instructions define the types of interactions and conduct expected from controllers. The instructions included in the following subsection must be adhered to since controllers will at times have a direct input into the development of scenario activities and subsequently the success of the Exercise.

CONTROLLER INSTRUCTIONS

A. Controllers shall pre-position themselves in the appropriate emergency response facility no later than 30 minutes prior to the commencement of Exercise activities.

B. Controllers must comply with instructions from the Exercise Lead Controller.

C. Prior to the commencement of Exercise activities, controllers shall test telecommunications to ensure operable communication links to the Exercise Lead Controller.

D. Prior to the commencement of Exercise activities, controllers shall synchronize their watches through the Exercise Lead Controller to ensure the coordinated dispatch of time-related messages and data.

E. Controller messages, specifically scenario, onsite, offsite and public information messages, must be approved by the facility Lead Controller prior to issuance.

F. Special messages and messages designated as contingency must be approved by the Exercise Lead Controller prior to issuance.

G. Information regarding scenario events or data must only be provided upon request from the appropriate players.

H. Information regarding scenario events or data must not be provided prior to the times noted on the message or data sheets.

I. Controllers will ensure that players do not use radios in Area 5 of the plant.

J. Exercise objectives are considered confidential information and are not to be provided to Wolf Creek participants.

SECTION 10.0

EVALUATOR ASSIGNMENTS AND INSTRUCTIONS

EVALUATOR ASSIGNMENTS

ASSIGNMENT	NAME
Lead Evaluator	____________
CR (Simulator) Evaluator	____________
CR HP/Chemistry	____________
CR Communications	____________
CR Plant Teams (dispatched from computer room)	____________
Security	____________
TSC Lead Evaluator	____________
TSC Dose Assessment	____________
TSC Rad. Assessment	____________
TSC Engineering	____________
TSC Communications	____________
OSC Lead Evaluator	____________
OSC Health Physics	____________
OSC Onsite Teams (4)	____________
	____________
	____________
	____________
PASS Team	____________
Offsite Monitoring Teams and Joint Radiological Monitoring Teams (4)	____________
	____________
EOF Lead Evaluator	____________
EOF Engineering	____________
EOF Dose Assessment	____________
EOF Rad Team	____________
EOF Communications	____________
EOF State Dose Assessment	____________
EOF State Coordination	____________
Information Clearinghouse	____________
	____________
Media Release Center	____________
	____________
Rumor Control - Wichita	____________
Rumor Control - KCPL	____________
State EOC	____________
	____________
State Forward Staging Area	____________
Coffey County EOC	____________
	____________

EVALUATOR ASSIGNMENTS

County Road and Bridge Department
Waverly School
Coffey County Ambulance

Coffey County Hospital

Host County - Lyon

EVALUATOR ASSIGNMENTS (con't)

A. Evaluators shall pre-position themselves in the appropriate emergency response facility no later than 30 minutes prior to the commencement of Exercise activities.

B. Evaluators must comply with instructions from the Exercise Lead Evaluator.

C. Prior to the commencement of Exercise activities, evaluators shall synchronize their watches through the Exercise Lead Evaluator to ensure a coordinated accounting of Exercise events and player activities.

D. Interactions with the players must be held to a minimum by all evaluators.

E. All evaluators shall take detailed notes of player activities utilizing the blank evaluation log sheets. Each evaluator should carefully note the arrival and departure times for players, the times at which major activities or milestones occur, and any problem areas encountered.

F. Evaluation checklists for the applicable functional area should be completed by each evaluator. The completed checklists will be used to determine if the assigned objectives were satisfactorily demonstrated.

G. Toward the end of Exercise activities, each facility Lead Evaluator shall distribute the attendance sheet and collect player comments. Player comment forms are included in the facility Lead Evaluators' packets.

H. All facility Lead Evaluators shall turn in their evaluation logs and checklists to the Exercise Lead Evaluator at the controller/evaluator critique after the Exercise. The date for this critique will be ____________________________________.

I. All facility Lead Evaluators shall submit a formal critique to be completed following the termination of Exercise activities. The format of the critique will be as follows:

    1.) Summary of Events and Overall Evaluation

    2.) Timeline of Activities

    3.) List of Observations, Improvement Items and Deficiencies

J. The facility Lead Evaluators shall ensure that players' paperwork, logs, notification forms, etc. produced in the course of Exercise activities are turned into the Exercise Lead Evaluator during the controller/evaluator critique.

EVALUATOR INSTRUCTIONS

Evaluation checklists are included in the following subsection.

Checklists are categorized by facility or response function and then into specific objectives to be demonstrated by that facility or response function. The objectives provided in the checklists correspond to the objectives to be demonstrated as identified in the matrices in Section 2.0.

The Evaluation Summary (following the Evaluation Checklists) should list the most significant positive or negative items noted by the evaluator during the Exercise. The following definitions apply to the headings on the EVALUATION SUMMARY:

Deficiency:	A significant failure or inadequacy. It indicates that the level of emergency preparedness does not provide reasonable assurances that adequate protective measures can and will be taken in the event of a radiological emergency. These are addressed through KGP-1210, "Performance Improvement Request."
Weakness:	A failure or inadequacy in any WCNOC-related emergency planning procedure, program, implementation, or documentation. It indicates that the level of preparedness could have precluded effective implementation of the emergency plan in the event of an actual emergency. These are addressed through KGP-1210.
Improvement Item	An opportunity to enhance the Emergency Planning Program that, if not implemented, would not reduce the effectiveness of the Emergency Planning Program. These items are below the threshold of KGP-1210 and are tracked through the E-Plan Action Item Tracking System.
Good Practice:	An item identified during drills and exercises which should be continued to enhance the implementation of the Emergency Planning Program.

EVALUATION INSTRUCTIONS

EVALUATION SUMMARY

Evaluator:__________________
Assignment:__________________

(Use additional pages as required)

POTENTIAL DEFICIENCIES:

POTENTIAL WEAKNESSES:

POTENTIAL IMPROVEMENT ITEMS:

GOOD PRACTICES:

EVALUATION INSTRUCTIONS

Evaluator: ________________
Assignment: ________________

Control Room

NRC Objective	Yes	No	N/O	N/A
I.1. Accident detection and assessment				
a. Did the Shift Supervisor properly evaluate the emergency?	___	___	___	___
I.2. Emergency classification				
a. Was an NUE declared promptly?	___	___	___	___
b. Was an Alert declared promptly?	___	___	___	___
I.3. Notification of onsite and offsite emergency responders				
a. Did the CR Communicator complete the Immediate Notification form (Form EP 01-3.1-1) and call the appropriate personnel within applicable time limits (15 min. for State and County; ASAP and within 1 hour for NRC);				
- for the NUE?	___	___	___	___
- for the Alert?	___	___	___	___
b. Did CR personnel promptly sound the emergency alarm and announce via GAI-tronics :				
- for the NUE? (Form EP 01-1.0-1)	___	___	___	___
- for the Alert? (Form EP 01-1.0-2)	___	___	___	___
c. Were followup notifications made to the State and County every 30 minutes after the Immediate Notification?	___	___	___	___
I.4. Communications				
a. Were phones and GAI-tronics operable for making notifications or transmitting information?	___	___	___	___
b. Were radios operable if phones were inoperable?	___	___	___	___

EVALUATION INSTRUCTIONS

Control Room

NRC Objective	Yes	No	N/O	N/A
I.5. Radiological exposure control				
a. Was CR habitability established?	___	___	___	___
b. Did the Shift Supervisor authorize overexposure or recommend the use of KI to any WCNOC emergency worker?	___	___	___	___
c. Was this authorization/recommendation based on criteria in EPPs 01-9.1 and 01-9.3 and were the proper forms completed?	___	___	___	___
I.6. Protective action recommendations				
a. Did the Shift Supervisor include any protective action recommendation on the Immediate Notification Forms				
- for the NUE?	___	___	___	___
- for the Alert?	___	___	___	___
b. If a) is "yes", were the recommendations based on the criteria in EPP 01-10.1?	___	___	___	___
I.8. Shift staffing				
a. Was a complete Control Room shift complement available throughout the Exercise?	___	___	___	___

EVALUATION INSTRUCTIONS

Evaluator:________________
Assignment:________________

Security

NRC Objective	Yes	No	N/O	N/A
I.4 Communications				
a. Were the following types of communication operable during the drill?				
- Phones	___	___	___	___
- Radios	___	___	___	___
I.7. Staff Augmentation				
a. Did Security augment the Control Room staff during the emergency as required by Table 1.1-1 of the WCGS Plan?	___	___	___	___

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

Technical Support Center

NRC Objective	Yes	No	N/O	N/A
I.1. Accident detection and assessment				
a. When the TSC was activated was the DED aware of all significant events prior to that time?	___	___	___	___
b. Did the REC promptly request that two Offsite Monitoring Teams be dispatched?	___	___	___	___
c. Did the DED convene regular managers' meetings and then update the balance of the TSC staff on the event status?	___	___	___	___
d. Did the TSC personnel actively support the Control Room's efforts to identify the cause of the incident and mitigate it?	___	___	___	___
e. Did the TSC Engineering Team work with the EOF Engineering Team to determine short and long range solutions to the incident?	___	___	___	___
f. Were ERDC Teams requested for dispatch to problem areas promptly?	___	___	___	___
g. Were the status boards maintained regularly and accurately?	___	___	___	___
I.2. Emergency classification				
a. Was the Site Area Emergency promptly declared by the DED?	___	___	___	___

EVALUATION INSTRUCTIONS

Technical Support Center

NRC Objective	Yes	No	N/O	N/A
I.3. Notification of onsite and offsite emergency responders				
a. Did the TSC Communicator complete the Immediate Notification Form (Form EP 01-3.1-1) and call the appropriate personnel within applicable time constraints (15 min. - State, County; ASAP and within one hour - NRC)?	___	___	___	___
NOTE: NRC communications are performed by the ENS Communicator using Form 01-3.1-2.				
b. Did the CR sound the Plant Emergency Alarm and announce, via GAI-tronics, Form EP 01-1.0-3				
- for a Site Area Emergency?	___	___	___	___
c. Were followup notifications made to the State and County every 30 minutes after the Immediate Notification?	___	___	___	___
d. Were the State, County and NRC notified of TSC activation?	___	___	___	___
I.4. Communications				
a. Were the following types of communication operable during the drill:				
- ERO Phones	___	___	___	___
- ENS Phone	___	___	___	___
- HPN Phone	___	___	___	___
- PIC ringdown	___	___	___	___
- Fax machine	___	___	___	___
- Radios	___	___	___	___
I.5. Radiological exposure control				
a. Did the DED authorize overexposure or recommend the use of KI to any WCNOC emergency worker?	___	___	___	___

EVALUATION INSTRUCTIONS

Technical Support Center

NRC Objective	Yes	No	N/O	N/A
b. Was this authorization/recommendation based on criteria in EPPs 01-9.1 and 01-9.3 and were the proper forms completed?	___	___	___	___
c. Was TSC habitability established and verified at least every hour?	___	___	___	___
- Did the airlock doors remain closed?	___	___	___	___
I.6. Protective action recommendations				
a. Did the DED include any protective action recommendations on the Immediate Notification Forms for the SAE?	___	___	___	___
b. If a. is "yes", were these recommendations based on discussions with the REC and OEC?	___	___	___	___
c. Were PARs posted correctly on the status boards and the notification form?	___	___	___	___
d. Were the PARs followed up by the DED with the County/State as to the status of their implementation?	___	___	___	___
e. Were Offsite Monitoring Teams aware of the PARs?	___	___	___	___
I.7. Staff Augmentation				
a. Did the TSC augment the Control Room staff as required by Inspection Procedure 82205 and Table 1.1-1 of the WCGS Plan?	___	___	___	___

EVALUATION INSTRUCTIONS

Technical Support Center

NRC Objective	Yes	No	N/O	N/A
I.8. Shift staffing				
a. Was the TSC activated within 30 minutes after the Alert was classified?	___	___	___	___
b. Was a complete TSC shift complement available throughout the Exercise?	___	___	___	___

EVALUATION INSTRUCTIONS

Evaluator:____________________
Assignment:____________________

PASS/Onsite Survey/ERDC Team

NRC Objective	Yes	No	N/O	N/A
I.4. Communications				
a. Were the radios operable?	___	___	___	___
I.5. Radiological exposure control				
a. Was respiratory protection required for the Team's assignment?	___	___	___	___
b. If a. is "yes", was the proper equipment available and used?	___	___	___	___
c. Were Team members supplied with correct range dosimetry and TLDs?	___	___	___	___
d. If samples were obtained, were they handled in a way to minimize exposure?	___	___	___	___
I.7. Staff Augmentation				
a. Did the teams augment the Control Room staff as required by Table 1.1-1 of the WCGS Plan?	___	___	___	___

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

Operations Support Center

NRC Objective	Yes	No	N/O	N/A
I.1. Accident detection and assessment				
a. Were Onsite Survey Teams/ERDC Teams briefed on:				
- radiation/contamination levels	___	___	___	___
- route to and from work area	___	___	___	___
- dosimetry, PC, respirator requirements	___	___	___	___
- allowable doses or stay time	___	___	___	___
- air monitoring and radiological control requirements	___	___	___	___
- their team identification	___	___	___	___
I.4. Communications				
a. Were the Teams in radio, GAI-tronics, or phone contact with the TSC or OSC at all times?	___	___	___	___
b. Were all the radios in the OSC emergency cabinet operable?	___	___	___	___
c. Were phone communications available between the OSC Supervisor's office and the TSC?	___	___	___	___
I.5. Radiological exposure control				
a. Was OSC habitability established then verified every hour?	___	___	___	___
b. Were the Team members issued adequate dosimetry for the radiological conditions they might encounter?	___	___	___	___
c. Was KI recommended to be taken by any Team member?	___	___	___	___
d. If c. is "yes", was KI readily available and were the proper forms completed?	___	___	___	___

EVALUATION INSTRUCTIONS

Operations Support Center

NRC Objective	Yes	No	N/O	N/A
I.7. Staff Augmentation				
a. Did the OSC augment the Control Room staff as required by Table 1.1-1 of the WCGS Plan?	___	___	___	___
I.8. Shift staffing				
a. Was the OSC activated within 30 minutes post-classification of the Alert?	___	___	___	___
b. Were all OSC ERO positions staffed throughout the Exercise?	___	___	___	___

EVALUATION INSTRUCTIONS

Evaluator: ________________
Assignment: ________________

Offsite Monitoring Teams and Joint Radiological Monitoring Teams

NRC Objective	Yes	No	N/O	N/A
I.4. Communications				
a. Were the vehicle radios operable?	___	___	___	___
I.5. Radiological exposure control				
a. Was respiratory protection required for the Team's assignment?	___	___	___	___
b. If a. is "yes", was the proper equipment available and used?	___	___	___	___
c. Were Team members supplied with correct range dosimetry and TLDs? (0-500 mR and 0-5 R for WCNOC; 0-200 mR and 0-20 R for State/County)	___	___	___	___
d. Were Team members briefed on:				
- magnitude and composition of any actual or potential radiological releases	___	___	___	___
- source of leak	___	___	___	___
- expected duration of release	___	___	___	___
- projected or measured offsite dose rates	___	___	___	___
- current and projected meteorological conditions	___	___	___	___
- location to join Offsite Monitoring Team, if applicable	___	___	___	___
e. Did Team members keep their exposure ALARA by moving to low background areas for counting of samples?	___	___	___	___
f. Was dosimetry checked periodically?	___	___	___	___

EVALUATION INSTRUCTIONS

Offsite Monitoring Teams and
Joint Radiological Monitoring Teams

NRC Objective	Yes	No	N/O	N/A
g. Were survey instruments used properly?	___	___	___	___
h. Were personnel, equipment and vehicle checked for contamination?	___	___	___	___
i. Was decontamination performed properly?	___	___	___	___
j. Was dosimetry checked and the readings recorded when the Team returned to the EOF?	___	___	___	___
k. Was KI available if recommended, and were the proper forms completed?	___	___	___	___
I.7. Staff Augmentation				
a. Did the teams augment the Control Room staff as required by Table 1.1-1 of the WCGS Plan?	___	___	___	___

FEMA Objectives

(For Joint Radiological Monitoring Teams only)

The following FEMA objectives will also be demonstrated: 1, 4, 5, 6, 8, 14.

Evaluation checklists from FEMA REP-15 "Exercise Evaluation Manual" will be used.

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

Emergency Operations Facility

NRC Objective	Yes	No	N/O	N/A
I.1. Accident detection and assessment				
a. When the EOF was activated, was the DEM aware of all significant events prior to that time?	___	___	___	___
b. Did the RAM promptly request that 4 Joint Radiological Monitoring Teams (JRMTs) be formed?	___	___	___	___
c. Did the DEM convene regular managers' meetings and then update the balance of the EOF staff on the event status?	___	___	___	___
d. Were EOF personnel actively engaged with TSC personnel to mitigate the incident?	___	___	___	___
e. Did the EOF Engineering Team work with the TSC Engineering Team to determine short, mid- and long range solutions to the incident?	___	___	___	___
I.2 Emergency classification				
a. Was a General Emergency promptly declared by the DEM?	___	___	___	___
I.3. Notification of onsite and offsite emergency responders				
a. Did the EOF Communicator complete the Immediate Notification Form (Form EP 3.1-1) and call the appropriate personnel within applicable time constraints (15 min. - State, County; ASAP and within one hour? - NRC?)	___	___	___	___

NOTE: NRC communications are performed by the ENS Communicator using Form 01-3.1-2.

EVALUATION INSTRUCTIONS

Emergency Operations Facility

NRC Objective	Yes	No	N/O	N/A
b. Did the CR sound the Plant Emergency Alarm and announce, via GAI-tronics, Form EP 01-1.0-4				
- for a General Emergency?	___	___	___	___
c. Were followup notifications made to the State and County every 30 minutes?	___	___	___	___
d. Were the State, County and NRC notified of TSC activation?	___	___	___	___
I.4. Communications				
a. Were the following communication lines operable:				
- ERO phones	___	___	___	___
- ENS phone	___	___	___	___
- HPN phone	___	___	___	___
- PIC ringdown	___	___	___	___
- Fax machine	___	___	___	___
- Radios	___	___	___	___
I.5. Radiological exposure control				
a. Did the DEM authorize over-exposure for any WCNOC emergency worker?	___	___	___	___
b. Was this authorization based on criteria in EPPs 01-9.1 and 01-9.3?	___	___	___	___
c. Was EOF habitability established and verified at least every hour?	___	___	___	___
- Did the airlock doors remain closed?	___	___	___	___
d. Was dosimetry positioned throughout the EOF?	___	___	___	___
e. Was dosimetry checked periodically?	___	___	___	___

EVALUATION INSTRUCTIONS

Emergency Operations Facility

NRC Objective	Yes	No	N/O	N/A
I.6. Protective action recommendations				
a. Did the DEM include new protective action recommendations on the Immediate Notification Form for the GE?	___	___	___	___
b. If a. is "yes", were these recommendations based on:				
- dose calculations,	___	___	___	___
- discussions with the RAM and TRM, or	___	___	___	___
- automatic PARs in EPP 01-10.1	___	___	___	___
c. Were PARs posted correctly on the status boards and the notification form?	___	___	___	___
d. Were PARs discussed with the State RAM and/or the KDEP representative?	___	___	___	___
e. Were changes in PARs transmitted to the State and County within 15 minutes of being made?	___	___	___	___
f. Was implementation of the PARs followed up by the DEM as to the status of their implementation?	___	___	___	___
g. Were JRMTs aware of the current PARs?	___	___	___	___
I.7. Staff Augmentation				
a. Did the EOF augment the Control Room staff as required by Table 1.1-1 of the WCGS Plan?	___	___	___	___
b. Was the WCNOC Representative at the County dispatched to the County EOC?	___	___	___	___

EVALUATION INSTRUCTIONS

Emergency Operations Facility

NRC Objective	Yes	No	N/O	N/A
I.8. Shift staffing				
a. Was the EOF activated within 90 minutes after the Site Area Emergency classification?	___	___	___	___
b. Did the EOFC have the EOF ready for activation within 60 minutes?	___	___	___	___
c. Was a complete EOF shift complement available throughout the exercise?	___	___	___	___

EVALUATION INSTRUCTIONS

Evaluator:________________
Assignment:________________

Information Clearinghouse (IC)
and Media Release Center (MRC)

NRC Objective	Yes	No	N/O	N/A
I.4. Communications				
a.) Were the following communication lines operable:				
- Facility phones?	___	___	___	___
- PIO Ringdown?	___	___	___	___
- Fax machine?	___	___	___	___
- Computer?	___	___	___	___
- Conference call to Wichita, Kansas City?	___	___	___	___
- Media phones?	___	___	___	___
b.) Were any wrong numbers found in the RETD?	___	___	___	___
I.7. Staff augmentation				
a.) Was the IC/MRC activated quickly (<45 minutes after arrival) after players were allowed in?	___	___	___	___
I.8. Shift staffing				
a.) Was a full complement of IC/MRC players present in the facilities?	___	___	___	___

	Yes	No	N/O	N/A
II.1. Activation of Emergency News Center (Joint Information Center)				
a.) Were press statements reviewed by WCNOC, State and County PIOs (if available) prior to their issuance?	___	___	___	___
b.) Were press statements approved by the DED/DEM if required?	___	___	___	___

EVALUATION INSTRUCTIONS

Evaluator:____________________
Assignment:____________________

Information Clearinghouse (IC)
and Media Release Center (MRC)

NRC Objective

	Yes	No	N/O	N/A
b.) Was up-to-date, accurate information available from the EOF?	___	___	___	___
c.) Were news conferences well organized?	___	___	___	___
d.) Were unanswered questions at the news conferences followed up with a response at a later news conference?	___	___	___	___
e.) Was accurate, up-to-date information presented during the news conference by:				
- the WC Spokesperson?	___	___	___	___
- the State PIO? (if available)	___	___	___	___
the County PIO? (if available)	___	___	___	___
II.2. Rumor Control				
1.) Did the WCPIO refute or verify rumors quickly?	___	___	___	___
2.) Were these rumors then passed on at the news conferences?	___	___	___	___
3.) Were the Wichita Information Team Supervisor and the KCP&L Rumor Control Coordinator informed of the status of the rumor?	___	___	___	___

FEMA Objectives

The following FEMA objectives will also be demonstrated: 1,2, 4, 12, 13 (shift change for IC State and County PIOs only).

Evaluation checklists from FEMA REP-15 "Exercise Evaluation Manual" will be used.

EVALUATION INSTRUCTIONS

Evaluator:________________
Assignment:________________

Wichita Office - Rumor Control
(Media Inquiry
Phone Teams
Public Concern)

NRC Objective	Yes	No	N/O	N/A
I.4. Communications				
a.) Were the following communication lines operable during the Exercise:				
- Phones?	___	___	___	__
- Fax machine?	___	___	___	__
- Computer?	___	___	___	__
I.7. Staff augmentation				
a.) Did the Wichita Rumor Control staff augment the plant emergency staff?	___	___	___	__
I.8. Shift Staffing				
a.) Were Wichita Rumor Control personnel notified within 30 minutes of the Alert classification?				
b.) Were all Rumor Control positions staffed throughout the Exercise?	___	___	___	__

	Yes	No	N/O	N/A
II.2. Rumor Control				
a.) Did the Rumor Control personnel log in calls or broadcasts on the proper form?	___	___	___	__
b.) Were questions answered according to the information available in the EPIG and released by the WCPIO?	___	___	___	__
c.) Did the Phone Team return any calls to impart further information, if requested?	___	___	___	__

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

Wichita Office - Rumor Control
(Media Inquiry
Phone Teams
Public Concern)

NRC Objective

	Yes	No	N/O	N/A
d.) Were any rumors passed on to the Wichita Information Team Supervisor or Manager?	___	___	___	__
e.) Did the WIT Supervisor or Manager pass the rumors on to the WCPIO in the Information Clearinghouse?	___	___	___	__
f.) Was the WIT Manager or Supervisor informed by the WCPIO as to the status of the rumor?	___	___	___	__
g.) Were the Rumor Control personnel aware of the status of the rumor?	___	___	___	__
h.) Did the Information Team Assistant ensure approved news statements were available in a timely fashion?	___	___	___	__
i.) Did the Information Team Assistant send news statements to the INPO network?	___	___	___	__

FEMA Objectives

The following FEMA objective will also be demonstrated: 13.

The Evaluation checklist from the FEMA REP-15 "Exercise Evaluation Manual" will be used.

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

KCP&L General Office - Rumor Control

NRC Objective	Yes	No	N/O	N/A
I.4. Communications				
a.) Were the following communication lines operable during the Exercise:				
- Phones?	___	___	___	___
- Fax machine?	___	___	___	___
- Computer?	___	___	___	___
- News conference telecon?	___	___	___	___
I.7. Staff augmentation				
a.) Did the KCP&L Rumor Control staff augment the plant emergency staff?	___	___	___	___
I.8. Shift Staffing				
a.) Were the KCP&L Rumor Control personnel notified within 30 minutes of the Alert classification?	___	___	___	___
b.) Were all Rumor Control positions staffed throughout the Exercise?	___	___	___	___

II.2 Rumor Control				
a.) Did the Rumor Control Coordinator (RCC) log any rumors and pass them on to the WCPIO in the IC?	___	___	___	___
b.) Was the RCC informed by the WCPIO as to the status of the rumor?	___	___	___	___
c.) Were the Media Monitoring Teams aware of the status of the rumor?	___	___	___	___

EVALUATION INSTRUCTIONS

Evaluator:________________
Assignment:________________

Emergency Operations Facility
Dose Assessment & Field Team Coordinator

FEMA Objective

The following FEMA objectives will be demonstrated: 1, 2, 3, 4, 5, 7, 9, 14.

Evaluation checklists from FEMA REP-15 "Exercise Evaluation Manual" will be used.

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

Emergency Operations Facility
State Coordination

FEMA Objective

The following FEMA objectives will be demonstrated: 1, 2, 3, 4, 5, 14, 30.

Evaluation checklists from FEMA REP-15 "Exercise Evaluation Manual" will be used.

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

State EOC

FEMA Objective

The following FEMA objectives will be demonstrated: 1, 2, 3, 4, 9, 10, 11, 17.

Evaluation checklists from FEMA REP-15 "Exercise Evaluation Manual" will be used.

EVALUATION INSTRUCTIONS

Evaluator:____________________
Assignment:____________________

State Forward Staging Area

FEMA Objective

The following FEMA objectives will be demonstrated:
1, 4, 5, 14, 17, 30 (for Kansas Wildlife & Parks individual only)

Evaluation checklists from FEMA REP-15 "Exercise Evaluation Manual" will be used.

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

Coffey County EOC

FEMA Objective

The following FEMA objectives will be demonstrated:
1, 2, 3, 4, 5, 10, 11, 14, 15, 16, 17, 31.

Evaluation checklists from FEMA REP-15 "Exercise Evaluation Manual" will be used.

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

County Road and Bridge Department

FEMA Objective

The following FEMA objectives will be demonstrated:
1, 2, 3, 4, 5, 14, 15, 17.

Evaluation checklists from FEMA REP-15 "Exercise Evaluation Manual" will be used.

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

Waverly School

FEMA Objective

The following FEMA objectives will be demonstrated: 4, 5, 14, 16.

Evaluation checklists from FEMA REP-15 "Exercise Evaluation Manual" will be used.

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

Coffey County Ambulance

FEMA Objective

The following FEMA objectives will be demonstrated: 5, 14, 20.

Evaluation checklists from FEMA REP-15 "Exercise Evaluation Manual" will be used.

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

Coffey County Hospital

FEMA Objective

The following FEMA objectives will be demonstrated: 5, 14, 21.

Evaluation checklists from FEMA REP-15 "Exercise Evaluation Manual" will be used.

EVALUATION INSTRUCTIONS

Evaluator:__________________
Assignment:__________________

Host County - Lyon

FEMA Objective

The following FEMA objectives will be demonstrated:
1, 3, 4, 5, 18, 19, 22, 30.

Evaluation checklists from FEMA REP-15 "Exercise Evaluation Manual" will be used.

SUMMARY OF FEMA AREAS REQUIRING CORRECTIVE ACTION (ARCA)

State Emergency Operations Center

1. The State EOC failed to notify the FEMA Regional Office of any emergency classification levels, protective actions, or status updates in accordance with state SOPs. (NUREG-0654, E.1 and E.2). This ARCA will be addressed through the demonstration of Objective 1.

Dose Assessment/Field Team Coordination

2. The second shift personnel arrived at the EOF without direct-reading dosimeters. As the EOF is located near the plant site, the second shift staff traversed the plume EPZ to arrive at the facility without the ability to determine the dose to which they may have been exposed. (NUREG-0654, K.3.a). This ARCA will be addressed through the demonstration of Objective 5.

3. The plume boundries were not accurately defined because the field monitoring teams were not properly directed. (NUREG-0654, I.10). This ARCA will be addressed through the demonstration of Objective 3.

Field Monitoring Teams

4. County members were not dispatched as part of the Joint Radiological Monitoring Teams as required by the plan. (NUREG-0654, I.8). This ARCA will be addressed through the demonstration of Objectives 6 and 8.

Media Release Center

5. The State PIO failed to include one subzone (E-2) in the description of the evacuated area during one media briefing. However, this subzone had been appropriately included in the EBS message released to the public. (NUREG-0654, G.3.a). This ARCA will be addressed through the demonstration of Objective 12.

Forward Staging Area

6. The Kansas Department of Wildlife and Parks did not conduct a shift change in accordance with the plans. This ARCA will be addressed through the demonstration of Objective 30.

Coffey County Hospital

7. The ambulance gurney on which the patient was transported and the covered floor area of the hospital across which the gurney traversed were not checked for contamination prior to releasing the ambulance crew. The crew was subsequently allowed to retrace their steps over the unmonitored area. (NUREG-0654, L.1; GM MS-1; FEMA-REP-14; FEMA-REP-15). This ARCA will be addressed through the demonstration of Objective 21.

8. Hospital staff monitoring the ambulance did not cover the survey meter (Bicron) probe with plastic. (NURET-0654, L.1; GM MS-1; FEMA-REP-14). This ARCA will be addressed through the demonstration of Objective 21.

9. The hospital staff did not use the portable survey instrument (CD V-700) earphones during monitoring. (NUREG-0654, L.1; GM MS-1; FEMA-REP-14; FEMA-REP-15). This ARCA will be addressed through the demonstration of Objective 21.

Coffey County Ambulance

10. One crew member touched the contaminated ground and patient's clothing prior to utilizing clean equipment, such as bandages and the blanket to wrap the patient. (NUREG-0654, L.1; GM MS-1; FEMA-REP-14). This ARCA will be addressed through the demonstration of Objective 20.

SUMMARY OF NRC WEAKNESSES

1. Weakness 482/9119-03: failure of the licensee to establish and maintain habitability in the emergency response facilities. Parts of this weakness were closed during the 1992 Exercise; however, the TSC airlock door did not remain closed, and so the weakness remains open pending further review of the TSC habitability procedure. This weakness will be addressed by the demonstration of Objective I(5).

2. Weakness 482/9214-01: delays in making initial notifications to the State and county and in activating the group pagers. Subsequent late activation of the TSC and OSC was also part of this weakness. This weakness cannot be closed until the next unannounced Exercise, tentatively scheduled for 1997.

3. Weakness 482/9213-01: the emergency classification of accident conditions was identified as a weakness during the July 1992 simulator walkthroughs. This weakness will be addressed by the demonstration of Objective I(2).

4. Weakness 482/9213-02: failure of licensee to make accurate and timely notifications and protective action recommendations to offsite authorities (identified during July 1992 simulator walkthroughs). This weakness will be addressed by the demonstration of Objectives I(3) and I(6).

5. Weakness 482/9213-03: failure of the dose assessment procedure to provide guidance on obtaining accurate integrated dose projections based on prior release conditions (identified during July 1992 simulator walkthroughs). The procedure has been revised, and this weakness will also be addressed by the demonstration of Objective I(1).