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10 CFR 50.59
10 CFR 50.90

July 31, 1990

U. S. NUCLEAR REGULATORY COMMISSION
Document Control Desk
Mail Station P1-137
Washington, D. C. 20555

Gentlemen:

DOCKETS 50-266 AND 50-301
TECHNICAL SPECIFICATION CHANGE REQUEST 140
STEAM GENERATOR OVERFILL PROTECTION
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

In accordance with the requirements of 10 CFR 50.59(c), 50.90, and 50.4, Wisconsin Electric Power Company (Licensee) hereby requests amendments to Facility Operating Licenses DPR-24 and DPR-27 for Point Beach Nuclear Plant, Units 1 and 2 respectively, to incorporate a change in the plant Technical Specifications. This proposed change will revise Table 15.4.1-2, Item 11, to include a test of the logic for high steam generator level (overfill protection).

We are proposing to include the requirement to test the logic for high steam generator level (steam generator overfill protection system) as part of Item 11, Table 15.4.1-1. In our March 20, 1990 response to Generic Letter 89-19, "Safety Implications of Control Systems in LWR Nuclear Power Plants", we committed to include this requirement. The bistables for the overfill protection system are already functionally tested monthly; the steam generator level transmitters are calibrated each refueling; and the feedwater isolation valves are tested each refueling. This proposed change will make the surveillances of the overfill protection system a requirement of the Technical Specification.

As required by 10 CFR 50.91(a), we have evaluated these changes in accordance with the standards specified in 10 CFR 50.92 to determine if the proposed changes constitute significant hazards considerations. A proposed amendment involves no significant hazards consideration if operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident NRC

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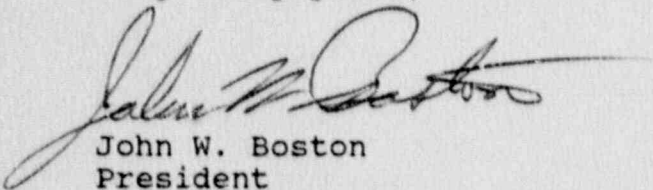
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previously evaluated, (2) create the possibility of a new or different kind of accident from any accident previously evaluated, or (3) involve a significant reduction in a margin of safety.

The proposed change to Table 15.4.1-1, Item 11, constitutes an additional control not presently included in the Technical Specifications. Since this testing is already conducted, adding the requirement to the Technical Specifications will not affect the probability or consequences of any accident nor will it create any new or different accident. By adding the requirement to the Technical Specifications, we are providing additional assurance that the surveillance testing will be conducted and, therefore, will not reduce any margin of safety. In addition, this is an example of a change not likely to involve a significant hazards consideration as provided at 51 FR 7751. We have, therefore, concluded that this proposed change does not result in a significant hazards consideration.

Please contact us if you have any questions concerning this request.

Very truly yours,

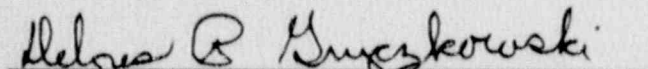


John W. Boston
President

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Copies to NRC Regional Administrator, Region III
NRC Resident Inspector

Subscribed and sworn to before me
this 31 day of July, 1990.



Notary Public, State of Wisconsin

My Commission expires 5-22-94.