

ENCLOSURE 4

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
NRC DOCKET NOS. 50-325 & 50-324
OPERATING LICENSE NOS. DPR-71 & DPR-62
REQUEST FOR LICENSE AMENDMENT
NUMAC STEAM LEAK DETECTION EQUIPMENT LICENSE AMENDMENT
CORRECTED PAGES

PAGE CHANGE INSTRUCTIONS

UNIT 1

<u>Removed Page</u>	<u>Inserted Page</u>
3/4 3-29	3/4 3-29

UNIT 2

<u>Removed Page</u>	<u>Inserted Page</u>
3/4 3-29	3/4 3-29

ENCLOSURE 5

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CORRECTED PAGES

TECHNICAL SPECIFICATION PAGE - UNIT 1

TABLE 4.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
4. <u>CORE STANDBY COOLING SYSTEMS ISOLATION</u>				
a. High Pressure Coolant Injection System Isolation				
1. HPCI Steam Line Flow - High Transmitter: Trip Logic:	NA ^(a) D	NA M	R ^(b) M	1, 2, 3 1, 2, 3
2. HPCI Steam Line High Flow Time Delay Relay	NA	R	R	1, 2, 3
3. HPCI Steam Supply Pressure - Low	NA	M	R	1, 2, 3
4. HPCI Steam Line Tunnel Temperature - High	NA	SA	R	1, 2, 3
5. Bus Power Monitor	NA	R	NA	1, 2, 3
6. HPCI Turbine Exhaust Diaphragm Pressure - High	NA	M	Q	1, 2, 3
7. HPCI Steam Line Ambient Temperature - High	NA	SA	R	1, 2, 3
8. HPCI Steam Line Area Δ Temperature - High	NA	SA	R	1, 2, 3
9. HPCI Equipment Area Temperature - High	NA	SA	R	1, 2, 3
10. Drywell Pressure - High Transmitter: Trip Logic:	NA ^(a) D	NA M	R ^(b) M	1, 2, 3 1, 2, 3

ENCLOSURE 6

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
NRC DOCKET NOS. 50-325 & 50-324
OPERATING LICENSE NOS. DPR-71 & DPR-62
REQUEST FOR LICENSE AMENDMENT
NUMAC STEAM LEAK DETECTION EQUIPMENT LICENSE AMENDMENT
CORRECTED PAGES

TECHNICAL SPECIFICATION PAGE - UNIT 2

TABLE 4.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
4. <u>CORE STANDBY COOLING SYSTEMS ISOLATION</u>				
a. High Pressure Coolant Injection System Isolation				
1. HPCI Steam Line Flow - High Transmitter: Trip Logic:	NA ^(a) D	NA M	R ^(b) M	1, 2, 3 1, 2, 3
2. HPCI Steam Line Flow - High Time Delay Relay	NA	R	R	1, 2, 3
3. HPCI Steam Supply Pressure - Low	NA	M	R	1, 2, 3
4. HPCI Steam Line Tunnel Temperature - High	NA	SA	R	1, 2, 3
5. Bus Power Monitor	NA	R	NA	1, 2, 3
6. HPCI Turbine Exhaust Diaphragm Pressure - High	NA	M	Q	1, 2, 3
7. HPCI Steam Line Ambient Temperature - High	NA	SA	R	1, 2, 3
8. HPCI Steam Line Area Δ Temperature - High	NA	SA	R	1, 2, 3
9. HPCI Equipment Area Temperature - High	NA	SA	R	1, 2, 3
10. Drywell Pressure - High Transmitter: Trip Logic:	NA ^(a) D	NA M	R ^(b) M	1, 2, 3 1, 2, 3



Carolina Power & Light Company

P.O. Box 10429
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ROY A. ANDERSON
Vice President
Brunswick Nuclear Plant

SERIAL: BSEP-93-0185
10 CFR 50.90
TSC 93TSB07

United States Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325 & 50-324/LICENSE NOS. DPR-71 & DPR-62
REQUEST FOR LICENSE AMENDMENT
NUMAC STEAM LEAK DETECTION EQUIPMENT LICENSE AMENDMENT - CORRECTED
PAGES

Gentlemen:

In accordance with the Code of Federal Regulations, Title 10, Parts 50.90 and 2.101, Carolina Power & Light Company (CP&L) hereby requests a revision to Appendix A of Operating Licenses DPR-71 and DPR-62 (the Technical Specifications) for the Brunswick Steam Electric Plant (BSEP), Units 1 and 2.

The proposed change corrects Technical Specification pages issued for Amendment 166 for Brunswick Unit 1 and Amendment 197 for Brunswick Unit 2, regarding NUMAC Steam Leak Detection Equipment. Specifically, on page 3/4 3-29 for each unit, the Channel Calibration frequency of Item 4.a.4, HPCI Steam Line Tunnel Temperature - High, was inadvertently left as quarterly (Q) rather than being revised to refuel (R). The text of CP&L's September 14, 1992 license amendment request and the NRC's safety evaluation for Amendments 166 and 197, dated October 14, 1993, addressed the frequency change from quarterly to refuel for this item. However, the revised pages provided with the September 14, 1992 license amendment request and the typed pages provided on September 21, 1993 did not reflect the requested change.

Enclosure 1 provides a detailed description of the proposed change and the basis for the change.

Enclosure 2 details, in accordance with 10 CFR 50.91(a), the basis for the Company's determination that the proposed change does not involve a significant hazards consideration.

Enclosure 3 provides an environmental evaluation which demonstrates that the proposed amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22(b), no environmental assessment needs to be prepared in connection with issuance of the amendment.

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Enclosure 4 provides page change instructions for incorporating the proposed revisions.

Enclosure 5 provides the typed Technical Specification page for Unit 1.

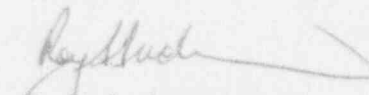
Enclosure 6 provides the typed Technical Specification page for Unit 2.

Carolina Power & Light Company is providing, in accordance with 10 CFR 50.91(b), Mr. Dayne H. Brown of the State of North Carolina with a copy of the proposed license amendment.

Carolina Power & Light Company requests that these corrected pages be included with the next license amendment scheduled to be issued, which the Company understands will be the Core Alterations amendments (NRC TAC Nos. M85162 & M85163). These revised Technical Specification pages should be effective as of the date of issuance of the amendment and to be implemented within 60 days for Unit 1 and to be effective and implemented upon completion of refueling outage 10 (B211R1) for Unit 2.

Please refer any questions regarding this submittal to Mr. W. Levis at (910) 457-2404.

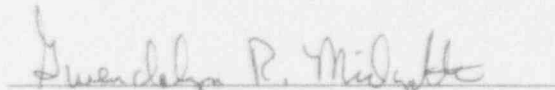
Very truly yours,


R. A. Anderson

MAT/mat (930185.000)

Enclosures

R. A. Anderson, having been first duly sworn, did depose and say that the information contained herein is true and correct to the best of his information, knowledge and belief; and the sources of his information are officers, employees, contractors, and agents of Carolina Power & Light Company.


Notary (Seal)

My commission expires: August 12, 1996

cc: Mr. Dayne H. Brown, State of North Carolina
Mr. S. D. Ebnetter, NRC Region II Administrator
Mr. P. D. Milano, NRC/NRR Senior Project Manager - BNP
Mr. R. L. Prevatte, NRC Senior Resident Inspector - BNP

ENCLOSURE 1

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
NRC DOCKET NOS. 50-325 & 50-324
OPERATING LICENSE NOS. DPR-71 & DPR-62
REQUEST FOR LICENSE AMENDMENT
NUMAC STEAM LEAK DETECTION EQUIPMENT LICENSE AMENDMENT
CORRECTED PAGES

BASIS FOR CHANGE REQUEST

Background:

The proposed change corrects Technical Specification pages issued for Amendment 166 for Brunswick Unit 1 and Amendment 197 for Brunswick Unit 2, regarding NUMAC Steam Leak Detection Equipment. Specifically, on page 3/4 3-29 for each unit, the Channel Calibration frequency of Item 4.a.4, HPCI Steam Line Tunnel Temperature - High, was inadvertently left as quarterly (Q) rather than being revised to refuel (R). The text of CP&L's September 14, 1992 license amendment request and the NRC's safety evaluation for Amendments 166 and 197, dated October 14, 1993, addressed the frequency change from quarterly to refuel for this item. However, the revised pages provided with the September 14, 1992 license amendment request and the typed pages provided on September 21, 1993 did not reflect the requested change.

Proposed Change:

The Channel Calibration frequency of Item 4.a.4 of Table 4.3.2-1, HPCI Steam Line Tunnel Temperature - High, is being revised from quarterly (Q) to refuel (R).

Basis:

Carolina Power & Light Company has reviewed the requested correction and determined that it is purely administrative in nature. The refuel Channel Calibration frequency for Item 4.a.4 of Table 4.3.2-1 was reviewed and approved by the NRC in the safety evaluation for Amendments 166 and 197 dated October 14, 1993. The proposed change merely revises the Channel Calibration frequency to properly reflect what has been reviewed and approved by the NRC.

ENCLOSURE 2

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
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10 CFR 50.92 EVALUATION

The Commission has provided standards in 10 CFR 50.92(c) for determining whether a significant hazards consideration exists. A proposed amendment to an operating license for a facility involves no significant hazards consideration if operation of the facility in accordance with the proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated, (2) create the possibility of a new or different kind of accident from any accident previously evaluated, or (3) involve a significant reduction in a margin of safety. Pursuant to 10 CFR 50.91(a)(1), Carolina Power & Light Company has reviewed this proposed license amendment request and determined that its adoption would not involve a significant hazards consideration. The basis for this determination are as follows:

Proposed Change:

The Channel Calibration frequency of Item 4.a.4 of Table 4.3.2-1, HPCI Steam Line Tunnel Temperature - High, is being revised from quarterly (Q) to refuel (R).

Basis:

The change does not involve a significant hazards consideration for the following reasons:

1. The proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated. The proposed change corrects Technical Specification pages issued for Amendment 166 for Brunswick Unit 1 and Amendment 197 for Brunswick Unit 2, regarding NUMAC Steam Leak Detection Equipment. Specifically, on page 3/4 3-29 for each unit, the Channel Calibration frequency of Item 4.a.4, HPCI Steam Line Tunnel Temperature - High, was inadvertently left as quarterly (Q) rather than being revised to refuel (R). The text of CP&L's September 14, 1992 license amendment request and the NRC's safety evaluation for Amendments 166 and 197, dated October 14, 1993, addressed the frequency change from quarterly to refuel for this item. Therefore, the proposed change is purely administrative in nature and can not involve a significant increase in the probability or consequences of an accident previously evaluated.
2. The proposed amendment does not create the possibility of a new or different kind

of accident from any accident previously evaluated. As stated above, the NRC's safety evaluation for Amendments 166 and 197, dated October 14, 1993, addressed the frequency change from quarterly to refuel for Item 4.a.4 of Table 4.3.2-1, HPCI Steam Line Tunnel Temperature - High. Therefore, the proposed change is purely administrative in nature and can not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed amendment does not involve a significant reduction in the margin of safety. The proposed change corrects Technical Specification pages issued for Amendment 166 for Brunswick Unit 1 and Amendment 197 for Brunswick Unit 2, regarding NUMAC Steam Leak Detection Equipment. The NRC's safety evaluation for Amendments 166 and 197, dated October 14, 1993, addressed the change of Channel Calibration frequency of Item 4.a.4, HPCI Steam Line Tunnel Temperature - High from quarterly (Q) to refuel (R). Therefore, the proposed change is purely administrative in nature and can not involve a significant reduction in the margin of safety.

ENCLOSURE 3

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ENVIRONMENTAL CONSIDERATIONS

10 CFR 51.22(c)(9) provides criterion for and identification of licensing and regulatory actions eligible for categorical exclusion from performing an environmental assessment. A proposed amendment to an operating license for a facility requires no environmental assessment if operation of the facility in accordance with the proposed amendment would not: (1) involve a significant hazards consideration; (2) result in a significant change in the types or significant increase in the amounts of any effluents that may be released offsite; (3) result in an increase in individual or cumulative occupational radiation exposure. Carolina Power & Light Company has reviewed this request and determined that the proposed amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment needs to be prepared in connection with the issuance of the amendment. The basis for this determination follows:

Proposed Change:

The Channel Calibration frequency of Item 4.a.4 of Table 4.3.2-1, HPCI Steam Line Tunnel Temperature - High, is being revised from quarterly to refuel (R).

Basis:

The change meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) for the following reasons:

1. As demonstrated in Enclosure 2, the proposed amendment does not involve a significant hazards consideration.
2. The proposed amendment does not result in a significant change in the types or significant increase in the amounts of any effluents that may be released offsite.

The proposed change corrects Technical Specification pages issued for Amendment 166 for Brunswick Unit 1 and Amendment 197 for Brunswick Unit 2, regarding NUMAC Steam Leak Detection Equipment. Specifically, on page 3/4 3-29 for each unit, the Channel Calibration frequency of Item 4.a.4, HPCI Steam Line Tunnel Temperature - High, was inadvertently left as quarterly (Q) rather than being revised to refuel (R). The text of The NRC's safety evaluation for Amendments 166 and

197, dated October 14, 1993, addressed the frequency change from quarterly to refuel for this item. Therefore, the proposed change is purely administrative in nature. As such, the change can not affect the types or amounts of any effluents that may be released offsite.

3. The proposed amendment does not result in an increase in individual or cumulative occupational radiation exposure.

As stated above, the NRC's safety evaluation for Amendments 166 and 197, dated October 14, 1993, addressed the frequency change from quarterly to refuel for Item 4.a.4 of Table 4.3.2-1, HPCI Steam Line Tunnel Temperature - High. Therefore, the proposed change is purely administrative in nature and has no affect on either individual or cumulative occupational radiation exposure.

ENCLOSURE 4

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CORRECTED PAGES

PAGE CHANGE INSTRUCTIONS

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<u>Removed Page</u>	<u>Inserted Page</u>
3/4 3-29	3/4 3-29

UNIT 2

<u>Removed Page</u>	<u>Inserted Page</u>
3/4 3-29	3/4 3-29

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ENCLOSURE 5

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TECHNICAL SPECIFICATION PAGE - UNIT 1

TABLE 4.3.2-1 (Continued)

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8. HPCI Steam Line Area Δ Temperature - High	NA	SA	R	1, 2, 3
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ENCLOSURE 6

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OPERATING LICENSE NOS. DPR-71 & DPR-62
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CORRECTED PAGES

TECHNICAL SPECIFICATION PAGE - UNIT 2

TABLE 4.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

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