

NRC FORM 366
(12-81)U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORTAPPROVED BY OMB
3150-0011
EXPIRES 4-30-82CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)01 | S | C | N | E | E | 1 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 56

CONT

01 | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 6 | 9 | 7 | 0 | 3 | 2 | 3 | 8 | 2 | 8 | 0 | 7 | 2 | 3 | 8 | 2 | 9
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35
REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | On March 23, 1982, air was found flowing into the Reactor Building (RB) through
03 | an instrument line test tee in penetration WB-13. The cap was missing from the
04 | test tee which constitutes a violation of Technical Specification 3.6.1. There
05 | is no evidence of abnormal radiation levels during operation. This incident did
06 | not prevent Pressure Switch LPS-22 from required operation for the associated
07 | RB Spray train, and no other systems were affected. Thus, there was no signifi-
08 | cant effect on the health and safety of the public.

09 | S | A | 11 | A | 12 | C | 13 | I | N | S | T | R | U | 14 | S | 15 | Z | 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35
SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

17 | 8 | 2 | 0 | 0 | 8 | 0 | 1 | X | 1
21 22 23 24 25 26 27 28 29 30 31 32
LER/NO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.

18 | A | 19 | G | 20 | Z | 21 | Z | 22 | 0 | 0 | 0 | Y | 23 | N | 24 | L | 25 | M | 2 | 3 | 5
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | The apparent cause of this incident was personnel error in that the pressure
11 | switch was not properly returned to service after the last calibration. The
12 | test tee cap was replaced and containment integrity reestablished. Investiga-
13 | tion of other instruments revealed no similar problems. Procedure modification
14 | will require independent verification of normal lineup.

15 | G | 28 | 0 | 0 | 0 | 29 | Hot shutdown | D | 31 | NRC Resident Inspector
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION

16 | Z | 33 | Z | 34 | NA | NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE

17 | 0 | 0 | 0 | 37 | Z | 38 | NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION

18 | 0 | 0 | 0 | 40 | NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
PERSONNEL INJURIES NUMBER DESCRIPTION

19 | Z | 42 | NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION

20 | N | 44 | NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
PUBLICITY ISSUED DESCRIPTION

NRC USE ONLY

NAME OF PREPARER J. F. Norris

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