

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

H. B. RAY
STATION MANAGER

July 15, 1982

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SCE
REGION V
TELEPHONE
(714) 492-7700

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. R. H. Engleken, Regional Administrator

Dear Sir:

Subject: Docket No. 50-361
30-day Report
Licensee Event Report No. 82-056
San Onofre Nuclear Generating Station, Unit 2

In accordance with Appendix A Technical Specification 6.9.1.13.b to Operating License NPF-10 for San Onofre Unit 2, this submittal provides the required 30-day written report for a reportable occurrence involving the Reactor Protection System (RPS) Instrumentation.

On June 15, 1982, while in Mode 3, one of four level instruments from each of the two steam generators were determined to be malfunctioning. The first, 2LI-1113-2 (Channel B, S/G 2E-089) failed the channel check during routine shift surveillance; the second, 2LI-1123-1 (Channel A, S/G 2E-088), was determined to be inoperable due to stripped threads.

In accordance with Technical Specification 3.3.2 Table 3.3-3, signals from two level instrumentation channels per steam generator are required for a Reactor Trip and a minimum of three per steam generator must be operable while in Modes 1 - 3. The associated Action Statement Number 9, requires that the inoperable channel must be placed in the bypassed or tripped condition within one hour and that the inoperable channel must be returned to operable status no later than the next cold shutdown. The two malfunctioning channels were placed in the bypass mode at the time they were determined to be inoperable. A Work Order was issued and instrument 2LI-1113-2 was repaired, calibrated and returned to service on June 16, 1982. Instrument 2LI-1123-1 was repaired by installing a temporary sleeve on June 18, 1982. The channel was calibrated and returned to service on July 6, 1982 during the cold shutdown mode. The temporary sleeve will be replaced by a permanent sleeve at a future date.

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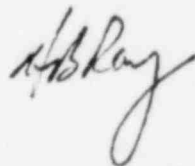
July 15, 1982

Action 9 of Technical Specification 3.3.2 also requires that if the inoperable channel is bypassed, the desirability of maintaining this channel in the bypassed condition, shall be reviewed in accordance with Technical Specification 6.5.1.6e. Technical Specification 6.5.1.6e states that the Onsite Review Committee (OSRC) shall be responsible for review and documentation of judgement concerning prolonged operation in bypass, channel trip and/or repair of defective protection channels of process variables placed in bypass since the last OSRC meeting. The channels were placed in a bypass mode and returned to service in a short period of time and since there is no decay heat in the core, the review by OSRC is not necessary.

Since 3 out of the 4 level instrumentation per steam generator were operable and only two per steam generator are required for a Reactor Trip, there was no effect on the public health or safety as a result of this incident. Licensee Event Report No. 82-056 is enclosed.

If you should require additional information concerning this occurrence, please contact me.

Sincerely,



Enclosure: LER No. 82-056

cc: A. E. Chaffee (USNRC Resident Inspector, San Onofre Unit 2)

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement

U. S. Nuclear Regulatory Commission
Office of Management Information and Program Control

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