



**Commonwealth Edison**

Quad Cities Nuclear Power Station  
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AMS-93-10

October 4, 1993

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2  
Monthly Performance Report  
NRC Docket Nos. 50-254 and 50-261

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of September 1993.

Respectfully,

COMMONWEALTH EDISON COMPANY  
QUAD-CITIES NUCLEAR POWER STATION

Anthony M. Scott  
System Engineering Supervisor

AMS/dak

Enclosure

cc: J. Martin, Regional Administrator  
T. Taylor, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

September 1993

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

## TABLE OF CONTENTS

- I. Introduction
- II. Summary of Operating Experience
  - A. Unit One
  - B. Unit Two
- III. Plant or Procedure Changes, Tests, Experiments, and Safety Related Maintenance
  - A. Amendments to Facility License or Technical Specifications
  - B. Facility or Procedure Changes Requiring NRC Approval
  - C. Tests and Experiments Requiring NRC Approval
  - D. Corrective Maintenance of Safety Related Equipment
- IV. Licensee Event Reports
- V. Data Tabulations
  - A. Operating Data Report
  - B. Average Daily Unit Power Level
  - C. Unit Shutdowns and Power Reductions
- VI. Unique Reporting Requirements
  - A. Main Steam Relief Valve Operations
  - B. Control Rod Drive Scram Timing Data
- VII. Refueling Information
- VIII. Glossary

## I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Matt Benson and Debra Kelley, telephone number 309-654-2241, extensions 2995 and 2240.

## II. SUMMARY OF OPERATING EXPERIENCE

### A. Unit One

Quad Cities Unit One began the month of September 1993 at full power. On September 18, load was reduced to 70% power to perform outboard Main Steam Isolation Valve (MSIV) closing time surveillances.

On September 19, the generator was taken off line twice to repair Combined Intermediate Valve (CIV) #6. The generator was off line a total of five and one half hours and was back to full power by the end of the day.

A few other load drops were performed per BPO which did not cause the average daily power level to decrease by greater than 20%.

### B. Unit Two

Quad Cities Unit Two began the month of September 1993 in cold shutdown as repairs were made to a leaking feed water check valve. The unit was made critical at 10:13 hours on September 13 and the generator was synchronized to the grid at 05:59 hours on September 14.

Power was reduced for testing of hydrogen water chemistry, during September, but the average daily power level remained above 80% of full power.

On September 29, an equipment operator discovered there were no cooling fans running on the unit two main transformer. Load was reduced to 210 MWe due to increased oil temperature. A broken contactor was replaced and full power was restored on September 30.



III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS,  
AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

## UNIT 1 &amp; 1/2 MAINTENANCE SUMMARY

NWR#	SYSTEM	EPN#	WORK REQUESTED	WORK PERFORMED
Q05234	1000	1-1001-3D	Repair 1D RHRSW pump discharge valve.	Cleaned yoke, sleeve, and bolts. Lubricated valve and yoke sleeve.
Q05805	1000	1-1099-74D	Replace hand wheel on D RHRSW pump discharge pressure stop valve.	Installed a new hand wheel.
Q06303	8800	1-8801B	Investigate/repair lost light indication.	Adjusted limit switch.
Q07283	2400	1-2402B	Repair/replace "B" comsip delphi flow indicator.	Replaced regulator diaphragms and tube assemblies.
Q07326	1000	1-1001-65D	Replace leaking sightglass on 1D RHRSW pump.	Found sightglass bent and leaking oil. Replaced sightglass and marked correct oil level.
Q08700	1600	1-1640-200 1-1641-223	Repair/recalibrate torus temperature recorder and transducer.	Replaced fan assembly with a new one. Swapped cards to make channels 5 and 6 operable. Calibrated.
Q08740	2300	1-2340-1 1-2340-10 1-2358	During HPCI high pressure test, required flow and RPM were not established. Investigate/repair.	Recalibrated control room square root converter, adjusted internal power supply, and calibrated the transmitter.
Q08741	6700	1/2-6700-068	Replace power switch and interlock switch on breaker.	Removed damaged switches and installed new switches.
Q09049	1000	1-1099-72D 1-1099-73D	Replace missing handwheels on 1D RHRSW pump vent valves.	Installed handwheels on both valves.

Q09369	750	1-750-7D	Repair power supply for IRM 16 chassis.	Replaced pre-regulator.
Q09374	300	1-305-126-0231	Replace blown diaphragm on scram inlet valve.	Found diaphragm ripped in half. Replaced diaphragm.
Q09665	3900	1-3903	Reinstall oiler on DG cooling water pump.	Added 3/4" nipple and new oiler inboard. Reset oiler level outboard. Installed oiler.
Q09803	1000	1-1001-65D	Replace slinger ring on RHRSW pump motor.	Replace slinger ring.
Q09804	7500	1/2-7506A	Investigate loss of indication on starting "A" SBTGT fan.	Installed new relay and overload heater.
Q10212	1000	1-1001-65D	Repair oil sightglass on 1D RHRSW pump.	Cut about 1/2 inch off glass and reinstalled. No leaks observed.
Q80936	756	1-756-5641C	Investigate problem with LPRM 56-41C indicating bypassed on 4 rod display.	Replaced two relays on LPRM card.
Q97538	1600	1-1640-200A	Replace fan in torus temperature recorder.	Replaced fan.
Q97759	1000	1-1001-65D	Repair 1D RHRSW pump inboard and outboard seal leakage.	Replaced mechanical seals with a new design. Replaced impeller rings, casing rings, fast coupling, bearings, and other miscellaneous hardware.



## UNIT 2 MAINTENANCE SUMMARY

NWR#	SYSTEM	EPN#	WORK REQUESTED	WORK PERFORMED
Q01954	3100	2-3141-11A	Blow out condenser vacuum transmitter sensing lines and recalibrate transmitters.	Found 11A pressure indicator inoperable. Blew out all sensing lines and recalibrated to plant specifications.
Q05918	750	2-750-10A	IRM/APRM Recorder behaving erratically. Investigate/repair.	Replaced 2-756-301A switch with a new one.
Q06748	1400	2-1402-26B 2-1402-29B 2-1402-35B	Repair leak on 2B core spray testable check valves.	Replaced valves, packing, and piping.
Q06979	300	2-305-126-3451 2-305-127-3451	Investigate/repair HCU for CRD 34-51. Rod unable to withdraw.	Adjusted seating pressure and spring tension on scram inlet and outlet valves.
Q07061	1100	2-1154	Recalibrate standby liquid control temperature indicating controller.	Installed new controller and calibrated.
Q08509	7800	2-7828-2	Repair MCC 28-2 light socket.	Found retainer ring on wrong side of socket. Replaced socket with a new one.
Q09619	2200	2-2252-12	Repair DG exciter cabinet panel cover.	Found 1 bolt sheared off in one of the cover holes. Replaced missing bolt. Welded cabinet to base.
Q09621	2200	2-2252-10	Attach DG relay and metering panel to floor.	Realigned, prepped, and welded base and cabinet to floor.

Q01954	3100	2-3141-11A	Blow out condenser vacuum transmitter sensing lines and recalibrate transmitters.	Found 11A pressure indicator inoperable. Blew out all sensing lines and recalibrated to plant specifications.
Q09756	700	2-700-7	Troubleshoot rod block monitor 7. Pegs upscale high when most rods are selected.	Cleaned card terminal contacts, reinstalled the boards. Tested successfully.
Q09757	7800	2-78294-A4	Investigate/repair MCC 29-4, standby gas treatment fan 1/2-7506A.	Installed new overload and heater.
Q09860	1700	2-1705-16A	Repair "A" fuel pool radiation monitor spiking.	Replaced radiation sensor, converter, and monitor tube.
Q09912	8300	2-8301-2B-1	Find and repair 125VDC ground.	Located ground in junction box. Removed cover, cleaned internals of all water and dirt. Installed new terminal strip in junction box.
Q09941	1700	2-1705-2B	Investigate/repair "B" main steam line log rad monitor.	Replaced monitor chassis, circuit board, and power supply module.
Q09965	6000	2-6000	Weld generator grounding transformer to support structure.	Welded ground cabinet to base.
Q10033	203	2-203-1D 2-203-2C 2-203-2B	Repair/replace 2-way pilots for the 1D inboard MSIV actuators.	Disassembled pilots, cleaned valve bodies, rebuilt piston assemblies as necessary, and reassembled valves.

#### IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

##### UNIT 1

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of occurrence</u>
93-014	09-02-93	APRM #3 and IRM 22 Bypass at same time with no 1/2 scram.
93-019	09-03-93	1A RHR pump started instead of 1A RHRSW pump.

##### UNIT 2

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of occurrence</u>
93-020	09-01-93	Missed continuous fire watch for the U2 H2 seal oil and turbine oil tank deluge.
93-021	08-24-93	U2 EDG relay and metering panel and excitation cabinet not attached to the floor.

## V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

APPENDIX C			
OPERATING DATA REPORT			
DOCKET NO.		50-254	
UNIT		One	
DATE		October 6, 1993	
COMPLETED BY		Matt Benson	
TELEPHONE		(309) 654-2241	
OPERATING STATUS			
0000 090193			
1. REPORTING PERIOD: 2400 093093 GROSS HOURS IN REPORTING PERIOD: 720			
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769 DESIGN ELECTRICAL RATING (MWe-NET): 789			
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A			
4. REASONS FOR RESTRICTION (IF ANY):			
	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	720.00	5749.20	148510.10
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	3421.90
7. HOURS GENERATOR ON LINE	714.50	5683.80	144075.40
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	909.20
9. GROSS THERMAL ENERGY GENERATED (MWH)	1723063.30	13296030.30	310877854.10
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	562680.00	4358059.00	100777149.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	537724.00	4169242.00	95044754.00
12. REACTOR SERVICE FACTOR	100.00	87.76	78.91
13. REACTOR AVAILABILITY FACTOR	100.00	87.76	80.73
14. UNIT SERVICE FACTOR	99.24	86.76	76.55
15. UNIT AVAILABILITY FACTOR	99.24	86.76	77.04
16. UNIT CAPACITY FACTOR (Using MDC)	97.12	82.76	65.67
17. UNIT CAPACITY FACTOR (Using Design MWe)	94.66	80.66	64.01
18. UNIT FORCED OUTAGE RATE	0.76	13.24	6.13
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			
20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP: _____			
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):			
	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			



## APPENDIX C

## OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT Two

DATE October 6, 1993

COMPLETED BY Matt Benson

TELEPHONE (309) 654-2241

## OPERATING STATUS

0000 090193

1. REPORTING PERIOD: 2400 093093 GROSS HOURS IN REPORTING PERIOD: 720

2. CURRENTLY AUTHORIZED POWER LEVEL (MWe): 2511 MAX > DEPEND > CAPACITY: 769  
DESIGN ELECTRICAL RATING (MWe-NET): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	421.80	3220.50	142395.75
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	2985.80
7. HOURS GENERATOR ON LINE	402.00	3041.10	138682.55
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	702.90
9. GROSS THERMAL ENERGY GENERATED (MWH)	920215.90	6536426.00	299199835.20
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	301778.00	2110794.00	96136055.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	285850.00	1992852.00	91023595.00
12. REACTOR SERVICE FACTOR	58.58	49.16	76.29
13. REACTOR AVAILABILITY FACTOR	58.58	49.16	77.89
14. UNIT SERVICE FACTOR	55.83	46.42	74.30
15. UNIT AVAILABILITY FACTOR	55.83	46.42	74.68
16. UNIT CAPACITY FACTOR (Using MDC)	51.63	39.56	63.42
17. UNIT CAPACITY FACTOR (Using Design MWe)	50.32	38.56	61.81
18. UNIT FORCED OUTAGE RATE	44.17	33.75	8.57

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD &lt; ESTIMATED DATE OF STARTUP:

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254  
UNIT One  
DATE October 6, 1993  
COMPLETED BY Matt Benson  
TELEPHONE (309) 654-2241

MONTH September 1993

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1.	<u>773</u>
2.	<u>708</u>
3.	<u>777</u>
4.	<u>716</u>
5.	<u>706</u>
6.	<u>775</u>
7.	<u>779</u>
8.	<u>779</u>
9.	<u>782</u>
10.	<u>783</u>
11.	<u>780</u>
12.	<u>744</u>
13.	<u>782</u>
14.	<u>782</u>
15.	<u>786</u>
16.	<u>787</u>

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17.	<u>789</u>
18.	<u>581</u>
19.	<u>361</u>
20.	<u>776</u>
21.	<u>732</u>
22.	<u>785</u>
23.	<u>784</u>
24.	<u>764</u>
25.	<u>762</u>
26.	<u>718</u>
27.	<u>757</u>
28.	<u>765</u>
29.	<u>765</u>
30.	<u>762</u>
31.	<u></u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

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APPENDIX B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265  
UNIT Two  
DATE October 5, 1993  
COMPLETED BY Matt Benson  
TELEPHONE (309) 654-2241

MONTH September 1993

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1.	<u>- 9</u>
2.	<u>- 8</u>
3.	<u>- 8</u>
4.	<u>- 8</u>
5.	<u>- 9</u>
6.	<u>- 9</u>
7.	<u>- 9</u>
8.	<u>- 9</u>
9.	<u>- 9</u>
10.	<u>- 9</u>
11.	<u>- 9</u>
12.	<u>- 8</u>
13.	<u>- 9</u>
14.	<u>439</u>
15.	<u>775</u>
16.	<u>788</u>

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17.	<u>670</u>
18.	<u>708</u>
19.	<u>749</u>
20.	<u>791</u>
21.	<u>719</u>
22.	<u>786</u>
23.	<u>788</u>
24.	<u>768</u>
25.	<u>769</u>
26.	<u>770</u>
27.	<u>749</u>
28.	<u>755</u>
29.	<u>701</u>
30.	<u>292</u>
31.	<u></u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

## APPENDIX D

### UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME One

COMPLETED BY Matt Benson

DATE October 6, 1993 REPORT MONTH September 1993

TELEPHONE 309-654-2241

[illegible]

APPENDIX D  
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Two

COMPLETED BY Matt Benson

DATE October 6, 1993 REPORT MONTH September, 1993

TELEPHONE 309-654-2241

[illegible]



## VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

### A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

### B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram timing for the reporting period. All scram timing as performed with reactor pressure greater than 800 PSIG.

RESULTS OF SCRAM TIMING MEASUREMENTS  
PERFORMED ON UNIT 1 & 2 CONTROL  
ROD DRIVES, FROM 01/01/93 TO 12/31/93

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
		5	20	50	90		
		0.375	0.900	2.00	3.5	7 sec.	Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
01-28-93	1	0.30	0.67	1.42	2.15	H-12 2.15	For Accumulator Replacement
02-12-93	1	0.33	0.72	1.54	2.77	D-5 2.77	Accumulator Replacement U2 Q05404
02-26-93	1	0.32	0.69	1.46	2.61	K-7 2.61	Scram Inlet Valve U2 Q05593
05-28-93	177	0.32	0.705	1.49	2.60	D-9 3.47	U2 Start Up Scram Timing Q2R12
07-13-93	4	0.33	0.73	1.51	2.60	D-9 2.69	For WR Test Q08085 Q07146 U2 Q08364 Q07147
07-23-93	90	0.31	0.69	1.45	2.53	D-8 3.4	U1 SEQA & Q08229
07-29-93	1	0.32	0.71	1.51	2.66	F-10 2.66	U2 Q08629 Accumlator
08-07-93	1	0.28	0.64	1.37	2.39	F-15 2.39	U1 Q09059 Scram Solenoid
08-19-93	1	0.28	0.60	1.26	2.19	A-8 2.19	U1 Q09374 Diaphragm
09-24-93	1	0.32	0.68	1.41	2.46	C-11 2.46	U2 Q08627 Accumulator

## VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD CITIES REFUELING  
INFORMATION REQUEST

QTP 300-532  
Revision 2  
October 1989

1. Unit: 01 Reload: 12 Cycle: 13
2. Scheduled date for next refueling shutdown: 3-14-94
3. Scheduled date for restart following refueling: 6-13-94
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:  
NOT AS YET DETERMINED
5. Scheduled date(s) for submitting proposed licensing action and supporting information:  
NOT AS YET DETERMINED
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:  
160 GE10 Fuel Bundles will be loaded during QIR13.
7. The number of fuel assemblies.
  - a. Number of assemblies in core: 724
  - b. Number of assemblies in spent fuel pool: 1557
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
  - a. Licensed storage capacity for spent fuel: 3657
  - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

QUAD CITIES REFUELING  
INFORMATION REQUEST

QTP 300-532  
Revision 2  
October 1989

1. Unit: Q2 Reload: 12 Cycle: 13
2. Scheduled date for next refueling shutdown: 09-24-94
3. Scheduled date for restart following refueling: 12-04-94
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:  
  
NOT AS YET DETERMINED.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:  
  
NOT AS YET DETERMINED.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:  
  
NONE AT PRESENT TIME.
7. The number of fuel assemblies.
  - a. Number of assemblies in core: 724
  - b. Number of assemblies in spent fuel pool: 2583
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
  - a. Licensed storage capacity for spent fuel: 3897
  - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009



## VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	- Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	- American National Standards Institute
APRM	- Average Power Range Monitor
ATWS	- Anticipated Transient Without Scram
BWR	- Boiling Water Reactor
CRD	- Control Rod Drive
EHC	- Electro-Hydraulic Control System
EOF	- Emergency Operations Facility
GSEP	- Generating Stations Emergency Plan
HEPA	- High-Efficiency Particulate Filter
HPCI	- High Pressure Coolant Injection System
HRSS	- High Radiation Sampling System
IPCLRT	- Integrated Primary Containment Leak Rate Test
IRM	- Intermediate Range Monitor
ISI	- Inservice Inspection
LER	- Licensee Event Report
LLRT	- Local Leak Rate Test
LPCI	- Low Pressure Coolant Injection Mode of RHRS
LPRM	- Local Power Range Monitor
MAPLHGR	- Maximum Average Planar Linear Heat Generation Rate
MCPR	- Minimum Critical Power Ratio
MFLCPR	- Maximum Fraction Limiting Critical Power Ratio
MPC	- Maximum Permissible Concentration
MSIV	- Main Steam Isolation Valve
NIOSH	- National Institute for Occupational Safety and Health
PCI	- Primary Containment Isolation
PCIOMR	- Preconditioning Interim Operating Management Recommendations
RBCCW	- Reactor Building Closed Cooling Water System
RBM	- Rod Block Monitor
RCIC	- Reactor Core Isolation Cooling System
RHRS	- Residual Heat Removal System
RPS	- Reactor Protection System
RWM	- Rod Worth Minimizer
SBGTS	- Standby Gas Treatment System
SBLC	- Standby Liquid Control
SDC	- Shutdown Cooling Mode of RHRS
SDV	- Scram Discharge Volume
SRM	- Source Range Monitor
TBCCW	- Turbine Building Closed Cooling Water System
TIP	- Traversing Incore Probe
TSC	- Technical Support Center