

Docket No. 50-423
B14614

Attachment 1

Millstone Nuclear Power Station, Unit No. 3
Proposed Revision to Technical Specifications
Reactor Trip System Interlocks
Marked-up Pages of Technical Specifications

September 1993

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TABLE 2.4-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TOTAL ALLOWANCE (TA)</u>	<u>Z</u>	<u>SENSOR ERROR (S)</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
16. Turbine Trip					
a. Low Fluid Oil Pressure	N.A.	N.A.	N.A.	≥ 500 psig	≥ 450 psig
b. Turbine Stop Valve Closure	N.A.	N.A.	N.A.	$\geq 1\%$ open	$\geq 1\%$ open
17. Safety Injection Input from ESF	N.A.	N.A.	N.A.	N.A.	N.A.
18. Reactor Trip System Interlocks					
a. Intermediate Range Neutron Flux, P-6	N.A.	N.A.	N.A.	$\geq 1 \times 10^{-10}$ amp	$\geq 6 \times 10^{-11}$ amp
b. Low Power Reactor Trips Block, P-7					
1) P-10 input (Note 5)	N.A.	N.A.	N.A.	$\leq 10\%$ of RTP**	$\leq 12.1\%$ of RTP**
2) P-13 input	N.A.	N.A.	N.A.	$\leq 10\%$ RTP** Turbine Impulse Pressure Equivalent	$\leq 12.1\%$ RTP** Turbine Impulse Pressure Equivalent
c. Power Range Neutron Flux, P-8					
1) Four Loops Operating	N.A.	N.A.	N.A.	$\leq 37.5\%$ of RTP**	$\leq 39.6\%$ of RTP**
2) Three Loops Operating	N.A.	N.A.	N.A.	$\leq 37.5\%$ of RTP**	$\leq 39.6\%$ of RTP**

**RTP = RATED THERMAL POWER

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TOTAL ALLOWANCE (TA)</u>	<u>Z</u>	<u>SENSOR ERROR (S)</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
d. Power Range Neutron Flux, P-9	N.A.	N.A.	N.A.	$\leq 51\%$ of RTP**	$\leq 53.1\%$ of RTP**
e. Power Range Neutron Flux, P-10 (Note G)	N.A.	N.A.	N.A.	$\geq 9\%$ of RTP**	$\geq 7.9\%$ of RTP**
19. Reactor Trip Breakers	N.A.	N.A.	N.A.	N.A.	N.A.
20. Automatic Trip and Interlock Logic	N.A.	N.A.	N.A.	N.A.	N.A.
21. Three Loop Operation Bypass Circuitry	N.A.	N.A.	N.A.	N.A.	N.A.

**RTP = RATED THERMAL POWER

TABLE 2.2-1 (Continued)TABLE NOTATIONS (Continued)

NOTE 3: (Continued)

 $K_6 = 0.00180/^{\circ}\text{F}$ for $T > T''$ and $K_6 = 0$ for $T \leq T''$,

 $T =$ As defined in Note 1,

 $T'' =$ Indicated T_{avg} at RATED THERMAL POWER (Calibration temperature for ΔT instrumentation, $\leq 587.1^{\circ}\text{F}$),

 $S =$ As defined in Note 1, and

 $f_2(\Delta I) = 0$ for all ΔI .
NOTE 4: The channel's maximum Trip Setpoint shall not exceed its computed Trip Setpoint by more than 2.7% ΔT span.

Note 5: The Setpoint is for increasing power

Note 6: The setpoint is for decreasing power.

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BASESReactor Trip System Interlocks (Continued)

P-8 On increasing power, P-8 automatically enables Reactor trips on low flow in one or more reactor coolant loops. On decreasing power, the P-8 automatically blocks the above listed trips.

P-9 On increasing power, P-9 automatically enables Reactor trip on Turbine trip. On decreasing power, P-9 automatically blocks Reactor trip on Turbine trip.

P-10 On increasing power, P-10 allows the manual block of the Intermediate Range trip and the Low Setpoint Power Range trip; and automatically blocks the Source Range trip and deenergizes the Source Range high voltage power. On decreasing power, the Intermediate Range trip and the Low Setpoint Power Range trip are automatically reactivated. Provides input to P-7.

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P-13 Provides input to P-7.

Three Loop Operation Bypass Circuitry

The Three Loop Operation Bypass Circuitry reactor trip ensures that a sufficient number and the correct combination of trip circuits remain available to provide necessary protection and mitigation capability during three loop operation. Should more than two channels in one train or two dissimilar channels in two trains be bypassed, a reactor trip will occur. In this manner, it is ensured that sufficient protective features remain to mitigate the consequences of analyzed transients.

INSERT A

P-10 On increasing power, P-10 provides input to P-7 to ensure that Reactor Trips on low flow in more than one reactor coolant loop, reactor coolant pump low shaft speed, pressurizer low pressure and pressurizer high level are active ^{when} before power reaches 11%. It also allows the manual block of the Intermediate Range trip and the Low Setpoint Power Range trip; and automatically blocks the Source Range trip and deenergizes the Source Range high voltage power.

On decreasing power, P-10 resets to automatically reactivate the Intermediate Range trip and the Low Setpoint Power Range trip ~~before~~ power drops below 9%. It also provides input to reset P-7. ^{before}

Attachment 2

Millstone Nuclear Power Station, Unit No. 3

Proposed Revision to Technical Specifications
Reactor Trip System Interlocks

Retyped Pages of Technical Specifications

September 1993

TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TOTAL ALLOWANCE (TA)</u>	<u>Z</u>	<u>SENSOR ERROR (S)</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
16. Turbine Trip					
a. Low Fluid Oil Pressure	N.A.	N.A.	N.A.	≥ 500 psig	≥ 450 psig
b. Turbine Stop Valve Closure	N.A.	N.A.	N.A.	$\geq 1\%$ open	$\geq 1\%$ open
17. Safety Injection Input from ESF	N.A.	N.A.	N.A.	N.A.	N.A.
18. Reactor Trip System Interlocks					
a. Intermediate Range Neutron Flux, P-6	N.A.	N.A.	N.A.	$\geq 1 \times 10^{-10}$ amp	$\geq 6 \times 10^{-11}$ amp
b. Low Power Reactor Trips Block, P-7					
1) P-10 input (Note 5)	N.A.	N.A.	N.A.	$\leq 11\%$ of RTP**	$\leq 12.1\%$ of RTP**
2) P-13 input	N.A.	N.A.	N.A.	$\leq 10\%$ RTP** Turbine Impulse Pressure Equivalent	$\leq 12.1\%$ RTP** Turbine Impulse Pressure Equivalent
c. Power Range Neutron Flux, P-8					
1) Four Loops Operating	N.A.	N.A.	N.A.	$\leq 37.5\%$ of RTP**	$\leq 39.6\%$ of RTP**
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TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TOTAL ALLOWANCE (TA)	Z	SENSOR ERROR (S)	TRIP SETPOINT	ALLOWABLE VALUE
d. Power Range Neutron Flux, P-9	N.A.	N.A.	N.A.	$\leq 51\%$ of RTP**	$\leq 53.1\%$ of RTP**
e. Power Range Neutron Flux, P-10 (Note 6)	N.A.	N.A.	N.A.	$\geq 9\%$ of RTP**	$\geq 7.9\%$ of RTP**
19. Reactor Trip Breakers	N.A.	N.A.	N.A.	N.A.	N.A.
20. Automatic Trip and Interlock Logic	N.A.	N.A.	N.A.	N.A.	N.A.
21. Three Loop Operation Bypass Circuitry	N.A.	N.A.	N.A.	N.A.	N.A.

**RTP = RATED THERMAL POWER

Table 2.2-1 (Continued)

TABLE NOTATIONS (Continued)

NOTE 3: (Continued)

K_6	=	$0.00180/^{\circ}\text{F}$ for $T > T''$ and $K_6 = 0$ for $T \leq T''$,
T	=	As defined in Note 1,
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S	=	As defined in Note 1, and
$f_2(\Delta I)$	=	0 for all ΔI .

NOTE 4: The channel's maximum Trip Setpoint shall not exceed its computed Trip Setpoint by more than 2.7% ΔT span.

NOTE 5: Setpoint is for increasing power.

NOTE 6: Setpoint is for decreasing power.

LIMITING SAFETY SYSTEM SETTINGS

BASES

Reactor Trip System Interlocks (Continued)

- P-8 On increasing power, P-8 automatically enables Reactor trips on low flow in one or more reactor coolant loops. On decreasing power, the P-8 automatically blocks the above listed trips.
- P-9 On increasing power, P-9 automatically enables Reactor trip on Turbine trip. On decreasing power, P-9 automatically blocks Reactor trip on Turbine trip.
- P-10 On increasing power, P-10 provides input to P-7 to ensure that Reactor Trips on low flow in more than one reactor coolant loop, reactor coolant pump low shaft speed, pressurizer low pressure and pressurizer high level are active when power reaches 11%. It also allows the manual block of the Intermediate Range trip and the Low Setpoint Power Range trip; and automatically blocks the Source Range trip and deenergizes the Source Range high voltage power.
- On decreasing power, P-10 resets to automatically reactivate the Intermediate Range trip and the Low Setpoint Power Range trip before power drops below 9%. It also provides input to reset P-7.
- P-13 Provides input to P-7.

Three Loop Operation Bypass Circuitry

The Three Loop Operation Bypass Circuitry reactor trip ensures that a sufficient number and the correct combination of trip circuits remain available to provide necessary protection and mitigation capability during three loop operation. Should more than two channels in one train or two dissimilar channels in two trains be bypassed, a reactor trip will occur. In this manner, it is ensured that sufficient protective features remain to mitigate the consequences of analyzed transients.