ATTACHMENT 1

PEACH BOTTOM ATOMIC POWER STATION
UNITS 2 AND 3

Docket Nos. 50-277
50-278

License Nos. DPR-44
DPR-56

TECHNICAL SPECIFICATION CHANGE
92-11 (revised)

List of Revised Pages

Unit 2	Unit 3
240u	240u
240w	240w

PBAPS

TABLE 3.15**

SEISMIC MONITORING INSTRUMENTATION

Instruments and Sensor Locations#	Measurement Range	Minimum Instruments Operable
1. Triaxial Time-History Accelerometers		
a. Containment Foundation (torus compartment)	-1 to 1g	1
b. Refueling Floor	-1 to 1g	1
c. RCIC Pump (Rm #7)	-1 to 1g	1
d. "C" Diesel Generator	-1 to 1g	1
2. Triaxial Peak Accelerographs		
a. Reactor Piping (Drywell)	0.01 to 2g	1
b. Refueling Floor	0.01 to 2g	1
c. "C" Diesel Generator	0.01 to 2g	1
3. Central Recording and Analysis System		
a. Cable Spreading Rm	-1 to 1g	1*

* With reactor control room annunciation
** Effective upon completion of installation
# Seismic instrumentation located in Unit 2

PBAPS

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\# Seismic instrumentation located in Unit 2

3.15/4.14 BASES

The operability of the seismic monitoring instrumentation ensures that sufficient capability is available to promptly determine the magnitude of a seismic event and evaluate the response of those features important to safety. This capability is required to permit comparison of the measured response to that used in the design basis for the plant.

The time-history data from the Triaxial Time-History Accelerometers is recorded in the cable spreading room using a solid-state central recording system. Following a seismic event, the data is downloaded into a computer so that the results and data can be analyzed.

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ATTACHMENT 2

PEACH BOTTOM ATOMIC POWER STATION
UNITS 2 AND 3

Docket Nos. 50-277
50-278

License Nos. DPR-44
DPR-56

TECHNICAL SPECIFICATION CHANGE
92-10 (revised)

List of Revised Pages

Unit 2	Unit 3
7	7

1.0 DEFINITIONS (Cont'd)

outage, the required surveillance testing need not be performed until the next regularly scheduled outage.

Reportable Event - A reportable event shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50.

Run Mode - In this mode the reactor system pressure is at or above 850 psig and the reactor protection system is energized with APRM protection (excluding the 15% high flux trip) and RBM interlocks in service.

Safety Limit - The safety limits are limits below which the reasonable maintenance of the cladding and primary systems are assured. Exceeding such a limit requires unit shutdown and review by the Nuclear Regulatory Commission before resumption of unit operation. Operation beyond such a limit may not in itself result in serious consequences, but it indicates an operational deficiency subject to regulatory review.

Secondary Containment Integrity - Secondary Containment integrity means that the reactor building is intact and the following conditions are met:

1. At least one door in each access opening is closed.

2. The standby gas treatment is operable.

3. All Reactor Building ventilation system automatic isolation valves are operable or deactivated in the isolation position.

Shutdown - The reactor is in a shutdown condition when the reactor mode switch is in the shutdown mode position and no core alterations are being performed.

Shutdown Mode - Placing the mode switch to the shutdown position initiates a reactor scram. After about 2 seconds, this SCRAM signal is bypassed. The SCRAM logic cannot be reset until a 10 second timer is complete. The SCRAM can then be reset to restore the normal valve line-up in the control rod drive hydraulic system.

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