

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

ASME CODE SECTION XI

INSERVICE INSPECTION AND TESTING PROGRAM

SECOND TEN YEAR INSPECTION INTERVAL
DECEMBER 16, 1983 - DECEMBER 16, 1993

Inservice Inspection and Approval

Prepared by: *[Signature]*
Reviewed by: *[Signature]*
Supt. Matl & Spec Proc

7-26-90
7/31/90
Date

Inservice Testing Review and Approval

Prepared by: Ben Stephens
Reviewed by: Gary Miller
Plant Sect XI Coordinator

7/29/87
7/29/87
Date

Approved by: *[Signature]*
Manager Production Plant Maintenance

7-31-90
Date

Revision 2
7/31/90

RECORDS OF REVISIONS

<u>Page No.</u>	<u>Rev No.</u>	<u>Page No.</u>	<u>Rev No.</u>
Review/Approval Sheet	2	2.1.1-1 thru 8	0
i	1	2.1.1-9	2
ii	2	2.1.1-10 thru 32	0
iii	2	2.1.1-33	1
1.1.1-1	0	2.1.1-34 thru 38	0
1.1.1-2	0	2.1.1-39	1
1.1.1-3 thru 9	1	2.1.1-40 thru 45	0
1.1.1-10 thru 22	0	2.1.1-46	1
1.1.1-23, 24	1	2.1.1-47 thru 52	0
1.1.1-25 thru 32	0	2.1.2-1 thru 2.1.2-5	0
1.1.1-33, 34	1	2.1.2-6	1
1.1.1-35, 36	0	2.1.2-7, 8, 9	0
1.1.1-37	1	2.1.2-10	1
1.1.1-38 thru 51	0	2.1.2-11 thru 20	0
1.1.2-1 thru 5	0	2.1.3-1 thru 8	0
1.1.2-6	1	2.1.4-1 thru 6	0
1.1.2-7	0	2.2-1	1
1.1.2-8, 9	1	2.3-1	1
1.1.2-10	0	2.3-2	1
1.1.2-11	1	2.4-1 thru 21	1
1.1.2-12 thru 15	0	2.5-1	2
1.1.2-16	1	2.5-2	0
1.1.2-17 thru 20	0	2.5-3, 4	1
1.1.3-1 thru 1.1.3-7	0	2.5-5	0
1.1.4-1 thru 1.1.4-6	0	2.5-6, 7, 8	1
1.2-1	1	2.5-9	0
1.3-1	1	2.5-10 thru 12	1
1.3-2	1	2.5-13	0
1.4-1 thru 1.4-26	1	2.5-14, 15	1
1.5-1	2	2.5-16	0
1.5-2	0	2.5-17, 18	1
1.5-3, 4	1	2.5-19	0
1.4-5, 6	0	2.5-20, 21, 22	1
1.5-7 thru 1.5-24	1	2.5-23 thru 2.5-31	0
1.5-25	0	2.5-32 thru 2.5-38	1
1.5-26	1	2.5-39, 40	0
1.5-27, 28	0	2.5-41	2
1.5-29, 30, 31	1	2.5-42	deleted
1.5-32 thru 1.5-36	0	2.5-43	0
1.5-37	1	2.5-44	1
1.5-38	0		
1.5-39, 40	1		
1.5-41, 42, 43	0		
1.5-44 thru 1.5-50	1		
1.5-51, 52	0		
1.5-53, 54	1		
1.5-55	deleted		
1.5-56	0		
1.5-57	1		

PI ISI/IST ii 7/31/90
2nd 10 YR PROGRAM REV 2

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
RECORD OF INSERVICE INSPECTION AND TESTING PROGRAM REVISIONS

PI ISI/IST 2nd 10 YR PROGRAM

<u>Revision (Rev) No.</u>	<u>Date of Issue</u>	<u>Remarks</u>
0	10/14/83	2nd Ten Year Update to ASME Code Requirements
1	7/13/87	Revision to 2nd Ten Year Program Revision to Requests for Relief Addition of Requests for Relief
2	7/31/90	Withdraw Request for Relief Revise Request for Relief

SECTION 1.5 REQUESTS FOR RELIEF FROM ASME CODE SECTION XI
REQUIREMENTS DETERMINED TO BE IMPRACTICAL
UNIT NO. 1 AND COMMON COMPONENTS

This section contains a tabulation of all the requirements contained in Section XI of the ASME Code that we have determined are impractical on Unit 1 and common components.

The Requests for Relief are numbered sequentially with the following exceptions:

<u>Request for Relief No.</u>	<u>Remarks</u>
6	Withdrawn
10	Withdrawn
15	Withdrawn
17	Withdrawn
18 - 20	Withdrawn
22	Withdrawn
32	Withdrawn
34	Withdrawn
36 - 39	Withdrawn
41 - 44	Withdrawn
46 - 47	Withdrawn
49	Withdrawn
51	Withdrawn
53	Withdrawn
55	Withdrawn
57c	Withdrawn
58	Withdrawn
62	Withdrawn
64 - 65	Withdrawn
67	Withdrawn



PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT 2
TEN YEAR INTERVAL - EXAMINATION SUMMARY

TABLE 3.1
PAGE 3 OF 3

ITEM NO.	COMPONENT OR SYSTEM	IDENTIFICATION	DESCRIPTION	NDE METHOD	TOTAL ITEMS	EXAMINATION AMOUNT & EXTENT	INSPECTION PERIOD	RUNNING %	REMARKS
	<u>HEAT EXCHANGERS</u>								
	<u>REGENERATIVE HEAT EXCHANGER</u>								
B3.150	<u>NOZZLE TO VESSEL WELDS</u>	NONE	---	---	---	---	---	---	
B3.160	<u>NOZZLE INSIDE RADIUS SECTION</u>			VOL	(6)	---	---	---	RELIEF NO <u>66</u>
		W-47IR	2-ISI-13B		1	---	---	---	
		W-49IR			1	---	---	---	
		W-50IR			1	---	---	---	
		W-53IR			1	---	---	---	
		W-54IR			1	---	---	---	
		W-56IR			1	---	---	---	

SECTION 2.5 REQUESTS FOR RELIEF FROM ASME CODE SECTION XI
REQUIREMENTS DETERMINED TO BE IMPRACTICAL
UNIT NO. 2

This section contains a tabulation of all the requirements contained in Section XI of the ASME Code that we have determined are impractical on Unit 2 components.

The numbering system used parallels Section 1.5 of this report (Unit No. 1 Requests for Relief) with the following exceptions:

<u>Request for Relief No.</u>	<u>Remarks</u>
4	Not used
6	Withdrawn
10	Not used
15	Not used
17	Withdrawn
18 - 20	Withdrawn
22	Withdrawn
23	Not used
24	Withdrawn
25 - 26	Not used
30 - 31	Not used
32	Withdrawn
34	Not used
35	Withdrawn
36 - 37	Not used
38 - 39	Withdrawn
40 - 41	Not used
42 - 44	Withdrawn
46 - 47	Withdrawn
49	Not used
51	Withdrawn
53	Withdrawn
55	Withdrawn
57c	Withdrawn
58	Withdrawn
61	Not used
62	Withdrawn
64 - 65	Withdrawn
67	Withdrawn

66. REQUEST FOR RELIEF

COMPONENT OR ITEM	CODE CLASS	PROGRAM TABLE	CODE ITEM	EXAM CATEGORY
STEAM GENERATORS - PRIMARY INLET/OUTLET NOZZLES	I	3.1	B3.140	B-D
MAIN STEAM & FEEDWATER NOZZLES	II	2.10	C2.22	C-B
PRESSURIZER - RELIEF, SURGE, SPRAY & SAFETY NOZZLES	I	3.1	B3.120	B-D
REGENERATIVE HEAT EXCHANGER NOZZLES	I	3.1	B3.160	B-D
ACCUMULATOR NOZZLES	II	2.10	C2.22	C-B

CODE REQUIREMENT

Volumetric (ultrasonic) examinations for the nozzle inner radius sections shall be conducted in accordance with Articles IWB-2500 and IWC-2500 for Code Class I and II, respectively.

BASIS

The code required volume will not be examined based on the following criteria:

- A. These nozzles do not experience high cyclic temperature gradients during normal operation, therefore the conditions for producing a thermal fatigue mechanism are not applicable.
- B. Presently, there is no comprehensive inspection technique available, nor guidance for such in the ASME Code, which would provide a conclusive assessment of the code required volumes of the inner radii, particularly since no preservice results are available for comparison.
- C. Upon consideration of the above factors, a best effort examination approach to these nozzle inner radius sections is not consistent with standard ALARA practices. An estimated 8 to 10 man-REM exposure rate over the interval, per unit, (at the present radiations levels), would be experienced in attempting to perform such inconclusive examinations.

ALTERNATE

The steam generator feedwater nozzles (Table 2.10) and the pressurizer spray nozzle (Table 3.1) may be susceptible to a thermal fatigue mechanism due to the potential for high cyclic temperature gradients, therefore an attempt will be made to ultrasonically examine these inner radius areas. If service defects are detected by these examinations, the nozzles listed in the above table shall be assessed for similar examinations. Meanwhile, if a more comprehensive technique is developed and qualified, it will be implemented.

SCHEDULE FOR IMPLEMENTATION

December 21, 1984