TOLEDO EDISON

EDISON PLAZA
300 MADISON AVENUE
TOLEDO, OHIO 43652-0001

Ltr

June 12, 1990
KB90-0483

Docket No. 50-346
License No. NPF-3

Document Control Desk
U. S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, MD 20555

Gentlemen:

Monthly Operating Report, June 1990
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit No. 1 for the month of June 1990.

If you have any questions, please contact Bilal Sarsour at (419) 321-7384.

Very truly yours,

Louis F. Storz
Plant Manager
Davis-Besse Nuclear Power Station

BMS/tld

Enclosures

cc: Mr. A. Bert Davis
Regional Administrator, Region III

Mr. Paul Byron
NRC Resident Inspector

Mr. T V. Wambach
NRC Senior Project Manager

IE24
1/1

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346
UNIT Davis-Besse #1
DATE July 12, 1990
COMPLETED BY Bilal Sarsour
TELEPHONE (419)321-7384

MONTH June, 1990

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	
16	0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-346
DATE July 12, 1990
COMPLETED BY Bilal Sarsour
TELEPHONE (419)321-7384

OPERATING STATUS

1. Unit Name: Davis-Besse #1
2. Reporting Period: June, 1990
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 918
7. Maximum Dependable Capacity (Net MWe): 874
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	4,343.0	104,472
12. Number Of Hours Reactor Was Critical	0.0	608.8	54,763.4
13. Reactor Reserve Shutdown Hours	0.0	0.0	5,393.7
14. Hours Generator On-Line	0.0	608.9	52,809.2
15. Unit Reserve Shutdown Hours	0.0	0.0	1,732.5
16. Gross Thermal Energy Generated (MWH)	0.0	1,603,325	126,567,245
17. Gross Electrical Energy Generated (MWH)	0.0	536,477	41,886,778
18. Net Electrical Energy Generated (MWH)	0.0	508,740	39,295,898
19. Unit Service Factor	0.0	14.0	50.5
20. Unit Availability Factor	0.0	14.0	52.2
21. Unit Capacity Factor (Using MDC Net)	0.0	13.4	43.0
22. Unit Capacity Factor (Using DER Net)	0.0	12.9	41.5
23. Unit Forced Outage Rate	100.0	43.6	29.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: July 5, 1990
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-346
UNIT NAME Davis-Besse #1
DATE July 12, 1990
COMPLETED BY Bilal Sarsour
TELEPHONE (419) 321-7364

REPORT MONTH June, 1990

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	90-02-01	S	120	C	4	NA	NA	NA	The unit outage which began on January 26, 1990 was still in progress through the end of June 1990.
4	90-06-05	S	264	C	4	Na	NA	NA	The outage was extended longer than anticipated to complete planned and scheduled work.
5	90-06-16	F	336	H	4	NA	NA	NA	The outage was extended longer to repair RC2 and RC11 valves. See Operational Summary for further details.

[1] F: Forced
S: Scheduled

[2] Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation from Previous Month
5-Load Reduction
9-Other (Explain)

[4] Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit I - Same Source
*Report challenges to Power Operated Relief Valves (PORVs) and Pressurizer Code Safety Valves (PCSVs)

OPERATIONAL SUMMARY
June, 1990

The unit outage which began on January 26, 1990, was still in progress through the end of June 1990.

The following are the more significant outage activities performed during the month of June 1990.

1. Completed the Integrated Steam and Feedwater Rupture Control System (SFRCS) Testing.

2. The installation of all main steam safety valves was successfully completed.

3. Completed the full flow test of the Auxiliary Feedwater System.

4. The Main Turbine Stop Valve testing was completed.

5. The Local Leak Rate Testing (LLRT) on the Emergency Hatch was successfully completed and all required LLRTs per ASME Section XI requirements were also completed.

6. Completed Reactor Coolant System (RCS) hydrostatic testing.

7. Containment Air Cooler #2 performance test was successfully completed and the results indicate satisfactory performance.

8. Completed repair of RC-11 (PORV Block Valve) and RC-2 (Pressurizer Spray Valve).

REFUELING INFORMATION Date: June 1990

1. Name of facility: Davis-Besse Unit 1

2. Scheduled date for next refueling outage? September 1991

3. Scheduled date for restart from current refueling: July 1990

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what in general will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

   Ans: A license amendment request to remove cycle-specific values from Section 3 of the Technical Specifications was submitted to the NRC on June 16, 1989, based on Generic Letter 88-16, and was approved in January 1990. Technical Specification bases changes relating to the reload report were submitted February, 1990 and the changes issued in June, 1990.

5. Scheduled date(s) for submitting proposed licensing action and supporting information: All Technical Specification changes required for Cycle 7 operation have been submitted and have been issued by the NRC.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   a. Sixty Batch Reload 3.38% enriched.

   b. New fuel design Mark B8A (Reconstitutable, removable upper end fitting, Zircaloy grid spacer, debris resistant lower end cap, lower prepressurization, and annealed guide tubes).

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool, and (c) the new fuel storage areas.

   (a) 177 (b) 328 (c) 0

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

   Present: 735 Increased size by: approximately 900 by 1994 is planned

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

   Date: 1996 - assuming ability to unload the entire core into the spent fuel pool is maintained