



**Entergy  
Operations**

Entergy Operations, Inc.  
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July 3, 1990

**W. T. Cottle**  
Vice President  
Nuclear Operations

U.S. Nuclear Regulatory Commission  
Mail Station P1-137  
Washington, D.C. 20555  
Attention: Document Control Desk

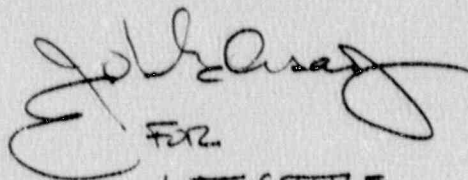
Gentlemen:

SUBJECT: Grand Gulf Nuclear Station  
Unit 1  
Docket No. 50-416  
License No. NPF-29  
Response to Generic Letter 90-04;  
Status of Implementation of Generic  
Safety Issue (GSI) Requirements or  
Corrective Actions  
AECM-90/0113

On May 11, 1990, Entergy Operations received Generic Letter 90-04, Request for Information on the Status of Licensee Implementation of Generic Safety Issues Resolved With Imposition of Requirements or Corrective Actions. Entergy Operations' response to Generic Letter 90-04 is attached.

If you have any questions, please advise.

Yours truly,

  
W. T. COTTLE

WTC:cg  
Attachment

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GENERIC LETTER 90-04  
Generic Safety Issues Resolved With Imposition of  
Requirements or Corrective Actions

<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*/DATE</u>	<u>SEE REMARKS NO.</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	BWR	NC	1
41 (B058)	BWR Scram Discharge Volume Systems	BWR	C(1/87)	2
43 (B107)	Reliability of Air Systems	All	I(12/90)	3
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All	I(4/92)	4
67.3.3 (A017)	Improved Accident Monitoring	All	I(4/92)	5
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All	C(11/84)	6
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All	C(9/85)	7
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All	C(4/88)	8

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\* NA Not Applicable  
NC No Changes Necessary  
C Complete  
I Incomplete  
E Evaluating

\*\* GSI 75 items derived from Salem ATWS events  
(Generic Letter 83-28)

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<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*/DATE</u>	<u>SEE REMARKS NO.</u>
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All	C(6/88)	9
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All	E(9/90)	10
75 (B078)	Items 3.1.1 & 3.1.2 - Post-Maintenance Testing (Reactor Trip System Components)	All	C(3/86)	11
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All	C(1/85)	12
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All	C(3/86)	13
75 (B088)	Item 3.2.3 - Post Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All	C(9/85)	14
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All	NA	
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	PWR	NA	
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	W/B&W	NA	



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<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*/DATE</u>	<u>SEE REMARKS NO.</u>
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	W/B&W	NA	
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	B&W	NA	
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability - Diverse Trip Features (System Functional Testing)	All	C(10/87)	15
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All	C(3/88)	16
86 (B084)	Long Range Plan for Dealing With Stress Corrosion Cracking in BWR Piping	BWR	C(4/90)	17
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	PWR	NA	
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	PWR	NA	
124	Auxiliary Feedwater System Reliability	ANO - 1 & 2 Rancho Seco, Parrie Is. 1 & 2, Crystal River -3, Ft. Calhoun	NA	

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<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*/DATE</u>	<u>SEE REMARKS NO.</u>
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All	NC	18
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All	NC	19
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All	NC	20
B-10	Behavior of BWR Mark III Containments	All BWR Mark III	NC	21
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants With OL Applications after 4/1/80	NA	22
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All	NC	23



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REMARKS NUMBER	GSI NUMBER	REMARKS
1	40	- The design of the Grand Gulf Nuclear Station as a BWR 6/Mark III precludes the application of NUREG-0803. Reference Entergy Operations response to Generic Letter 81-34, dated 2/8/82 (AECM-82/0046)
2	41	- Plant modifications implemented (via Design Change Package (DCP) No. 81/5007, dated 11/12/86) in accordance with Operating License Condition # 2.C (15). Closure date based on startup from First Refueling Outage (1/6/87)
3	43	- Final implementation based on scheduled Refueling Outage (RF04) modifications and completion of Filter Regulator Program. Details provided in response to Generic Letter 88-14 (AECM-89/0032, dated 2/20/89)
4	51	- Entergy Operations' response provided in response to Generic Letter 89-13, dated 1/29/90 (AECM-90/0007)
5	67.3.3	- Entergy Operations' response provided in letter dated 6/22/90 (AECM-90/0118)
6	** 75 (B-076)	- Entergy Operations' response provided in letter dated 11/2/84 (AECM-84/0465) - NRC evaluation documented in SER (MAEC-85/0163, dated 5/10/85)
7	75 (B-085)	- Entergy Operations' response provided in letter dated 11/2/84 (AECM-84/0465) - Entergy Operations' supplementary response provided in letters dated 8/5/85 and 9/27/85 (AECM-85/0237 and AECM-85/0291, respectively) - NRC evaluation documented in SER (MAEC-85/0353, dated 10/22/85)

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REMARKS NUMBER	GSI NUMBER	REMARKS
8	75 (B-077)	<ul style="list-style-type: none"><li>- Entergy Operations' response to item 2.1 (Part 1 and Part 2) provided in letters dated 11/4/83, 11/19/84, 8/26/85, and 4/1/88 (AECM-83/0723, AECM-84/0508, AECM-85/0250, and AECM-88/0062, respectively)</li><li>- NRC evaluation (Part 1) documented in SER (MAEC-86/0215, dated 7/9/86 - Equipment Classification)</li><li>- NRC evaluation (Part 2) documented in SER (MAEC-89/0119, dated 4/5/89 - Vendor Interface)</li></ul>
9	75 (B-086)	<ul style="list-style-type: none"><li>- Entergy Operations' response provided in letters, dated 11/4/83 and 6/28/85 (AECM-83/0723 and AECM-85/0201, respectively)</li><li>- Entergy Operations' supplementary response provided in letter dated 6/17/88 (AECM-88/0121)</li><li>- NRC evaluation documented in SER (MAEC-89/0330, dated 11/8/89)</li></ul>
10	75 (L003)	<ul style="list-style-type: none"><li>- 9/30/90 date based on NRC response due date for Generic Letter 90-03</li></ul>
11	75 (B-078)	<ul style="list-style-type: none"><li>- Entergy Operations' response provided in letters dated 11/4/83 and 12/14/84 (AECM-83/0723 and AECM-84/0533, respectively)</li><li>- Entergy Operations' supplementary response provided in letter dated 3/7/86 (AECM-86/0059)</li><li>- NRC evaluation documented in SER (MAEC-87/0185, dated 7/31/87)</li></ul>
12	75 (B-079)	<ul style="list-style-type: none"><li>- Entergy Operations' response provided in letters dated 11/4/83 and 12/14/84 (AECM-83/0723 and AECM-84/0533, respectively)</li><li>- Entergy Operations' supplementary response provided in letter dated 1/16/85 (AECM-85/0004)</li><li>- NRC evaluation documented in SER (MAEC-85/0278, dated 8/14/85)</li></ul>



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REMARKS NUMBER	GSI NUMBER	REMARKS
13	75 (B-087)	<ul style="list-style-type: none"><li>- Entergy Operations' response provided in letters dated 9/30/85 and 3/7/86 (AECM-85/0265 and AECM-86/0059, respectively)</li><li>- NRC evaluation documented in SER (MAEC-87/0185, dated 7/31/87)</li></ul>
14	75 (B-088)	<ul style="list-style-type: none"><li>- Entergy Operations' response provided in letters dated 11/4/83 and 9/30/85 (AECM-83/0723 and AECM-85/0265, respectively)</li><li>- NRC evaluation documented in SER (MAEC-85/0413, dated 12/6/85)</li></ul>
15	75 (B-092)	<ul style="list-style-type: none"><li>- Entergy Operations' response provided in letters dated 11/4/83, 9/11/84, and 6/12/85 (AECM-83/0723, AECM-84/0349, and AECM-85/0178, respectively)</li><li>- Entergy Operations' supplementary response provided in letter dated 10/16/87 (AECM-87/0193)</li><li>- NRC evaluation documented in SER (MAEC-88/0209, dated 8/5/88)</li></ul>
16	75 (B-093)	<ul style="list-style-type: none"><li>- Entergy Operations' response provided in letters dated 9/11/84, 6/12/85, 10/16/87 and 3/30/88 (AECM-84/0349, AECM-85/0178, AECM-87/0193, AECM-88/0057, respectively)</li><li>- NRC evaluation documented in SERs (MAEC-89/0131, dated 4/14/89 and MAEC-89/0188, dated 5/31/89)</li></ul>
17	86 (B-084)	<ul style="list-style-type: none"><li>- Generic Letter 88-01 (MAEC-88/0019) required response to IGSCC issue</li><li>- Entergy Operations provided response to Generic Letter 88-01 in letters dated 8/8/88 (AECM-88/0153), and 7/14/89 (AECM-89/0130)</li><li>- Entergy Operations notified the NRC by letter dated 11/20/85 (AECM-85/0372) that Induction Heat Stress Improvement (IHSI) was completed on recirculation system during Fall 1985 outage</li></ul>

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REMARKS NUMBER	GSI NUMBER	REMARKS
		- Rev. 4 of GGNS ISI program incorporated response to Generic Letter 88-01. Rev. 4 implemented 2/10/89
		- Entergy Operations supplementary response to Generic Letter 88-01 provided in letter dated 4/23/90 (AECM-90/0063)
18	A13 (B017)	- Applicable Technical Specifications and bases submitted with original license application; therefore, no additional post-license changes were required
19	A13 (B022)	- Applicable Technical Specifications and bases submitted with original license application; therefore, no additional post-license changes were required
20	A35	- Compliance with Branch Technical Position PSB-1 reviewed and approved in SER dated 9/81
21	B-10	- Entergy Operations' response to Humphrey Containment Concerns provided in letters dated 10/22/82 (AECM-82/0497), 11/18/82 (AECM-82/0540), 12/3/82 (AECM-82/0574), 12/31/82 (AECM-82/0641), 3/23/83 (AECM-83/0146), 9/7/84 (AECM-84/0443), 1/24/85 (AECM-85/0018), 2/25/85 (AECM-85/0046), 4/23/85 (CIOG letter to Denton), 9/13/85 (AECM-85/0287), 10/4/85 (AECM-85/0233), 11/27/85 (AECM-85/0376), 1/28/86 (AECM-86/0012), 5/6/86 (AECM-86/0143) and 8/14/86 (AECM-86/0175)
		- NRC review and approval documented in SER dated 3/23/87
22	B-36	- Grand Gulf submitted OL application on 4/28/78
23	B-63	- Grand Gulf's original issuance of Technical Specifications address surveillance and Limiting Condition for Operation of Pressure Isolation Valves (PIVs).