



Entergy Operations

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June 29, 1990

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U. S. Nuclear Regulatory Commission
Document Control Desk
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Subject: Arkansas Nuclear One - Units 1 & 2
Docket Nos. 50-313 and 50-368
License Nos. DPR-51 and NPF-6
Response to GL 90-04 - GSI Status Request

Gentlemen:

Generic Letter (GL) 90-04, dated April 25, 1990 (ØCNAØ49Ø21), transmitted a request for information concerning the implementation status of generic safety issues (GSI) at ANO-1 and 2. We have researched the docketed correspondence and submittals in our files and completed the Enclosure 1 table for each ANO unit, in accordance with guidance provided in GL 90-04. The attached information provides a status summary for each GSI with appropriate cross-references to other docketed transmittals containing further details. In that regard, this submittal serves merely as an index and does not contain any new or revised commitments, and should be treated accordingly by the NRC staff.

Please do not hesitate to contact my office should you require additional information regarding this issue.

Very truly yours,

James J. Fisicaro
James J. Fisicaro
Manager, Licensing

JJF/RBT
Attachments

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FACILITY NAME: ANO-1
50-313
ENTERGY OPERATIONS, INC.

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability Of Air Systems	All Plants	C 6/29/89	NRC Ltr. #CNA#68922
51 (L913)	Improving The Reliability Of Open-Cycle Service Water Systems	All Plants	I 8/90	NRC Ltr. #CNA#59008
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C 12/86	Last mods 1R7 outage
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C 8/7/85	NRC SE #CNA#88508
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C 12/13/88	NRC Ltr. #CNA128813

*In accordance with GL 90-04 guidance for completing this column.

**The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C 8/20/86	NRC SE 1CNA088603
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C 12/13/88	NRC Ltr. 0CNA128813
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	C 12/13/88	NRC Ltr. 0CNA128813
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	All Plants	C 12/17/85	NRC SE 1CNA128502
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C 7/10/85	NRC SE 1CNA078505
75 (B087)	Items 3.2.1 & 3.2.2 - Post - Maintenance Testing (All Other Safety-Related Components)	All Plants	I 12/31/90	NRC Ltr. 0CNA128813 Procedure currently being upgraded
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C 7/10/85	NRC SE 1CNA078505
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C 12/27/85	NRC SE 1CNA128505

<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C 4/16/86	NRC SE 1CNA048618
75 (B062)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C 3/10/89	TS Amendment 117 NRC Ltr. 1CNA038902
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W & B&W Plants	C 3/10/89	TS Amendment 117 NRC Ltr. 1CNA038902
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	C 10/20/86	NRC SE 1CNA108605

<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C 2/3/87	NRC SE 1CNA028707
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C 3/19/87 C 7/11/89	NRC SE 1CNA038703 NRC SE 1CNA078903
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding on Auxiliary Feedwater Pumps	All PWRs	C 6/7/88	NRC Ltr. 1CNA068804
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	C 7/89	GL 88-17 responses & R.A.I. response
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	C 8/24/88	NRC SE 1CNA083808
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	C 10/15/84	TS Amendment 84 NRC Ltr. 1CNA108403

<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C 10/15/84	TS Amendment 84 NRC Ltr. 1CNA108403
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & MMP-1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	C 1977	Millstone mods
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Absorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C 4/20/81	TS changes by order

FACILITY NAME: ANO-2
50-368
ENTERGY OPERATIONS, INC.

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability Of Air Systems	All Plants	C 6/29/89	NRC Ltr. ØCNAØ68922
51 (L913)	Improving The Reliability Of Open- Cycle Service Water Systems	All Plants	I 8/90	NRC Ltr. ØCNAØ59ØØ8
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C 5/88	Last mods 2R6
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C 8/7/85	NRC SE ØCNAØ885Ø8
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C 12/13/88	NRC Ltr. ØCNA128813

*In accordance with GL 90-04 guidance for completing this column.

**The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C 10/14/86	NRC SE 2CNA108601
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C 12/13/88	NRC Ltr. 0CNA128813
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	C 12/13/88	NRC Ltr. 0CNA128813
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	All Plants	C 1/16/86	NRC SE 2CNA018604
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C 10/29/85	NRC SE 2CNA108503
75 (B087)	Items 3.2.1 & 3.2.2 - Post - Maintenance Testing (All Other Safety-Related Components)	All Plants	I 12/31/90	NRC Ltr. 0CNA128813 Procedure currently being upgraded
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C 10/29/85	NRC SE 2CNA108503
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C 1/21/86	NRC SE 2CNA018606

<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C 4/16/86	NRC SE #CNA#48618
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	NA	
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W & B&W Plants	NA	
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA	

<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C 2/3/87	NRC SE 0CNA028707
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C 7/12/88 C 7/17/89	NRC SE 2CNA078804 NRC SE 2CNA078902
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding on Auxiliary Feedwater Pumps	All PWRs	C 6/7/88	NRC Ltr. 0CNA068804
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	I 4/91 (2R8)	Schedule proposed in GL 88-17 responses
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	I 4/91 (2R8)	Schedule proposed in AP&L Ltr. 2CAN078905
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	C 1/29/85	TS Amendment 62 NRC Ltr. 2CNA018506

<u>GSI/(MPA NO.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C 1/29/85	TS Amendment 62 NRC Ltr. 2CNA018506
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & MMP-1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	NC	ANO-2 in construction when mods required
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Absorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C 4/20/81	15 changes by order