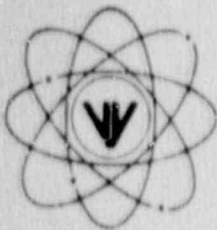


# VERMONT YANKEE NUCLEAR POWER CORPORATION



Ferry Road, Brattleboro, VT 05301-7002

REPLY TO:  
ENGINEERING OFFICE  
580 MAIN STREET  
BOLTON, MA 01740  
(508) 779-6711

June 28, 1990  
BVY 90-073

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

References: a. License No. DPR-28 (Docket No. 50-271)  
b. Letter, USNRC to All Holders of Operating Licenses and  
Construction Permits for Nuclear Power Reactors, NYY 90-85,  
dated April 25, 1990 (Generic Letter 90-04).

Subject: Vermont Yankee Response to Request for Information on the Status of Licensee  
Implementation of Generic Safety Issues Resolved with Imposition of  
Requirements or Corrective Actions

Dear Sir:

Generic Letter 90-04 [Reference (b)] requested licensees to review and provide documentation of the current implementation status of all Generic Safety Issues (GSI's) identified that apply to their respective facilities. As described in Generic Letter 90-04, a GSI is a safety concern, as identified and characterized in NUREG-0933, that affects the design, construction, or operation of all, several, or a class of nuclear power plants and may have the potential for safety improvements at such plants. Generic Letter 90-04 requested that licensees respond by June 29, 1990.

Attached to this letter, please find the information requested in Reference (b) for GSI's as they pertain to the Vermont Yankee facility. If you should have any questions regarding the attached or require further information, please contact this office.

Very truly yours,

VERMONT YANKEE NUCLEAR POWER CORPORATION

*Leonard A. Tremblay, Jr.*  
Leonard A. Tremblay, Jr.  
Senior Licensing Engineer

cc: USNRC Region I Administrator  
USNRC Resident Inspector - VYNPS  
USNRC Project Manager - VYNPS

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# ATTACHMENT I

FACILITY NAME: Vermont Yankee  
 DOCKET NO.: 50-271  
 LICENSEE: Vermont Yankee Nuclear Power Corporation

## STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>Title</u>	<u>Applicability</u>	<u>Status</u>	<u>Comments</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	All BWRs	NC	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	C	FVY 83-1, dated 1/10/83 FVY 81-14, dated 1/27/81
43 (B107)	Reliability of Air Systems	All Plants	I	Note 1
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I	Note 2
67.3.3 (A017)	Improved Accident Monitoring	All Plants	I	Note 3
75 (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	FVY 84-118, dated 10/2/84 FVY 84-116, dated 9/25/84 FVY 84-25, dated 3/23/84 FVY 83-117, dated 11/7/83
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C Note 4	FVY 85-108, dated 11/20/85 FVY 84-25, dated 3/23/84 FVY 83-117, dated 11/7/83

ATTACHMENT I  
(Continued)

<u>GSI/(MPA No.)</u>	<u>Title</u>	<u>Applicability</u>	<u>Status</u>	<u>Comments</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C	FVY 88-104, dated 12/23/88 FVY 88-053, dated 7/1/88 FVY 86-52, dated 6/2/86 FVY 85-63, dated 7/3/85 FVY 84-25, dated 3/23/84
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	FVY 88-104, dated 12/23/88 FVY 86-52, dated 6/2/86 FVY 85-63, dated 7/3/85 FVY 84-25, dated 3/23/84
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	C	FVY 85-63, dated 7/3/85 FVY 84-73, dated 6/26/84 FVY 84-25, dated 3/23/84
75 (B078)	Items 3.1.1 and 3.1.2 - Post-Maintenance Testing (Reactor Trip System Components)	All Plants	C	FVY 83-117, dated 11/7/83
75 (B079)	Item 3.1.3 - Post-Maintenance Testing - Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	FVY 83-117, dated 11/7/83
75 (B087)	Items 3.2.1 and 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	FVY 83-117, dated 11/7/83
75 (B088)	Item 3.2.3 - Post-Maintenance Testing - Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	FVY 83-117, dated 11/7/83



ATTACHMENT I  
(Continued)

<u>GSI/(MPA No.)</u>	<u>Title</u>	<u>Applicability</u>	<u>Status</u>	<u>Comments</u>
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	NA	Not applicable to BWRs.
75 (B081)	Items 4.2.1 and 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventive Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	NA	
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	NA	
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Technical Specification Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	NA	
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA	
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability - Diverse Trip Features (System Functional Testing)	All Plants	C	FVY 84-25, dated 3/23/84 FVY 83-117, dated 11/7/83

ATTACHMENT I  
(Continued)

<u>GSI/(MPA No.)</u>	<u>Title</u>	<u>Applicability</u>	<u>Status</u>	<u>Comments</u>
75 (B093)	Items 4.5.2 and 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C	FVY 85-63, dated 7/3/85 FVY 84-25, dated 3/23/84 FVY 83-117, dated 11/7/83
86 (B084)	Long Range Plan for Dealing With Stress Corrosion Cracking in BWR Piping	All BWRs	C	BVY 89-70, dated 7/25/89 FVY 88-62, dated 7/27/88 FVY 85-121, dated 12/13/85
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	NA	
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	NA	
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie, Island 1 and 2, Crystal River - 3 Ft. Calhoun	NA	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	C	FVY 84-133, dated 11/6/84 FVY 84-124, dated 10/22/84
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C	FVY 84-133, dated 11/6/84 FVY 84-124, dated 10/22/84
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek and NMP-1	NA	

ATTACHMENT I  
(Continued)

<u>GSI/(MPA No.)</u>	<u>Title</u>	<u>Applicability</u>	<u>Status</u>	<u>Comments</u>
A-35 (B023)	Adequacy of Off-Site Power Systems	All Plants	C	FVY 86-21, dated 3/14/86 FVY 86-5, dated 1/22/86 FVY 84-129, dated 11/2/84 FVY 84-69, dated 6/21/84 FVY 84-46, dated 5/15/84 FVY 82-128, dated 12/7/82 FVY 82-68, dated 6/8/82 FVY 82-56, dated 5/19/82
B-10	Behavior of BWR Mark II <sub>1</sub> Containments	ALL BWR Mark III Plants	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	NC	



ATTACHMENT I  
(Continued)

NOTE 1: As described in VY Letter BVY 89-17, dated 2/16/89, all modifications identified to be implemented relative to Generic Safety Issue (GSI) No. 43 will be completed before startup (10/1990) from the next scheduled refueling outage. The outstanding work consists of the following:

- The installation of a permanent fix to modify the inflatable door seal system for the Reactor Building equipment access doors.
- Testing of Main Steam Isolation Valve (MSIV) Accumulator Systems.
- Installation of filters and larger accumulators on the four safety relief valve accumulator assemblies.
- Completion of the revisions to the VY air quality monitoring and maintenance program to enhance the capabilities of maintaining proper instrument air quality.

NOTE 2: As described in VY Letter BVY 90-007, dated 1/22/90, all modifications identified to be implemented relative to Generic Safety Issue (GSI) No. 51 will be completed before startup (10/90) from the next scheduled refueling outage. The outstanding work consists of the following:

- Perform water and substrate sampling annually to detect presence of both Asiatic clams and Zebra mussels.
- The continuation of existing monitoring programs to verify that several safety-related heat exchangers meet their design/functional requirements and the initiation of similar monitoring programs for those components not presently included in the monitoring programs.
- The continued implementation of a valve inspection program which includes a long-range schedule for taking Service Water valves out of service, disassembling the valves for inspection, and repairing as required.
- Development of a computerized system flow model to ensure functional adequacy of the Service Water System and to determine system locations susceptible to erosion or silting. Based on an evaluation of the results of the model, an action plan addressing any further efforts required on this issue will be developed.
- Completion of any required plant procedure changes based upon implementation of the above-listed items.

ATTACHMENT I  
(Continued)

- NOTE 3: As described in VY Letter 86-122, dated 12/19/86, VY believed that implementation of RG 1.97 commitments (GSI 67.3.3 items) would be completed during the 1989 refueling outage. However, during the development of the updated separation criteria, it was determined that certain RG 1.97, Category 1, instrument loops did not fully meet required separation criteria. The NRC was notified of this situation on 5/29/90. Modifications to rectify these deviations will be completed before startup from the 1990 refueling outage (approximately 10/90).
- NOTE 4: Correspondence with the NRC has indicated that GSI 75, Item 1-2, has been completed. However, the data and information capabilities of the plant GEPAC Process Computer System, as described in VY Letter 83-117, dated 11/7/83, are to be replaced with the new ERFIS Computer System. The capabilities of the new ERFIS Computer System will envelope all the functions performed by the GEPAC computer and extend beyond to provide additional functions. As described in VY Letter FVY 86-122, dated 12/19/86, the ERFIS Computer System is scheduled to be fully operational before startup (10/90) from the 1990 refueling outage.