

Duquesne Light Company

Beaver Valley Power Station
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Vice President - Nuclear Group

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June 2

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

Reference: Beaver Valley Power Station, Unit No. 1 and No. 2
BV-1 Docket No. 50-334, License No. DPR-66
BV-2 Docket No. 50-412, License No. NPF-73
Response to Generic Letter No. 90-04

Gentlemen:

Enclosure 1 to Generic Letter 90-04 has been completed for
BVPS-1 and BVPS-2. Please contact my staff if you require further
status information.

Very truly yours,

J. D. Sieber
J. D. Sieber
Vice President
Nuclear Group

cc: Mr. J. Beall, Sr. Resident Inspector
Mr. T. T. Martin, NRC Region I Administrator
Mr. A. W. DeAgazio, Project Manager
Mr. R. Saunders (VEPCO)

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FACILITY NAME: Beaver Valley Power Station, Unit #1
 DOCKET NO.: 50-334
 LICENSEE: Duquesne Light Co.

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	N/A	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	N/A	
43 (B107)	Reliability Of Air Systems	All Plants	I	Note 1
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I	Note 2
67.3.3 (A017)	Improved Accident Monitoring	All Plants	I, E	Note 3
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	Note 4
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C	Note 5

*Please follow attached guidance for completing this column.

**The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C	Note 4
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	Note 6
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	E	Note 7
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	All Plants	C	Note 4
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	Note 4
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	Note 4
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	Note 4
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C	Note 4

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	Note 8
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	Note 9
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W & B&W Plants	C	Note 10
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	N/A	

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C	Note 4
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C	Note 4 (4.5.2) Note 11 (4.5.3)
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	N/A	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	Note 12
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	C, I	Note 13
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	N/A	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	C	Note 14

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C	Note 14
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	N/A	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	C	Note 15
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	N/A	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	N/A	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C	Note 16

Notes for BVPS-1 Generic Letter 90-04 Response

- Note 1 - Completion of various hardware changes is planned for the ninth refueling outage (currently 12/10/92 - 2/18/93). Refer to letter dated 2/14/90 from J. D. Sieber to the NRC.
- Note 2 - Completion of various program reviews and hardware changes is planned for the eighth refueling outage (currently 4/12/91 - 6/21/91). Refer to letter dated 1/29/90 from J. D. Sieber to the NRC.
- Note 3 - Completion of hardware changes is planned for the ninth refueling outage. Evaluation of the neutron flux monitoring issue is planned for response by 8/15/90. Other items require NRC action. Refer to letter dated 1/31/90 from J. D. Sieber to the NRC.
- Note 4 - Final DLC correspondence on this subject was provided from J. J. Carey to D. G. Eisenhut on 11/4/83.
- Note 5 - Final DLC correspondence on this subject was provided from J. J. Carey to S. A. Varga on 11/14/85.
- Note 6 - Final DLC correspondence on this subject was provided from J. J. Carey to the NRC on 5/4/87.
- Note 7 - DLC response is projected for 10/1/90.
- Note 8 - Final DLC correspondence on this subject was provided from J. J. Carey to S. A. Varga on 3/10/84 for issue 4.2.2 and on 6/17/85 for issue 4.2.1.
- Note 9 - Final DLC correspondence on this subject was provided from J. J. Carey to P. S. Tam on 3/4/86.
- Note 10 - Final DLC correspondence on this subject was provided from J. J. Carey to P. S. Tam on 10/27/86.
- Note 11 - Final DLC correspondence on this subject was provided from J. J. Carey to S. A. Varga on 5/31/85.
- Note 12 - Final DLC correspondence on this subject was provided from J. D. Sieber to the NRC on 3/3/88.
- Note 13 - Final DLC correspondence on the subject of Generic Letter 87-12 was provided by J. D. Sieber to the NRC on 1/13/89. Letters dated 2/23/89 and 3/17/89 from J. D. Sieber to the NRC describe "Programmed Enhancements" (G.L. 88-17) which are to be completed by 8R.
- Note 14 - Technical Specification Amendment 49 (3/30/82) implemented this change.
- Note 15 - Final DLC correspondence on this subject was provided by J. J. Carey to S. A. Varga on 5/3/82.
- Note 16 - Implemented by NRC order dated April 20, 1981.

FACILITY NAME: Beaver Valley Power Station, Unit #2
 DOCKET NO.: 50-412
 LICENSEE: Duquesne Light Co.

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	N/A	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	N/A	
43 (B107)	Reliability Of Air Systems	All Plants	I	Note 1
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I	Note 2
67.3.3 (A017)	Improved Accident Monitoring	All Plants	NC	
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	NC	
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C	Note 3

*Please follow attached guidance for completing this column.

**The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	NC	
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	NC	
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	E	Note 4
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	All Plants	NC	
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	NC	
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	NC	
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	NC	
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	NC	

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	NC	
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	NC	
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W & B&W Plants	NC	
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	N/A	

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	NC	
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	NC	
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	N/A	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	Note 5
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	C, I	Note 6
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	N/A	
A-12 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	NC	

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	NC	
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	N/A	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	NC	
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	N/A	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NC	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	NC	

Notes for EVPS-2 Generic Letter 90-04 Response

- Note 1 - Completion of hardware changes is planned for the second refueling outage (currently 8/31/90 - 11/9/90). Refer to letter dated 2/14/90 from J. D. Sieber to the NRC.
- Note 2 - Completion of various program reviews and hardware changes is planned for the second refueling outage. Refer to letter dated 1/29/90 from J. D. Sieber to the NRC.
- Note 3 - Final DLC correspondence on this subject was provided from J. D. Sieber to the NRC on 3/22/88.
- Note 4 - Response projected for 10/1/90
- Note 5 - Final DLC correspondence on this subject was provided from J. D. Sieber to the NRC on 3/3/88.
- Note 6 - Final DLC correspondence on the subject of Generic Letter 87-12 was provided by J. D. Sieber to the NRC on 1/13/89. Letters dated 2/23/89 and 3/17/89 from J. D. Sieber to the NRC describe "Programmed Enhancements" (GL 88-17) which are to be completed by the second refueling outage.