

WOLF CREEK

NUCLEAR OPERATING CORPORATION

Forrest T. Rhodes
Vice President
Engineering & Technical Services

June 28, 1990

ET 90-0108

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station PL-137
Washington, D. C. 20555

Subject: Docket No. 50-482: Response to Generic Letter 90-04,
"Request for Information on the Status of Licensee
Implementation of Generic Safety Issues Resolved with
Imposition of Requirements of Corrective Actions"

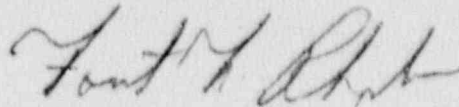
Gentlemen:

The purpose of this letter is to provide Wolf Creek Nuclear Operating Corporation's (WCNOC) response to Generic Letter 90-04, "Request for Information on the Status of Licensee Implementation of Generic Safety Issues Resolved with Imposition of Requirements or Corrective Actions". The Generic Letter requests WCNOC to review and provide documentation of the current implementation status of all generic safety issues identified in the Generic Letter that apply to Wolf Creek Generating Station (WCGS). Enclosure 1 from the Generic Letter has been utilized, in accordance with the instruction provided, and is provided as an attachment to this letter.

The status provided in the attachment is based on a review of applicable generic letters, NUREG documents, Nuclear Regulatory Commission (NRC) Safety Evaluation Report for WCGS (NUREG-0881) its supplements and other correspondence between the NRC and the WCGS Operating Agent.

If you have any questions concerning this matter, please contact me or Mr. H. K. Chernoff of my staff.

Very truly yours,



Forrest T. Rhodes
Vice President
Engineering & Technical Services

FTR/jra

cc: R. D. Martin (NRC), w/a
D. V. Pickett (NRC), w/a
M. E. Skow (NRC), w/a
J. S. Wiebe (NRC), w/a

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FACILITY NAME: Wolf Creek Generating Station
DOCKET NO.: 50-482
LICENSEE: Wolf Creek Nuclear Operating Corporation

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability Of Air Systems	All Plants	C	NRC letter dated 2/17/89 WM 88-0277 (10/25/88) WM 89-0042 (2/3/89)
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I	ET 90-0023 (1/30/90) Confirmatory letter 30 days following restart from Refuel V (12/91)
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C	NRC letter & SER dated 7/5/89; SLNRC 82-031 (7/6/82); SLNRC 83-019 (4/15/83); SLNRC 84-107 (8/16/84); License Condition 2.C.(7), Att. 3 Item (3) (6/4/85); KMLNRC 86-236 (12/15/86)
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	NRC letter & SER dated 6/25/85; KMLNRC 84-023 (2/29/84)
75 (B085)	Item 1.2 - Post-Trip Review - Data And Information Capability	All Plants	C	NRC letter & SER dated 7/14/86; KMLNRC 84-023 (2/29/84)

*Please follow attached guidance for completing this column.

**The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

<u>GSJ/MPA No.</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C	NRC letter & SER dated 9/1/88; KMLNRC 84-023 (2/29/84); WM 87-0151 (5/29/87)
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	NRC letter & SER dated 5/31/89; KMLNRC 84-023 (2/29/84)
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	E	KMLNRC 84-023 (2/29/84) KMLNRC 85-051 (2/26/85) KMLNRC 86-231 (12/10/86) GL 90-03 response due 9/25/90
75 (B078)	Item 3.1.1 & 3.1.2 - Post-Maintenance Testing (Reactor Trip System Components)	All Plants	C	NRC letter & SER dated 10/7/86; KMLNRC 84-023 (2/29/84)
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	NRC letter & SER dated 10/22/86; KMLNRC 84-023 (2/29/84); KMLNRC 86-173 (10/3/86)
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	NRC letter & SER dated 10/7/86; KMLNRC 84-023 (2/29/84)
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	NRC letter & SER dated 10/7/86; KMLNRC 84-023 (2/29/84); KMLNRC 86-173 (10/3/86)
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C	NRC letter & SER dated 7/21/86; NRC Inspection Report 87-18 dated 9/17/87; KMLNRC 84-023 (2/29/84)
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	NRC letter & SER dated 7/21/86; KMLNRC 84-023 (2/29/84); KMLNRC 85-150 (6/6/85); KMLNRC 85-179 (7/17/85)

<u>GSJ/(MPA No.)</u>	<u>TITLE</u>	<u>APPL/CABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	NRC letter & SER dated 7/19/88; KMLNRC 84-023 (2/29/84)
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W & B&W Plants	C	OL Amendment 26 dated 3/1/89; WM 87-0177 (6/29/87)
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA	
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C	NRC letter & SER dated 10/7/86; KMLNRC 84-023 (2/29/84)
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C	NRC letter & SER dated 5/31/89; KMLNRC 83-147 (11/15/83); KMLNRC 84-023 (2/29/84)
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	KMLNRC 86-027 (2/11/86) WM 88-0146 (5/24/88) (GL 88-03 response)
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	C	ET 88-0193 (12/23/88) WM 89-0041 (2/2/89) NRC Inspection Reports 89-17 and 90-18; WCNOG is evaluating Auto Closure Interlock removal.
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3, Ft. Calhoun	NA	

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	NC	Prior to OL issuance. Tech Spec 3/4.7.8
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	NC	Prior to OL issuance. Tech Spec 3/4.7.8
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMF-1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	NC	Prior to OL issuance. SER Section 8.2
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NC	Prior to OL issuance. SER Section 9.4.
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	NC	Prior to OL issuance. Tech Spec 3/4.4.6.2

Guidance For Completing Status Column

- (1) Provide a separate entry for each licensed reactor unit. If the information is identical for multiple units, so state.
- (2) If a GSI is not applicable to a unit(s), enter "N/A".
- (3) If a GSI is applicable but no changes were necessary to implement the resolution, enter "NC". If the GSI implementation was completed prior to issuance of the operating license, enter "NC", as no post-licensing changes were necessary.
- (4) If a GSI is applicable, submittal of information and/or changes were necessary and such submittals were made or changes are complete, enter "C". Also identify the licensee's document(s) to the NRC which certified completion. Provide a brief explanation of the outstanding work in the "Comments" column.
- (5) If a GSI is applicable and changes are necessary but such changes are not yet fully implemented, enter "I" and the projected month and year of completion. Provide a brief explanation of the outstanding work in the "Comments" column.
- (6) If implementation guidance for a resolved GSI was issued recently and the licensee is still evaluating the appropriate response, enter "E" and the projected response date.
- (7) The "Comments" column may be used to explain any entry in the "Status" column.