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Alabama Power

the southern electric system

June 27, 1990

Docket Nos. 50-348
50-364

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

Gentlemen:

Joseph M. Farley Nuclear Plant
Response To Generic Letter 90-04
Request For Information On The Status Of Licensee
Implementation Of Generic Safety Issues Resolved
With Imposition Of Requirements Or Corrective Actions

By letter of April 25, 1990, the NRC requested information concerning the status of implementation of all generic safety issues (GSIs). To comply with this request, Alabama Power Company undertook to review the appropriate correspondence in its files and prepared the enclosed status. This attachment gives the current status of the GSI requirements and also references the relevant supporting documentation. Please note that the remarks regarding the current status of each item are brief and do not describe all of the complex details of each GSI requirement. No new or revised commitments are contained in this transmittal. Should the NRC Staff desire more specific information regarding any particular GSI requirement, then reference can be made to the specific submittals identified in the attachment.

Taking these constraints into account, the information provided is true to the best of my knowledge and belief. If you have any questions, please advise.

Respectfully submitted,

W. G. Hairston, III

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Enclosure

cc: Mr. S. D. Ebner
Mr. S. T. Hoffman
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Facility Name: Farley Nuclear Plant
 Docket Nos.: 50-348, 50-364
 Licensee: Alabama Power Company

Enclosure

Status of Licensee Implementation of Generic Safety Issues
 Resolved With Imposition Of Requirements Or Corrective Actions⁽¹⁾

<u>GSI/ MPA No.</u>	<u>Title</u>	<u>Applicability</u>	<u>Status</u>	<u>Comments</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability of Air Systems	All Plants	I 6/91 (Unit 1) I 12/90 (Unit 2)	APCo's letters of 02-03-89 and 06-01-89 (in response to Generic Letter 88-14) committed to modifications during the Unit 1 10th and Unit 2 7th refueling outages.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I 7/91	APCo's letter of 01-23-90 (in response to Generic Letter 89-13) committed to a report within 30 days of completion of all activities to be conducted during the Unit 1 10th and Unit 2 7th refueling outages. See also APCo letters dated 05-26-81, 10-29-82 and 03-22-83.
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C	Closed by NRC letters dated 01-07-87 and 02-12-87. See also APCo letters dated 03-30-84, 06-29-84, 04-10-85 and 08-08-86.

⁽¹⁾ Unless otherwise noted, information refers to both Units 1 and 2.

<u>GSI/ MPA No.</u>	<u>Title</u>	<u>Applicability</u>	<u>Status</u>	<u>Comments</u>
75 (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	Closed by NRC SERs dated 05-21-85 and 10-29-85. See also APCo letters dated 11-04-83 and 09-06-85.
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C	Closed by NRC SER dated 06-12-85. See also APCo letters dated 11-04-83 and 06-04-85.
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C	Closed by NRC SERs dated 12-15-86 and 01-07-87 as supplemented by NRC letter dated 08-18-88. See also APCo letters dated 11-04-83, 05-08-87 and 08-12-88.
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	Closed by NRC SER dated 05-01-90. See also APCo letters dated 11-04-83, 02-06-84, 02-15-84, 04-22-85, 11-30-89 and 03-20-90.
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	E 9/90	NRC SER dated 12-15-86 which closed this issue was objected to by APCo. By letter dated 04-14-87 the NRC rejected APCo's request of 03-17-87. Generic Letter 90-03, however, was issued on 03-20-90 clarifying the NRC position. APCo intends to respond to this generic letter by 09-30-90. See also APCo letters dated 11-04-83, 06-25-84 and 04-22-85.
75 (B078)	Items 3.1.1 and 3.1.2 - Post-Maintenance Testing (Reactor Trip System Components)	All Plants	C	Closed by NRC SER dated 09-10-85. See also APCo letter dated 11-04-83.

<u>GSI/ MPA No.</u>	<u>Title</u>	<u>Applicability</u>	<u>Status</u>	<u>Comments</u>
75 (B079)	Item 3.1.3 - Post-Maintenance Testing - Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	Closed by NRC SER dated 11-12-85. See also APCo letter dated 11-04-83.
75 (B087)	Items 3.2.1 and 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	Closed by NRC SER dated 09-10-85 and 03-19-86. See also APCo letters dated 11-04-83, 02-15-84, 07-17-85 and 11-14-85.
75 (B088)	Item 3.2.3 - Post-Maintenance Testing - Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	Closed by NRC SER dated 11-12-85. See also APCo letters dated 11-04-83, 02-15-84 and 04-22-85.
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C	Closed by NRC SER dated 09-10-85. See also APCo letter dated 11-04-83.
75 (B081)	Items 4.2.1 and 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	Closed by NRC SER dated 10-24-85. See also APCo letters dated 11-04-83 and 04-22-85.
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	Closed by NRC letter dated 09-05-84. See also NRC SER dated 09-20-83 approving APCo's shunt trip design. See also APCo letters dated 05-10-83, 05-31-83, 06-30-83, 08-12-83, 08-25-83, 09-13-83, 10-20-83 and 11-04-83.

<u>GSI/ MPA No.</u>	<u>Title</u>	<u>Applicability</u>	<u>Status</u>	<u>Comments</u>
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	Technical Specifications approved by License Amendments 67 and 59 dated 10-24-86. See also APCo letters dated 08-09-85, 10-31-85, 12-16-85 and 08-01-86.
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA	
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability - Diverse Trip Features (System Functional Testing)	All Plants	C	Closed by NRC SER dated 09-10-85. See also APCo letter dated 11-04-83.
75 (B093)	Items 4.5.2 and 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C	Closed by NRC SERs dated 02-27-87 and 06-13-89. See also APCo letters dated 11-04-83, 04-22-85 and 01-18-89.
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	NRC letter dated 10-28-88 closed this issue for APCo. See also APCo letters dated 05-24-88 and 09-09-88.
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	I 6/91 (Unit 1) I 12/90 (Unit 2)	APCo's letters of 01-27-89 and 12-07-89 (in response to Generic Letter 88-17) committed to a goal of all hardware modifications to be implemented during the Unit 1 10th and Unit 2 7th refueling outages. All non-hardware changes were complete as of 05-03-90. See also APCo letters dated 09-18-87 and 12-29-88.

<u>GSI/ MPA No.</u>	<u>Title</u>	<u>Applicability</u>	<u>Status</u>	<u>Comments</u>
124	Auxiliary Feedwater System Reliability	ANO - 1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3, Ft. Calhoun	NA	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	C	Technical Specifications approved by License Amendment 26 dated 03-01-82 for Unit 1 and License Amendment 13 dated 05-05-82 for Unit 2. See also APCo letters dated 03-20-81, 05-28-81 and 07-06-84. See also License Amendments 55 and 46 dated 12-19-84 for Units 1 and 2.
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C	Technical Specifications approved by License Amendment 26 dated 03-01-82 for Unit 1 and Amendment 13 dated 05-05-82 for Unit 2. See also APCo letters dated 03-20-81, 05-28-81 and 07-06-84. See also License Amendments 55 and 46 dated 12-19-84 for Units 1 and 2.
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP - 1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	C (Unit 1) NC (Unit 2)	NUREG-0117 (SER Supplement 4) approved APCo's design (09-80) for both Units 1 and 2. Changes to Unit 2 were accomplished prior to issuance of its operating license. See also APCo letters dated 11-07-77, 01-15-79, 10-10-79, 12-11-79, 05-01-80 and 07-17-80.
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	

<u>GSI/ MPA No.</u>	<u>Title</u>	<u>Applicability</u>	<u>Status</u>	<u>Comments</u>
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Absorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C (Unit 1) NC (Unit 2)	Unit 2 original Technical Specifications included necessary changes. Unit 1 Technical Specifications were changed by order of the NRC on 04-20-81. See also APCo letter dated 03-24-80.