



# LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION

P.O. BOX 618, NORTH COUNTRY ROAD • WADING RIVER, N.Y. 11792

WILLIAM E. STEIGER, JR.

ASSISTANT VICE PRESIDENT-NUCLEAR OPERATIONS

JUN 14 1990

SNRC-1726

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

Long Island Lighting Company  
Response to Generic Letter 90-04  
Shoreham Nuclear Power Station - Unit 1  
Docket No. 50-322

Ref: (1) Generic Letter 90-04, "Request For Information On The Status Of Licensee Implementation Of Generic Safety Issues Resolved With Imposition Of Requirements Or Corrective Actions," April 25, 1990.

Gentlemen:

The Long Island Lighting Company (LILCO) has reviewed the implementation status of generic safety issues (GSIs) for which a final technical resolution has been reached and which are applicable to the Shoreham Nuclear Power Station (SNPS). Attachment 1 to this letter lists the GSIs included in Enclosure 1 of Generic Letter 90-04. This attachment was prepared in accordance with the guidelines in the Generic Letter and is LILCO's response to the reporting request in the Generic Letter.

Should you have any questions or require additional information, please do not hesitate to contact my office.

Very truly yours,

W. E. Steiger, Jr.  
Assistant Vice President  
Nuclear Operations

MP/ap  
Attachment

9006210018 900614  
PDR ADOCK 05000322  
P PNU

cc: S. Brown  
T. T. Martin  
L. Doerflein

A018  
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FACILITY NAME: Shoreham Nuclear Power Station<sup>(1)</sup>  
DOCKET NO.: 50-322  
LICENSEE: Long Island Lighting Company

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES  
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/ (MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u> <sup>(2)</sup>	<u>COMMENTS</u> <sup>(3) (4)</sup>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	NC	SSER-1 (9/81) open item 61; SNRC-659 (1/11/82); SNRC-703 (5/13/82); SNRC-767 (9/9/82); SNRC-768 (11/3/82); SNRC-772 (11/17/82); SNRC-900 (6/3/83); SSER-4 (9/83) item 61 resolved; SNPS EOPs written to Rev. 4 of the BWROG EPGs.
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NC	SER (4/81), section 4.6.2; Inspection Report 83-02, section 7.3, 7.4; Inspection Report 84-20, section 2.3.
43 (B107)	Reliability Of Air Systems	All Plants	I	SNRC-1552 (2/21/89), item IV.A.1 is open: the original air sampling performed indicates particulate size in excess of design. LILCO's investigation revealed that the prefilters and after filters had spacers which were comprised of the wrong material. These were replaced and the air was retested 3/1/90. Report from Pall Trincor due 6/30/90; item IV.A.3.b. is deferred: development of testing procedures.

NOTES

- (1) Full Power Operating License issued April 21, 1989.
- (2) "NC" indicates GSI completion pre-OL; "C" indicates GSI completion post-OL; "NA" indicates not applicable; "I" indicates not yet fully implemented.
- (3) Reference documents are identified regardless of "Status" entry.
- (4) SER (Safety Evaluation Report) and SSER (Supplement to Safety Evaluation Report) relate to NUREG-0420.

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51 (L913)	Improving The Reliability Of Open-Cycle Service Water Systems	All Plants	I	SNRC-590 (7/7/81); SNRC-682 (3/30/82); Inspection Report 83-28; SNRC-1665 (1/18/90); recommended actions of GL-89-13 have been indefinitely deferred.
67.3.3 (A017)	Improved Accident Monitoring	All Plants	I	USAR Appendix 3B, section 3B-1.97; SNRC-863 (4/14/83); SSER-1 (9/81) section 22; SNRC-1264 (6/6/86); SSER-10 (4/89) section 22; SNRC-1588 (4/18/89); SNRC-1621 (9/27/89); TMI I.D.2, II.B.3, and neutron flux monitoring instrumentation indefinitely deferred.
75 (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	NC	SNRC-1013 (3/9/84); SNRC-1116 (12/4/84); SSER-9 (12/85), section 15.3; SSER-10 (4/89), section 15.3.
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	I	SNRC-1013 (3/9/84); SNRC-1621 (9/27/89); installation of Phase II SPDS indefinitely deferred.
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	NC	SNRC-1013 (3/9/84); SNRC-1116 (12/4/84); SNRC-1184 (6/21/85); NRC letters dated 11/6/89 and 12/23/88; SSER-10 (4/89), section 15.3.
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	NC	SNRC-1013 (3/9/84); NRC letter dated 12/23/88; SSER-10 (4/89), section 15.3.
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	NC	SNRC-1013 (3/9/84); SNRC-1116 (12/4/84); NRC letter dated 12/23/88; SSER-10 (4/89), section 15.3.

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75 (B078)	Items 3.1.1 & 3.1.2 - Post-Maintenance Testing (Reactor Trip System Components)	All Plants	NC	SNRC-1012 (3/9/84); NRC letter dated 9/19/88; SSER-10 (4/89), section 15.3.
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	NC	SNRC-1013 (3/9/84); SNRC-1217 (12/6/85); NRC letter dated 4/9/86; SSER-10 (4/89), section 15.3.
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	NC	SNRC-1013 (3/9/84); NRC letter dated 9/19/88; SSER-10 (4/89), section 15.3.
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	NC	SNRC-1013 (3/9/84); SNRC-1217 (12/6/85); NRC letter dated 4/9/86; SSER-10 (4/89), section 15.3.
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	NC	SSER-10 (4/89), section 15.3.
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	NA	---
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	NA	---
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W & B&W Plants	NA	---

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<u>CSI/MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u> <sup>(2)</sup>	<u>COMMENTS</u> <sup>(3) (4)</sup>
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA	---
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	I	SNRC-1013 (3/9/84); SNRC-1116 (12/4/84); SNRC-1184 (6/21/85); SNRC-1264 (6/6/86); NRC letter dated 4/11/85; SNRC-1404 (12/29/87); SNRC-1621 (9/27/89); technical specification change for testing backup SCRAM valves during refueling outages indefinitely deferred.
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	NC	SNRC-1013 (3/9/84); SNRC-1184 (6/21/85); SNRC-1116 (12/4/84); SNRC-1404 (12/29/87); NRC letters dated 4/11/85 and 6/19/89; SSER-10 (4/89), section 15.3.
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NC	SNRC-955 (8/23/83); Inspection Report 84-21; SNRC-1059 (7/10/84); SNRC-1479 (7/28/88); SNRC-1530 (1/19/89).
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	NA	---
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	NA	---
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	NA	---

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A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	NC	Inspection Report 80-18; SER (4/81), section 3.9.1.1; SNRC-594 (7/15/81); SSER-1 (9/81), section 3.9.1.1; Inspection Report 82-04; Inspection Report 84-07; Inspection Report 84-14; Technical Specification 3/4.7.5.
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	NC	(see GSI A-13 (B017) entry).
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & LAMP-1	NA	---
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	NC	SER (4/81), sections 8.2, 8.4.2; SNRC-566 (5/15/81); SSER-1 (9/81) section 8.4.2.
B10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	---
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	---
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	NC	SER (4/81), open item 15, section 5.2.2; SNRC-577 (5/27/81); SSER-1 (9/81) item 15; SNRC-654 (1/6/82); SSER-3 (2/83), item 15, section 3.9.6.