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July 13, 1982

L. V. MAURIN
Vice President
Nuclear Operations

W3P82-1776
Q-3-B41

Mr. R. L. Tedesco
U.S. Nuclear Regulatory Commission
Assistant Director of Licensing
Washington, D.C. 20553

SUBJECT: WATERFORD 3 SES
DOCKET NO. 50-382
REACTOR COOLANT PUMP
SHEARED SHAFT (SSER3 OI#5)

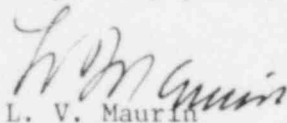
Dear Mr. Tedesco:

During a recent telephone conversation with Mr. Jack Rosenthal of your Instrumentation and Control Systems Branch on the subject of the Waterford 3 Reactor Coolant Pump Sheared Shaft Analysis he requested that we provide the following official confirmation.

The Reactor Coolant Pump Sheared Shaft Analysis presented in our response to FSAR Question 211.45 is based on a plant specific analysis using the steam generator pressure differential reactor trip system which we have committed to provide on Waterford 3 and which is described in our response to FSAR Question 211.105. The analysis is not based on the sequential tripping control concept used on CESSAR 80 plants.

We hope this allows ICSB to close out this open SER item. If you have any further questions, please do not hesitate to call.

Very truly yours,


L. V. Maurin

LVM/MJM/pco

cc: S. Black
J. Rosenthal

Boo!